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U.S. Nuclear Regulatory Commission  
ATTN: Document Control Desk  
Washington, DC 20555-0001

Braidwood Station, Unit 2  
Facility Operating License No. NPF-77  
NRC Docket No. STN 50-457

Subject: Core Operating Limits Report, Braidwood Unit 2 Cycle 19

The purpose of this letter is to transmit the Core Operating Limits Report (COLR) for Braidwood Unit 2 Cycle 19 in accordance with Technical Specification 5.6.5, "Core Operating Limits Report (COLR)." Braidwood Unit 2 Cycle 19 COLR, Revision 9 was implemented during Braidwood Unit 2 Refueling Outage 18 in support of Cycle 19 operation. Note that the revision number is based on a numbering convention that continues from the previous revision of the Unit 2 COLR, i.e., Braidwood Unit 2 Cycle 18, Revision 8.

If you have any questions regarding this matter, please contact Mr. Phillip Raush, Regulatory Assurance Manager, at (815) 417-2800.

Sincerely,

A handwritten signature in black ink, appearing to read "Mark Kanavos".

Mark Kanavos  
Site Vice President  
Braidwood Station

Attachment: Core Operating Limits Report (COLR) for Braidwood Unit 2 Cycle 19, Revision 9

cc: NRC Regional Administrator, Region III  
NRC Senior Resident Inspector – Braidwood Station  
NRC Project Manager, NRR – Braidwood and Byron Stations

CORE OPERATING LIMITS REPORT (COLR)

FOR

BRAIDWOOD UNIT 2 CYCLE 19

EXELON TRACKING ID:

COLR BRAIDWOOD 2 REVISION 9

CORE OPERATING LIMITS REPORT (COLR) for BRAIDWOOD UNIT 2 CYCLE 19

1.0 CORE OPERATING LIMITS REPORT

This Core Operating Limits Report (COLR) for Braidwood Station Unit 2 Cycle 19 has been prepared in accordance with the requirements of Technical Specification 5.6.5 (ITS).

The Technical Specification Safety Limits and Limiting Conditions for Operation (LCOs) affected by this report are listed below:

- SL 2.1.1 Reactor Core Safety Limits (SLs)
- LCO 3.1.1 SHUTDOWN MARGIN (SDM)
- LCO 3.1.3 Moderator Temperature Coefficient (MTC)
- LCO 3.1.4 Rod Group Alignment Limits
- LCO 3.1.5 Shutdown Bank Insertion Limits
- LCO 3.1.6 Control Bank Insertion Limits
- LCO 3.1.8 PHYSICS TESTS Exceptions – MODE 2
- LCO 3.2.1 Heat Flux Hot Channel Factor ( $F_Q(Z)$ )
- LCO 3.2.2 Nuclear Enthalpy Rise Hot Channel Factor ( $F_{\Delta H}^N$ )
- LCO 3.2.3 AXIAL FLUX DIFFERENCE (AFD)
- LCO 3.2.5 Departure from Nucleate Boiling Ratio (DNBR)
- LCO 3.3.1 Reactor Trip System (RTS) Instrumentation
- LCO 3.3.9 Boron Dilution Protection System (BDPS)
- LCO 3.4.1 Reactor Coolant System (RCS) Pressure, Temperature, and Flow Departure from Nucleate Boiling (DNB) Limits
- LCO 3.9.1 Boron Concentration

The portions of the Technical Requirements Manual (TRM) affected by this report are listed below:

- TRM TLCO 3.1.b Boration Flow Paths – Operating
- TRM TLCO 3.1.d Charging Pumps – Operating
- TRM TLCO 3.1.f Borated Water Sources – Operating
- TRM TLCO 3.1.g Position Indication System – Shutdown
- TRM TLCO 3.1.h Shutdown Margin (SDM) – MODE 1 and MODE 2 with  $k_{eff} \geq 1.0$
- TRM TLCO 3.1.i Shutdown Margin (SDM) – MODE 5
- TRM TLCO 3.1.j Shutdown and Control Rods
- TRM TLCO 3.1.k Position Indication System – Shutdown (Special Test Exception)

CORE OPERATING LIMITS REPORT (COLR) for BRAIDWOOD UNIT 2 CYCLE 19

2.0 OPERATING LIMITS

The cycle-specific parameter limits for the specifications listed in Section 1.0 are presented in the following subsections. These limits are applicable for the entire cycle unless otherwise identified. These limits have been developed using the NRC-approved methodologies specified in Technical Specification 5.6.5.

2.1 Reactor Core Safety Limits (SLs) (SL 2.1.1)

2.1.1 In MODES 1 and 2, the combination of Thermal Power, Reactor Coolant System (RCS) highest loop average temperature, and pressurizer pressure shall not exceed the limits specified in Figure 2.1.1.

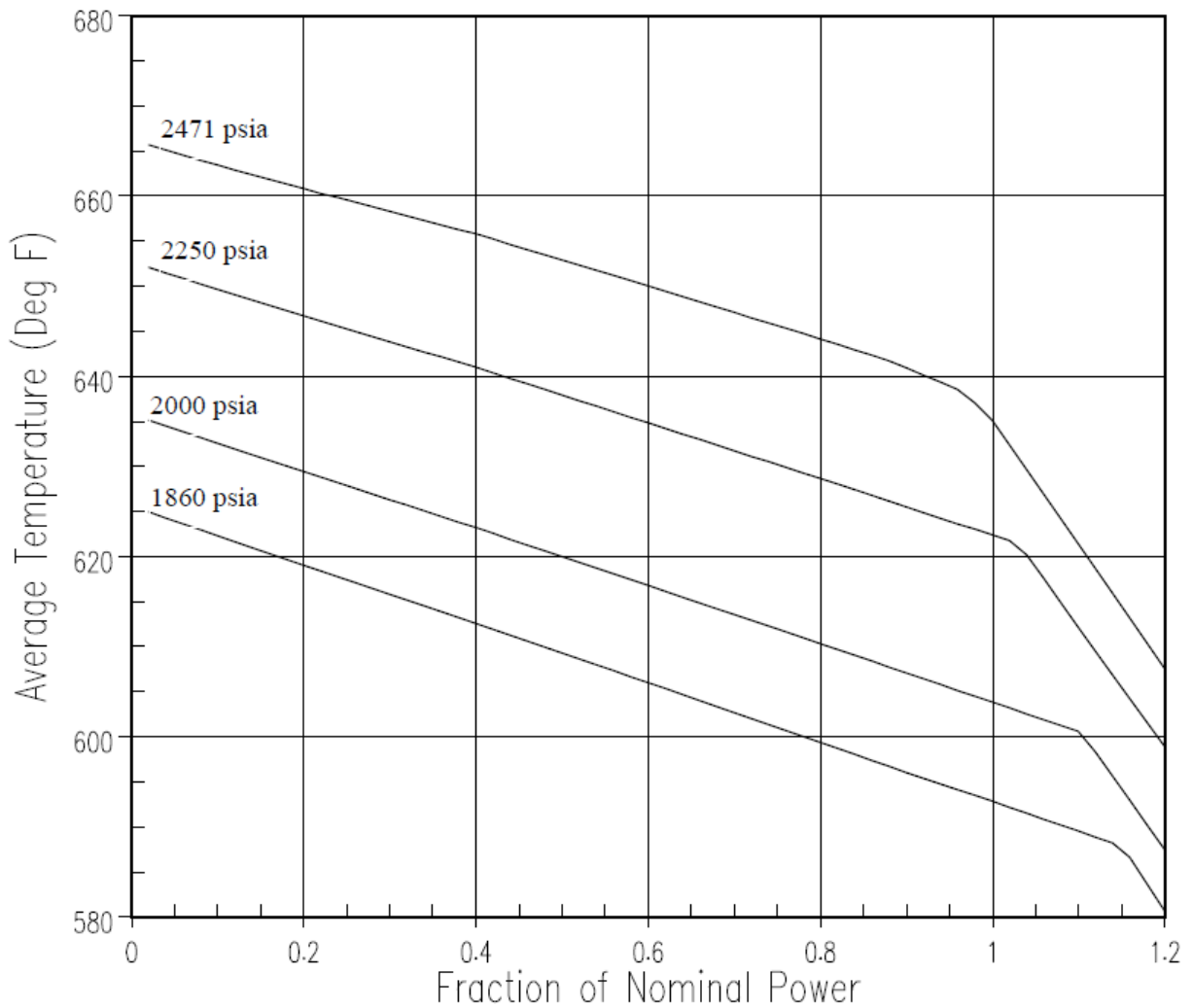


Figure 2.1.1: Reactor Core Limits

CORE OPERATING LIMITS REPORT (COLR) for BRAIDWOOD UNIT 2 CYCLE 19

2.2 SHUTDOWN MARGIN (SDM)

The SDM limit for MODES 1, 2, 3, and 4 is:

2.2.1 The SDM shall be greater than or equal to 1.3%  $\Delta k/k$  (LCOs 3.1.1, 3.1.4, 3.1.5, 3.1.6, 3.1.8, 3.3.9; TRM TLCOs 3.1.b, 3.1.d, 3.1.f, 3.1.h, and 3.1.j).

The SDM limit for MODE 5 is:

2.2.2 SDM shall be greater than or equal to 1.3%  $\Delta k/k$  (LCO 3.1.1, LCO 3.3.9; TRM TLCOs 3.1.i and 3.1.j).

2.3 Moderator Temperature Coefficient (MTC) (LCO 3.1.3)

The Moderator Temperature Coefficient (MTC) limits are:

2.3.1 The BOL/ARO/HZP-MTC upper limit shall be  $+2.296 \times 10^{-5} \Delta k/k/^\circ F$ .

2.3.2 The EOL/ARO/HFP-MTC lower limit shall be  $-4.6 \times 10^{-4} \Delta k/k/^\circ F$ .

2.3.3 The EOL/ARO/HFP-MTC Surveillance limit at 300 ppm shall be  $-3.7 \times 10^{-4} \Delta k/k/^\circ F$ .

2.3.4 The EOL/ARO/HFP-MTC Surveillance limit at 60 ppm shall be  $-4.3 \times 10^{-4} \Delta k/k/^\circ F$ .

where: BOL stands for Beginning of Cycle Life  
ARO stands for All Rods Out  
HZP stands for Hot Zero Thermal Power  
EOL stands for End of Cycle Life  
HFP stands for Hot Full Thermal Power

2.4 Shutdown Bank Insertion Limits (LCO 3.1.5)

2.4.1 All shutdown banks shall be fully withdrawn to at least 224 steps.

2.5 Control Bank Insertion Limits (LCO 3.1.6)

2.5.1 The control banks, with Bank A greater than or equal to 224 steps, shall be limited in physical insertion as shown in Figure 2.5.1.

2.5.2 Each control bank shall be considered fully withdrawn from the core at greater than or equal to 224 steps.

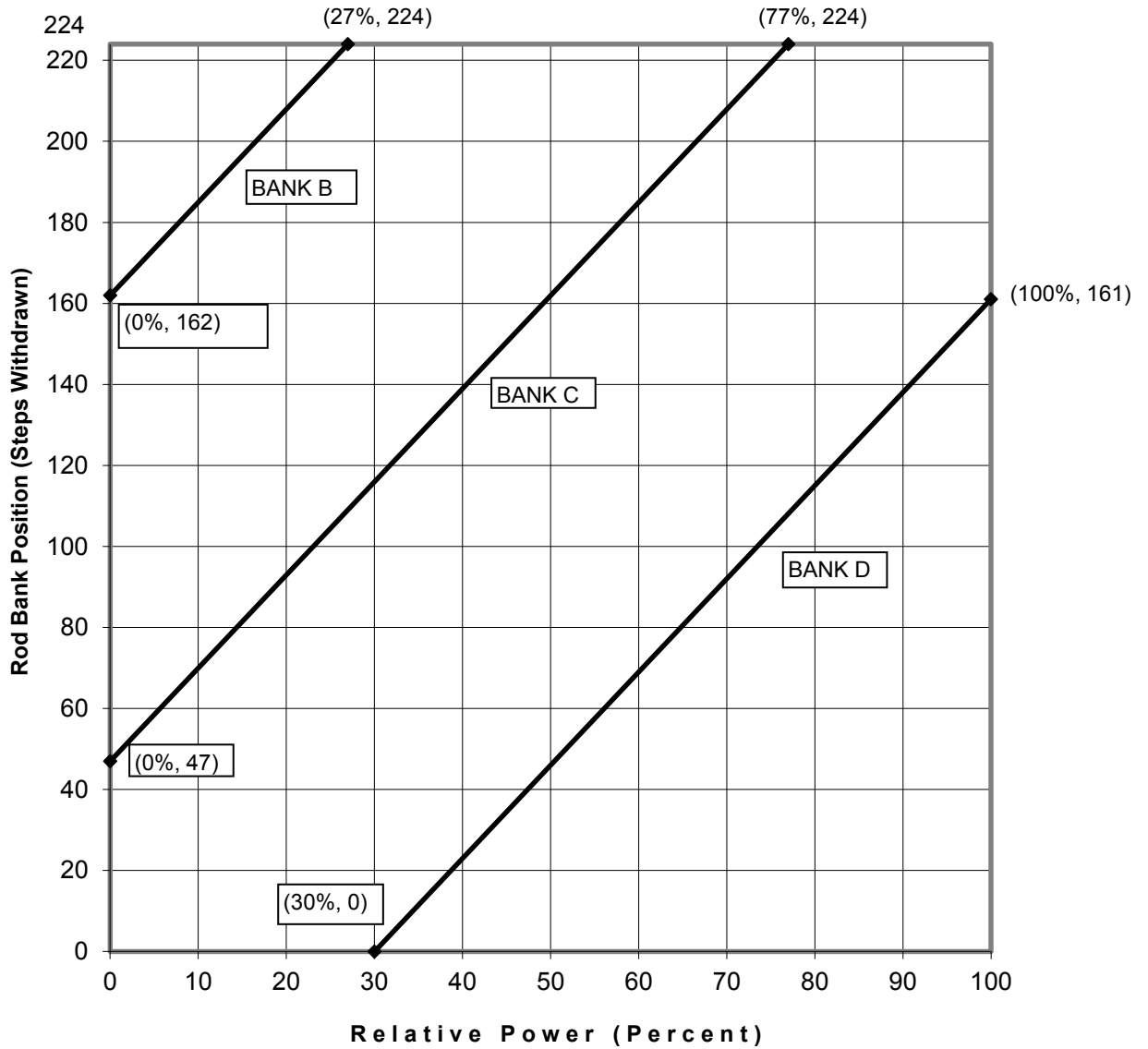
2.5.3 The control banks shall be operated in sequence by withdrawal of Bank A, Bank B, Bank C and Bank D. The control banks shall be sequenced in reverse order upon insertion.

2.5.4 Each control bank not fully withdrawn from the core shall be operated with the following overlap limits as a function of park position:

Park Position (step)	Overlap Limit (step)
226	111
227	112
228	113
229	114

CORE OPERATING LIMITS REPORT (COLR) for BRAIDWOOD UNIT 2 CYCLE 19

**Figure 2.5.1:  
Control Bank Insertion Limits Versus Percent Rated Thermal Power**



CORE OPERATING LIMITS REPORT (COLR) for BRAIDWOOD UNIT 2 CYCLE 19

2.6 Heat Flux Hot Channel Factor ( $F_Q(Z)$ ) (LCO 3.2.1)

2.6.1 Total Peaking Factor:

$$F_Q(Z) \leq \frac{F_Q^{RTP}}{0.5} \times K(Z) \text{ for } P \leq 0.5$$

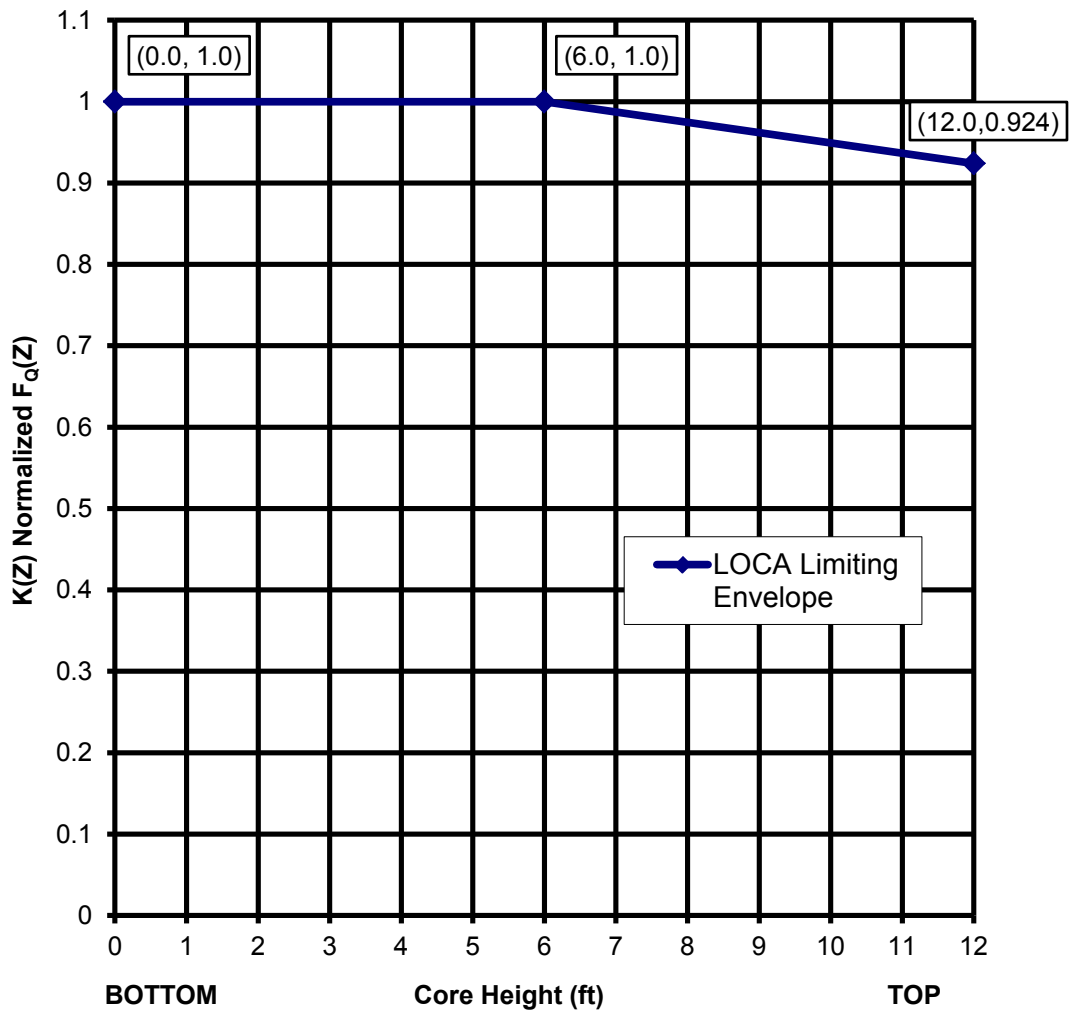
$$F_Q(Z) \leq \frac{F_Q^{RTP}}{P} \times K(Z) \text{ for } P > 0.5$$

where: P = the ratio of THERMAL POWER to RATED THERMAL POWER

$$F_Q^{RTP} = 2.60$$

K(Z) is provided in Figure 2.6.1.

**Figure 2.6.1**  
**K(Z) - Normalized  $F_Q(Z)$  as a Function of Core Height**



CORE OPERATING LIMITS REPORT (COLR) for BRAIDWOOD UNIT 2 CYCLE 19

2.6.2 W(Z) Values:

a) When the Power Distribution Monitoring System (PDMS) is OPERABLE,  $W(Z) = 1.00000$  for all axial points.

b) When PDMS is inoperable,  $W(Z)$  is provided as:

- 1) Table 2.6.2.a are the normal operation  $W(Z)$  values for the full cycle and correspond to the AXIAL FLUX DIFFERENCE (AFD) Acceptable Operation Limits provided in Figure 2.8.1.a. The normal operation  $W(Z)$  values have been determined at burnups of 150, 6000, 14000, and 20000 MWD/MTU.
- 2) The EOL-only normal operation  $W(Z)$  values provided in Table 2.6.2.b may be used for cycle burnups  $\geq 18000$  MWD/MTU. The EOL-only  $W(Z)$  values correspond to the REDUCED AXIAL FLUX DIFFERENCE (AFD) Acceptable Operation Limits provided in Figure 2.8.1.b. The EOL-only normal operation  $W(Z)$  values have been determined at burnups of 18000 and 20000 MWD/MTU and the last column of  $W(Z)$  values is a duplicate of the 20000 MWD/MTU values. If invoked, the EOL-only  $W(Z)$  values are to be used for the remainder of the cycle unless superseded by a subsequent analysis.

Table 2.6.2.c shows the  $F_{Q(z)}^C$  penalty factors that are greater than 2% per 31 Effective Full Power Days (EFPD). These values shall be used to increase the  $F_{Q(z)}^W$  as per Surveillance Requirement 3.2.1.2. A 2% penalty factor shall be used at all cycle burnups that are outside the range of Table 2.6.2.c.

2.6.3 Uncertainty:

The uncertainty,  $U_{FQ}$ , to be applied to the Heat Flux Hot Channel Factor  $F_Q(Z)$  shall be calculated by the following formula

$$U_{FQ} = U_{qu} \bullet U_e$$

where:

$U_{qu}$  = Base  $F_Q$  measurement uncertainty = 1.05 when PDMS is inoperable  
( $U_{qu}$  is defined by PDMS when OPERABLE.)

$U_e$  = Engineering uncertainty factor = 1.03

2.6.4 PDMS Alarms:

$F_Q(Z)$  Warning Setpoint = 2%  $F_Q(Z)$  Margin

$F_Q(Z)$  Alarm Setpoint = 0%  $F_Q(Z)$  Margin



CORE OPERATING LIMITS REPORT (COLR) for BRAIDWOOD UNIT 2 CYCLE 19

**Table 2.6.2.a**

**Full Cycle W(Z) versus Core Height for AFD Acceptable Operation Limits in Figure 2.8.1.a**  
(Top and Bottom 8% Excluded per WCAP-10216)

Height (feet)	150 MWD/MTU	6000 MWD/MTU	14000 MWD/MTU	20000 MWD/MTU
0.00 (core bottom)	1.3070	1.4779	1.4125	1.4005
0.20	1.2935	1.4452	1.3942	1.3797
0.40	1.2866	1.4387	1.4003	1.3715
0.60	1.2789	1.4258	1.3878	1.3623
0.80	1.2742	1.4024	1.3646	1.3449
1.00	1.2638	1.3684	1.3483	1.3311
1.20	1.2501	1.3528	1.3202	1.3130
1.40	1.2397	1.3367	1.3048	1.2982
1.60	1.2438	1.3242	1.2865	1.2795
1.80	1.2425	1.3042	1.2711	1.2624
2.00	1.2284	1.2825	1.2548	1.2443
2.20	1.2115	1.2573	1.2396	1.2270
2.40	1.1963	1.2339	1.2243	1.2150
2.60	1.1794	1.2075	1.2133	1.2004
2.80	1.1672	1.1882	1.2038	1.1865
3.00	1.1609	1.1836	1.1968	1.1718
3.20	1.1538	1.1770	1.1883	1.1761
3.40	1.1463	1.1725	1.1802	1.1822
3.60	1.1412	1.1664	1.1697	1.1862
3.80	1.1367	1.1619	1.1650	1.1901
4.00	1.1321	1.1553	1.1599	1.1921
4.20	1.1346	1.1477	1.1544	1.1926
4.40	1.1355	1.1402	1.1478	1.2017
4.60	1.1351	1.1306	1.1411	1.2094
4.80	1.1348	1.1220	1.1417	1.2150
5.00	1.1334	1.1124	1.1431	1.2185
5.20	1.1302	1.1018	1.1439	1.2205
5.40	1.1267	1.0927	1.1452	1.2221
5.60	1.1254	1.0908	1.1508	1.2395
5.80	1.1332	1.0972	1.1662	1.2549
6.00	1.1407	1.1030	1.1798	1.2662
6.20	1.1464	1.1106	1.1913	1.2747
6.40	1.1522	1.1173	1.1999	1.2794
6.60	1.1549	1.1229	1.2065	1.2801
6.80	1.1577	1.1276	1.2103	1.2799
7.00	1.1585	1.1327	1.2121	1.2748
7.20	1.1574	1.1384	1.2081	1.2657
7.40	1.1599	1.1447	1.2040	1.2547
7.60	1.1690	1.1490	1.1961	1.2388
7.80	1.1777	1.1532	1.1862	1.2229
8.00	1.1851	1.1565	1.1764	1.2056
8.20	1.1905	1.1633	1.1626	1.1877
8.40	1.1955	1.1713	1.1528	1.1799
8.60	1.1982	1.1780	1.1481	1.1714
8.80	1.2016	1.1883	1.1480	1.1664
9.00	1.2064	1.1979	1.1471	1.1610
9.20	1.2170	1.2118	1.1425	1.1673
9.40	1.2189	1.2195	1.1455	1.1990
9.60	1.2227	1.2279	1.1850	1.2440
9.80	1.2215	1.2326	1.2210	1.2820
10.00	1.2160	1.2352	1.2540	1.3170
10.20	1.2176	1.2445	1.2840	1.3470
10.40	1.2255	1.2551	1.3040	1.3720
10.60	1.2354	1.2641	1.3150	1.3950
10.80	1.2467	1.2737	1.3190	1.4130
11.00	1.2540	1.2816	1.3150	1.4240
11.20	1.2527	1.2903	1.2830	1.4130
11.40	1.2473	1.3007	1.2910	1.4010
11.60	1.2493	1.3055	1.2570	1.3842
11.80	1.2512	1.3107	1.2381	1.3762
12.00 (core top)	1.2567	1.3258	1.2269	1.3719

**Note:** W(Z) values at 20000 MWD/MTU may be applied to cycle burnups greater than 20000 MWD/MTU to prevent W(Z) function extrapolation

CORE OPERATING LIMITS REPORT (COLR) for BRAIDWOOD UNIT 2 CYCLE 19

Table 2.6.2.b EOL-only W(Z) versus Core Height for AFD Acceptable Operation Limits in Figure 2.8.1.b (Top and Bottom 8% Excluded per WCAP-10216)			
Height (feet)	18000 MWD/MTU	20000 MWD/MTU	25104 MWD/MTU
0.00 (core bottom)	1.2413	1.2338	1.2338
0.20	1.2270	1.2184	1.2184
0.40	1.2268	1.2119	1.2119
0.60	1.2189	1.2055	1.2055
0.80	1.2037	1.1927	1.1927
1.00	1.1933	1.1820	1.1820
1.20	1.1748	1.1688	1.1688
1.40	1.1635	1.1572	1.1572
1.60	1.1531	1.1437	1.1437
1.80	1.1457	1.1343	1.1343
2.00	1.1395	1.1269	1.1269
2.20	1.1313	1.1179	1.1179
2.40	1.1257	1.1089	1.1089
2.60	1.1260	1.1096	1.1096
2.80	1.1246	1.1098	1.1098
3.00	1.1239	1.1129	1.1129
3.20	1.1244	1.1191	1.1191
3.40	1.1265	1.1249	1.1249
3.60	1.1354	1.1415	1.1415
3.80	1.1462	1.1571	1.1571
4.00	1.1562	1.1707	1.1707
4.20	1.1659	1.1842	1.1842
4.40	1.1739	1.1957	1.1957
4.60	1.1803	1.2052	1.2052
4.80	1.1851	1.2127	1.2127
5.00	1.1891	1.2181	1.2181
5.20	1.1913	1.2205	1.2205
5.40	1.1931	1.2221	1.2221
5.60	1.2061	1.2395	1.2395
5.80	1.2222	1.2549	1.2549
6.00	1.2352	1.2662	1.2662
6.20	1.2453	1.2747	1.2747
6.40	1.2518	1.2794	1.2794
6.60	1.2551	1.2801	1.2801
6.80	1.2566	1.2799	1.2799
7.00	1.2542	1.2748	1.2748
7.20	1.2465	1.2657	1.2657
7.40	1.2376	1.2547	1.2547
7.60	1.2242	1.2388	1.2388
7.80	1.2098	1.2229	1.2229
8.00	1.1947	1.2056	1.2056
8.20	1.1769	1.1877	1.1877
8.40	1.1668	1.1799	1.1799
8.60	1.1590	1.1714	1.1714
8.80	1.1552	1.1664	1.1664
9.00	1.1509	1.1610	1.1610
9.20	1.1509	1.1673	1.1673
9.40	1.1697	1.1990	1.1990
9.60	1.2148	1.2440	1.2440
9.80	1.2545	1.2820	1.2820
10.00	1.2910	1.3170	1.3170
10.20	1.3226	1.3470	1.3470
10.40	1.3462	1.3720	1.3720
10.60	1.3642	1.3950	1.3950
10.80	1.3756	1.4130	1.4130
11.00	1.3791	1.4240	1.4240
11.20	1.3556	1.4130	1.4130
11.40	1.3520	1.4010	1.4010
11.60	1.3246	1.3842	1.3842
11.80	1.3099	1.3762	1.3762
12.00 (core top)	1.3004	1.3719	1.3719

**Note:** W(Z) values at 20000 MWD/MTU may be applied to cycle burnups greater than 20000 MWD/MTU to prevent W(Z) function extrapolation

CORE OPERATING LIMITS REPORT (COLR) for BRAIDWOOD UNIT 2 CYCLE 19

<b>Table 2.6.2.c Penalty Factors in Excess of 2% per 31 EFPD</b>	
<b>Cycle Burnup (MWD/MTU)</b>	<b>Penalty Factor <math>F^c_{q(z)}</math></b>
0	1.0200
490	1.0208
830	1.0400
1030	1.0450
1206	1.0470
1382	1.0480
1557	1.0478
1980	1.0465
3250	1.0218
3300	1.0200
13521	1.0200
13697	1.0204
13873	1.0236
14048	1.0237
14224	1.0232
14400	1.0226
14576	1.0220
14752	1.0213
14928	1.0205
15104	1.0200

Notes:

Linear interpolation is adequate for intermediate cycle burnups.

All cycle burnups outside the range of Table 2.6.2.c shall use a 2% penalty factor for compliance with the 3.2.1.2 Surveillance Requirements.

CORE OPERATING LIMITS REPORT (COLR) for BRAIDWOOD UNIT 2 CYCLE 19

2.7 Nuclear Enthalpy Rise Hot Channel Factor ( $F_{\Delta H}^N$ ) (LCO 3.2.2)

2.7.1  $F_{\Delta H}^N \leq F_{\Delta H}^{RTP} [1.0 + PF_{\Delta H}(1.0 - P)]$

where: P = the ratio of THERMAL POWER to RATED THERMAL POWER (RTP)

$$F_{\Delta H}^{RTP} = 1.70$$

$$PF_{\Delta H} = 0.3$$

2.7.2 Uncertainty:

The uncertainty,  $U_{F_{\Delta H}}$ , to be applied to the Nuclear Enthalpy Rise Hot Channel Factor  $F_{\Delta H}^N$  shall be calculated by the following formula:

$$U_{F_{\Delta H}} = U_{F_{\Delta H}m}$$

where:

$$U_{F_{\Delta H}m} = \text{Base } F_{\Delta H}^N \text{ measurement uncertainty} = 1.04 \text{ when PDMS is inoperable} \\ (U_{F_{\Delta H}m} \text{ is defined by PDMS when OPERABLE.})$$

2.7.3 PDMS Alarms:

$$F_{\Delta H}^N \text{ Warning Setpoint} = 2\% F_{\Delta H}^N \text{ Margin}$$

$$F_{\Delta H}^N \text{ Alarm Setpoint} = 0\% F_{\Delta H}^N \text{ Margin}$$

2.8 AXIAL FLUX DIFFERENCE (AFD) (LCO 3.2.3)

2.8.1 When PDMS is inoperable, the AXIAL FLUX DIFFERENCE (AFD) Acceptable Operation Limits are provided in the Figures described below or the latest valid PDMS Surveillance Report, whichever is more conservative.

a) Figure 2.8.1.a is the full cycle AFD Acceptable Operation Limits associated with the full cycle W(Z) values in Table 2.6.2.a.

b) Figure 2.8.1.b is the Reduced AFD Acceptable Operation Limits which may be applied after 18000 MWD/MTU. The Reduced AFD Acceptable Operation Limits are associated with the EOL-only W(Z) values in Table 2.6.2.b. Prior to changing to Figure 2.8.1.b, confirm that the plant is within the specified AFD envelope.

2.8.2 When PDMS is OPERABLE, no AFD Acceptable Operation Limits are applicable.

2.9 Departure from Nucleate Boiling Ratio (DNBR) (LCO 3.2.5)

2.9.1  $DNBR_{APSL} \geq 1.563$

The Axial Power Shape Limiting DNBR ( $DNBR_{APSL}$ ) is applicable with THERMAL POWER  $\geq 50\%$  RTP when PDMS is OPERABLE.

2.9.2 PDMS Alarms:

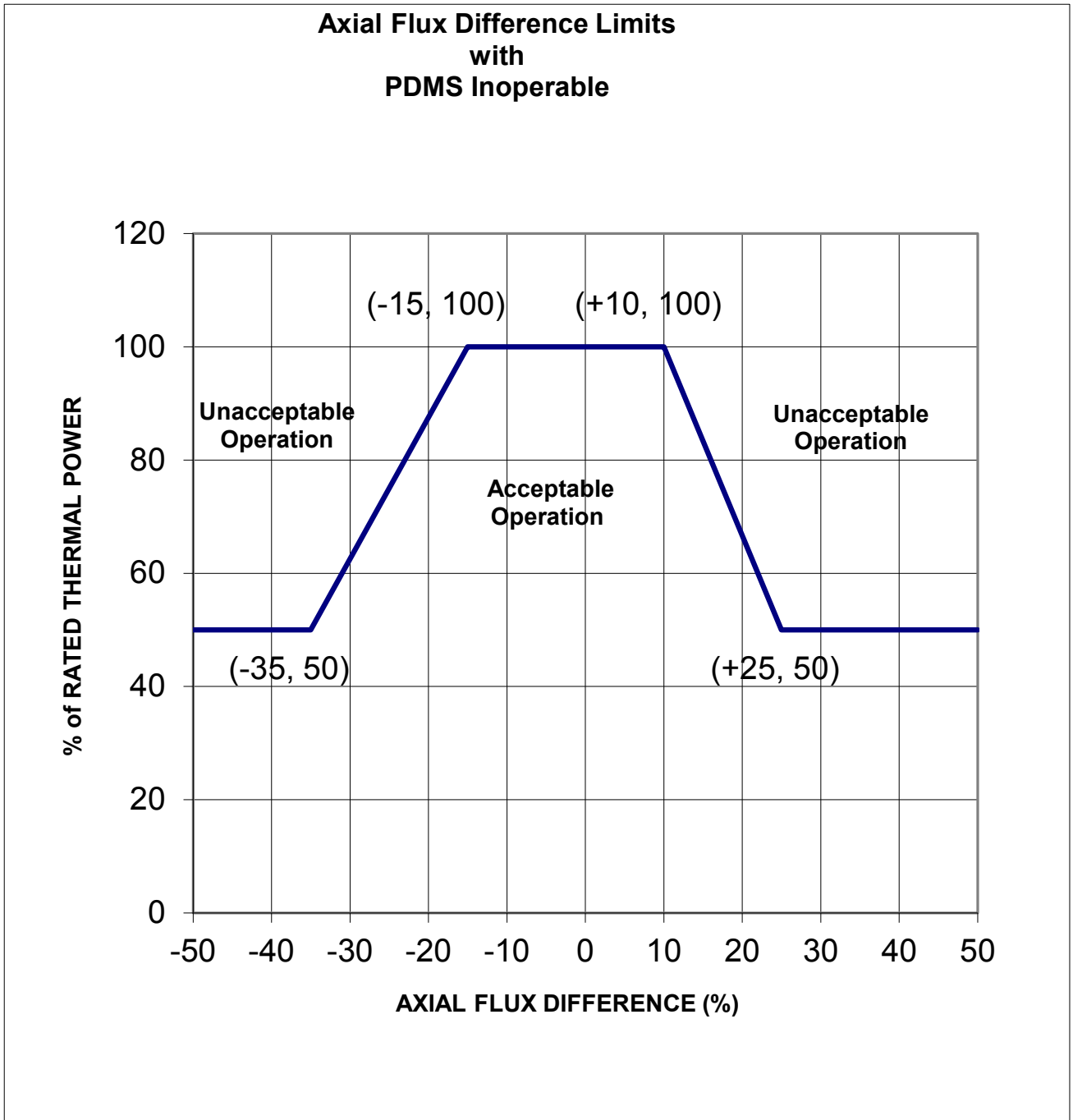
$$DNBR \text{ Warning Setpoint} = 2\% DNBR \text{ Margin}$$

$$DNBR \text{ Alarm Setpoint} = 0\% DNBR \text{ Margin}$$

CORE OPERATING LIMITS REPORT (COLR) for BRAIDWOOD UNIT 2 CYCLE 19

**Figure 2.8.1.a:  
Axial Flux Difference Limits  
as a Function of Rated Thermal Power**

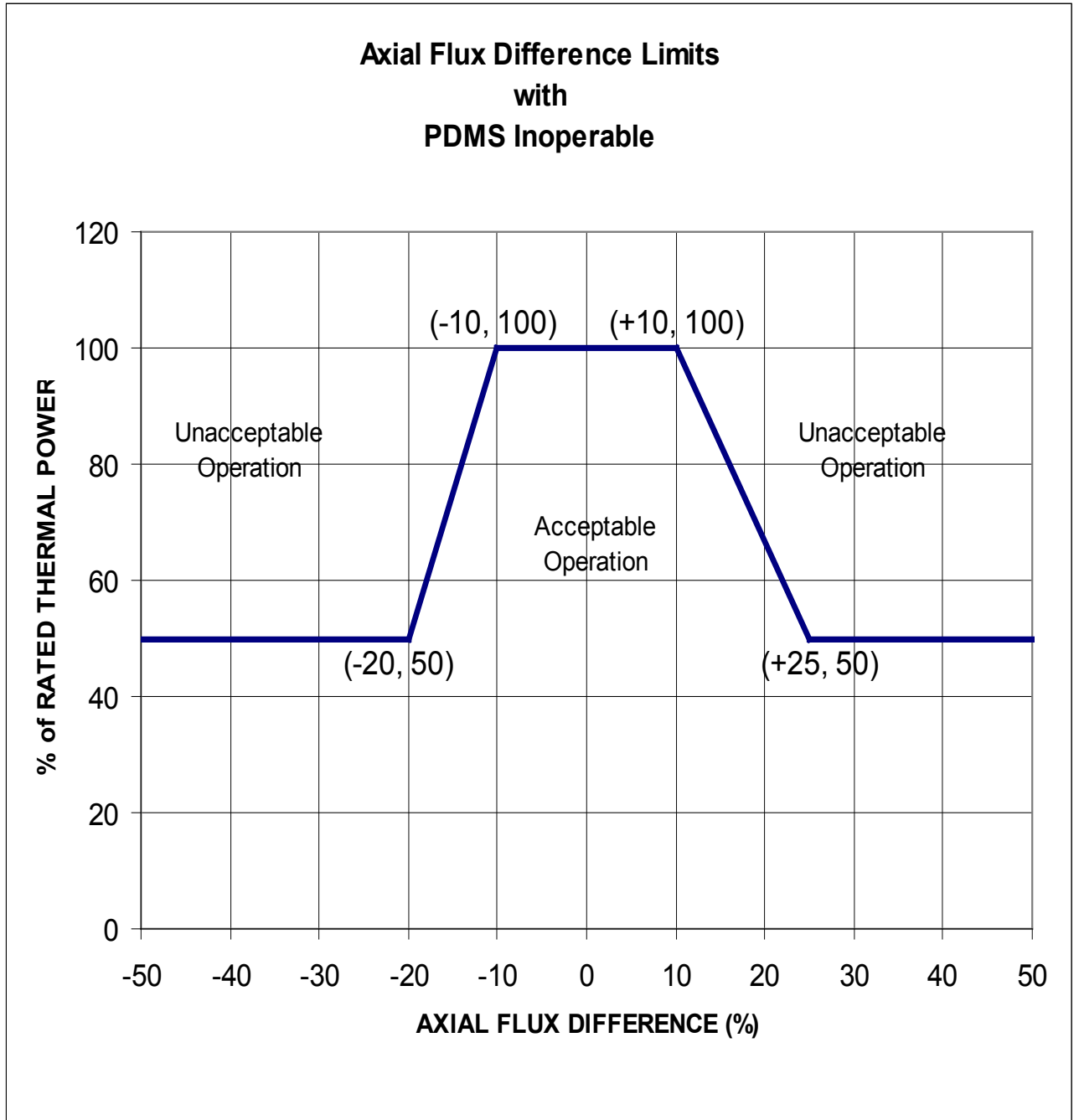
**(Full Cycle)**



CORE OPERATING LIMITS REPORT (COLR) for BRAIDWOOD UNIT 2 CYCLE 19

**Figure 2.8.1.b:  
Reduced Axial Flux Difference Limits  
as a Function of Rated Thermal Power**

**(Cycle burnup  $\geq 18000$  MWD/MTU)**



CORE OPERATING LIMITS REPORT (COLR) for BRAIDWOOD UNIT 2 CYCLE 19

2.10 Reactor Trip System (RTS) Instrumentation (LCO 3.3.1) - Overtemperature  $\Delta T$  Setpoint Parameter Values

- 2.10.1 The Overtemperature  $\Delta T$  reactor trip setpoint  $K_1$  shall be equal to 1.325.
- 2.10.2 The Overtemperature  $\Delta T$  reactor trip setpoint  $T_{avg}$  coefficient  $K_2$  shall be equal to 0.0297 / °F.
- 2.10.3 The Overtemperature  $\Delta T$  reactor trip setpoint pressure coefficient  $K_3$  shall be equal to 0.00135 / psi.
- 2.10.4 The nominal  $T_{avg}$  at RTP (indicated)  $T'$  shall be less than or equal to 588.0 °F.
- 2.10.5 The nominal RCS operating pressure (indicated)  $P'$  shall be equal to 2235 psig.
- 2.10.6 The measured reactor vessel  $\Delta T$  lead/lag time constant  $\tau_1$  shall be equal to 8 sec.
- 2.10.7 The measured reactor vessel  $\Delta T$  lead/lag time constant  $\tau_2$  shall be equal to 3 sec.
- 2.10.8 The measured reactor vessel  $\Delta T$  lag time constant  $\tau_3$  shall be less than or equal to 2 sec.
- 2.10.9 The measured reactor vessel average temperature lead/lag time constant  $\tau_4$  shall be equal to 33 sec.
- 2.10.10 The measured reactor vessel average temperature lead/lag time constant  $\tau_5$  shall be equal to 4 sec.
- 2.10.11 The measured reactor vessel average temperature lag time constant  $\tau_6$  shall be less than or equal to 2 sec.
- 2.10.12 The  $f_1(\Delta I)$  "positive" breakpoint shall be +10%  $\Delta I$ .
- 2.10.13 The  $f_1(\Delta I)$  "negative" breakpoint shall be -18%  $\Delta I$ .
- 2.10.14 The  $f_1(\Delta I)$  "positive" slope shall be +3.47% / %  $\Delta I$ .
- 2.10.15 The  $f_1(\Delta I)$  "negative" slope shall be -2.61% / %  $\Delta I$ .

CORE OPERATING LIMITS REPORT (COLR) for BRAIDWOOD UNIT 2 CYCLE 19

- 2.11 Reactor Trip System (RTS) Instrumentation (LCO 3.3.1) - Overpower  $\Delta T$  Setpoint Parameter Values
- 2.11.1 The Overpower  $\Delta T$  reactor trip setpoint  $K_4$  shall be equal to 1.072.
  - 2.11.2 The Overpower  $\Delta T$  reactor trip setpoint  $T_{avg}$  rate/lag coefficient  $K_5$  shall be equal to 0.02 / °F for increasing  $T_{avg}$ .
  - 2.11.3 The Overpower  $\Delta T$  reactor trip setpoint  $T_{avg}$  rate/lag coefficient  $K_5$  shall be equal to 0 / °F for decreasing  $T_{avg}$ .
  - 2.11.4 The Overpower  $\Delta T$  reactor trip setpoint  $T_{avg}$  heatup coefficient  $K_6$  shall be equal to 0.00245 / °F when  $T > T''$ .
  - 2.11.5 The Overpower  $\Delta T$  reactor trip setpoint  $T_{avg}$  heatup coefficient  $K_6$  shall be equal to 0 / °F when  $T \leq T''$ .
  - 2.11.6 The nominal  $T_{avg}$  at RTP (indicated)  $T''$  shall be less than or equal to 588.0 °F
  - 2.11.7 The measured reactor vessel  $\Delta T$  lead/lag time constant  $\tau_1$  shall be equal to 8 sec.
  - 2.11.8 The measured reactor vessel  $\Delta T$  lead/lag time constant  $\tau_2$  shall be equal to 3 sec.
  - 2.11.9 The measured reactor vessel  $\Delta T$  lag time constant  $\tau_3$  shall be less than or equal to 2 sec.
  - 2.11.10 The measured reactor vessel average temperature lag time constant  $\tau_6$  shall be less than or equal to 2 sec.
  - 2.11.11 The measured reactor vessel average temperature rate/lag time constant  $\tau_7$  shall be equal to 10 sec.
  - 2.11.12 The  $f_2(\Delta I)$  "positive" breakpoint shall be 0 for all  $\Delta I$ .
  - 2.11.13 The  $f_2(\Delta I)$  "negative" breakpoint shall be 0 for all  $\Delta I$ .
  - 2.11.14 The  $f_2(\Delta I)$  "positive" slope shall be 0 for all  $\Delta I$ .
  - 2.11.15 The  $f_2(\Delta I)$  "negative" slope shall be 0 for all  $\Delta I$ .



CORE OPERATING LIMITS REPORT (COLR) for BRAIDWOOD UNIT 2 CYCLE 19

2.12 Reactor Coolant System (RCS) Pressure, Temperature, and Flow Departure from Nucleate Boiling (DNB) Limits (LCO 3.4.1)

2.12.1 The pressurizer pressure shall be greater than or equal to 2209 psig.

2.12.2 The RCS average temperature ( $T_{avg}$ ) shall be less than or equal to 593.1 °F.

2.12.3 The RCS total flow rate shall be greater than or equal to 386,000 gpm.

2.13 Boron Concentration

2.13.1 The refueling boron concentration shall be greater than or equal to the applicable value given in the Table below (LCO 3.9.1). The reported “prior to initial criticality” value also bounds the end-of-cycle requirements for the previous cycle.

2.13.2 To maintain  $keff \leq 0.987$  with all shutdown and control rods fully withdrawn in MODES 3, 4, or 5 (TRM TLCO 3.1.g Required Action B.2 and TRM TLCO 3.1.k.2), the Reactor Coolant System boron concentration shall be greater than or equal to the applicable values given in the Table below.

COLR Section	Conditions	Boron Concentration (ppm)
2.13.1	a) prior to initial criticality	1671
	b) for cycle burnups $\geq 0$ MWD/MTU and $< 16000$ MWD/MTU	1809
	c) for cycle burnups $\geq 16,000$ MWD/MTU	1452
2.13.2	a) prior to initial criticality	1738
	b) all other times in life	1995