Question: 76

Given the following:

- Unit 1 is cooling down for refueling outage.
- RCS Tave is 349 °F.
- RCS Pressure is 360 psig.

Which ONE of the following correctly states:

1) The **maximum** number of charging pumps that are allowed per Tech Specs 3.1.G, RCS Overpressure Mitigation for sustained operation.
2) The basis for that requirement?

A. 1. **one** charging pump capable of injecting into the RCS.
2. To ensure that the mass addition transient, can be relieved by the operation of a **single** PORV or equivalent.

B. 1. **one** charging pump capable of injecting into the RCS.
2. To ensure that the mass addition transient, can be relieved by the operation of **both** PORVs, or equivalent.

C. 1. **two** charging pumps capable of injecting into the RCS.
2. To ensure that the mass addition transient, can be relieved by the operation of **both** PORVs, or equivalent.

D. 1. **two** charging pumps capable of injecting into the RCS.
2. To ensure that the mass addition transient can be relieved by the operation of a **single** PORV, or equivalent.

Question: 77

Unit 1 initially operating at 100% power when a reactor trip occurs due to "A" reactor trip breaker spuriously opening.

On transition to ES-0.1, The RO reports the following:

- Annunciator 1D-E5, CHG PP TO REGEN HX HI/LO FLOW, has alarmed.
- Charging flow indicates 25 gpm.
- Letdown flow is 0 gpm.
- RO identified 1-CH-TV-1204A, Inside Letdown Trip Valve, is closed and cannot be re-opened.

Which ONE of the following states:

1) The operating Team used _____ to close and de-activate 1-CH-TV-1204B, Outside Letdown Trip Valve.

2) The Letdown penetration is _____.

A. 1) 1-OPT-CT-306, Containment Integrity Verification
   2) Inoperable (4 hour clock in effect)

B. 1) 1-OPT-CT-306, Containment Integrity Verification
   2) Operable (Exit the 4 hour clock)

C. 1) 0-OP-CT-001, Containment Isolation Barriers
   2) Inoperable(4 hour clock in effect)

D. 1) 0-OP-CT-001, Containment Isolation Barriers
   2) Operable (Exit the 4 hour clock)

Question: 78

An entry into 1-FR-S.1, Response to Nuclear Power Generation ATWS was made from step 1 of E-0, Reactor Trip or SI.

The following conditions currently exist:

- The reactor trip breakers are closed.
- Rods are being inserted manually.
- Control Bank D is at 24 steps.
- Power range instruments are all indicating 8%.
- Intermediate Range SUR is negative.
- An operator has just been dispatched to open reactor trip breakers per step 8 of 1-FR-S.1.

Which of the following conditions is required by 1-FR-S.1 to allow a return to 1-E-0?

A. Both of the RPS MG Set output breakers must be opened.

B. Both of the reactor trip breakers must be opened.

C. Power range indication must be reduced below 5%.

D. All rods must be fully inserted.

Question: 79

Initial Conditions

Unit 1 is operating at 100% power.

- All control channels have been aligned to Channel IV in preparation for Channel III process PTs.
- Turbine First Stage Pressure, 1-MS-PT-1446, fails low.
- The Team has completed the Immediate Actions of 0-AP-53.00, Loss of Vital Instrumentation/Controls.

Which ONE of the following states:

1) Rod Control _____ required to be placed in Manual in accordance with 0-AP-53.00.
2) In accordance with Tech Spec 3.7-1, Item 20, Operator Action 13; 1-OPT-RP-001, Check of Permissive Status Lights P-6, P-7, P-8, and P-10, _____ required to be completed.

REFERENCE PROVIDED

A. 1) is not
   2) is

B. 1) is not
   2) is not

C. 1) is
   2) is

D. 1) is
   2) is not

Question: 80

Initial Conditions for Unit 1:

- Unit 1 at 100% power.
- Delta Flux is at -2.7% with a target of -1%.
- Spurious Instrument failure causes rods to insert.

Current Conditions:

- Reactor Power is 89% and stable.
- Delta Flux is at -15%.
- Tave is 571.5 °F, Tref is 571.0 °F.
- Annunciator 1E-E3, Delta Flux Deviation is lit.
- Annunciator 1G-G8, Rod Bank D Lo Limit is lit.

Based on the current conditions, which ONE of the following states:

1) The **next** action required to be taken to comply with Technical Specifications.
2) The **most restrictive** Tech. Spec basis for this CONDITION.

(REFERENCE PROVIDED)

A. 1) Restore Delta flux to target band within 1-hour or reduce power to < 50% in 1-hour.
   2) Minimize the effects of Xenon redistribution during load-follow maneuvers.

B. 1) Restore Delta flux to target band within 1-hour or reduce power to < 50% in 1-hour.
   2) Provide a limit on rod worth in the event of a rod ejection accident.

C. 1) Reduce reactor power to < 50% within 30 minutes.
   2) Minimize the effects of Xenon redistribution during load-follow maneuvers.

D. 1) Reduce reactor power to < 50% within 30 minutes.
   2) Provide a limit on rod worth in the event of a rod ejection accident.

Question: 81

Initial Conditions:

- Unit 1 and Unit 2 operating at 100% power.
- Unit 2 is performing 2-PT-41.1, CC Pump Performance.
- CC is **split out** in the Turbine Building.
- 1-CC-E-1A, "A" CC HX, has been isolated due to a through wall SW leak.

Current Conditions:

- Annunciators 1B-A7 and 1B-B7, Channel 1 and Channel 2, CTMT PART +.1 PSI are received.
- CTMT pressure is 10.6 PSIA and rising slowly.
- The operating Team is performing ARP 1B-A7, and has raised SW flow to the "C" CC Heat Exchanger.

Which ONE of the following describes:

1) The effect on Unit 1 Containment Temperature.
2) The basis of the sloped line from 70 °F to 100 °F on the Containment Allowable Air Partial Pressure VS. Service Water Temperature Curve (Figure TS-3.8-1) is _____.

(REFERENCE PROVIDED)

A. 1) No change.
   2) MSLB Peak Calculated Pressure

B. 1) Lower.
   2) LOCA depressurization

C. 1) No change.
   2) LOCA depressurization

D. 1) Lower.
   2) MSLB Peak Calculated Pressure

Question: 82

Given the following:

- Unit 1 is ramping down in accordance with 0-AP-23.00, Rapid Load Reduction.
- Emergency Boration has been initiated.
- Emergency Borate Flow, 1-CH-FI-1110, rises to 80 gpm then drops to 0 gpm indicated flow.
- Emergency Borate MOV, 1-CH-MOV-1350, indicates intermediate.
- "A" Boric Acid Transfer Pump, 1-CH-P-2A, "FAST" light is LIT.
- "A" Boric Acid Storage Tank level, LI-1-106/LI-1-161, indicate 93% and stable.

Which ONE of the following states:

1) The cause for the change Emergency Borate Flow, 1-CH-FI-1110.
2) Are the minimum Boric Acid flowpaths OPERABLE?

A. 1) 1-CH-P-2A shaft has sheared.
   2) No.

B. 1) 1-CH-P-2A shaft has sheared.
   2) Yes.

C. 1) 1-CH-FI-1110, Emergency Borate Flow has failed low.
   2) No.

D. 1) 1-CH-FI-1110, Emergency Borate Flow has failed low.
   2) Yes.

Question: 83

Initial Conditions:

- The reactor has been tripped and SI initiated due to a Large Break LOCA inside Containment.
- "A" Containment Spray pump tripped on startup.
- "A" Outside Recirc Spray pump (ORS) discharge pressure and amp indications are fluctuating.

Which ONE of the following identifies:

1) The operator _____ required to **locally** open the supply breaker after placing the "A" ORS pump in PTL.
2) The required procedural flow path is 1-E-0, Reactor Trip or SI; 1-E-1, Loss of Reactor or Secondary Coolant; and _____.

A. 1) is
   2) 1-ES-1.2, Post LOCA Cooldown and Depressurization

B. 1) is not
   2) 1-ES-1.3, Transfer to Cold Leg Recirculation

C. 1) is not
   2) 1-ES-1.2, Post LOCA Cooldown and Depressurization

D. 1) is
   2) 1-ES-1.3, Transfer to Cold Leg Recirculation

Question: 84

Initial Conditions:

- Unit 1 is operating at 100%.
- 1-CC-TV-105B, RCP "B" CLR CC RTN TV, fails closed.
- Annunciator 1C-B1, RCP 1B CC RETURN LO FLOW alarms.
- The crew enters 1-AP-9.00, RCP Abnormal Conditions.

Current Conditions:

- The following additional annunciators are Lit.
  - 1C-G4, RCP FRAME ALERT.
- RCP 'B' parameter changes over the last 5 minutes

Parameter	Initial reading	Current reading
Frame Vibration	1.6 mils	2.9 mils and stable
Shaft Vibration	5.5 mils	10.8 mils and stable

Which ONE of the following answers the questions below:

1) Is it required at this time to remove the unit from service and secure the 'B' RCP in accordance with 1-AP-9.00, RCP Abnormal Conditions?
2) What is the Tech Spec Bases for the most limiting LCO that is entered?

A. 1) No.
   2) Provide cooling water for the removal of residual and sensible heat from the Reactor Coolant system, cool the containment recirculation air coolers, and the reactor coolant pump motor coolers.

B. 1) No.
   2) Ensures the containment atmosphere will be isolated from the outside environment in the event of a release of radioactive material to the containment atmosphere or pressurization of the containment.

C. 1) Yes.
   2) Provide cooling water for the removal of residual and sensible heat from the Reactor Coolant system, cool the containment recirculation air coolers, and the reactor coolant pump motor coolers.

D. 1) Yes.
   2) Ensures the containment atmosphere will be isolated from the outside environment in the event of a release of radioactive material to the containment atmosphere or pressurization of the containment.

Question: 85

Unit 1 is operating at 100% power; Unit 2 is in Refueling shutdown.

- The eighth fuel assembly has just been lifted into the Manipulator Crane when it is dropped, and bubbles begin issuing from the assembly.
- Containment is open.
- 1-VS-F-42, Relay Room Emergency Supply Fan, is tagged out for maintenance.

Which ONE of the following identifies:

1) In accordance with 0-AP-22.00, Fuel Handling Abnormal Conditions, Containment Closure is required to be set within _____ minutes.
2) In accordance with TS 3.21 Basis, the Minimum number of OPERABLE trains of the Control Room Emergency Ventilation System is _____.

A. 1) 45
   2) 3

B. 1) 60
   3) 3

C. 1) 45
   2) 2

D. 1) 60
   2) 2

Question: 86

Initial Conditions:

With the plant at 100% power, a loss of all feedwater occurs.

Current Conditions:

- The crew is NOT successful at tripping the reactor manually.
- The reactor fails to trip automatically.
- The crew enters FR-S.1, RESPONSE TO NUCLEAR POWER GENERATION - ATWS.

In accordance with the FR-S.1 background document which ONE of the following states:

1) The **maximum** amount of time for tripping the turbine to prevent unacceptable consequences.
2) The **basis** for tripping the main turbine shortly after the onset of the event.

A. 1) 30 seconds.
   2) Shut down the reactor by allowing the RCS to heat up.

B. 1) 60 seconds.
   2) Shut down the reactor by allowing the RCS to heat up.

C. 1) 60 seconds.
   2) Prevent the RCS from exceeding its RCS pressure limit.

D. 1) 30 seconds.
   2) Prevent the RCS from exceeding its RCS pressure limit.

Question: 87

Initial Conditions:

- Unit 1 is operating at 100% power.
- "A" charging pump, 1-CH-P-1A, running.
- Breaker 25J3, EDG #3 Output breaker to 2J bus, tagged out for breaker PMs.
- "A" RSST Pilot Wire Lockout followed one (1) minute later by "C" RSST Pilot Wire Lockout occurs due to Maintenance Crew error in the Switchyard.

Which ONE of the following identifies:

1) The _____ and "B" Charging Pumps are running on Unit 1.
2) The non-running charging pump is _____.

A. 1) "A"
   2) Operable

B. 1) "A"
   2) Inoperable

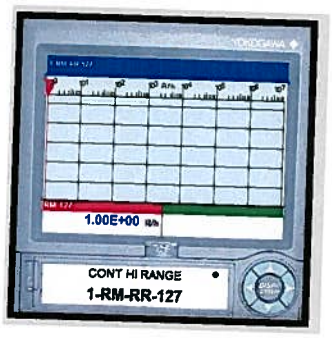
C. 1) "C"
   2) Operable

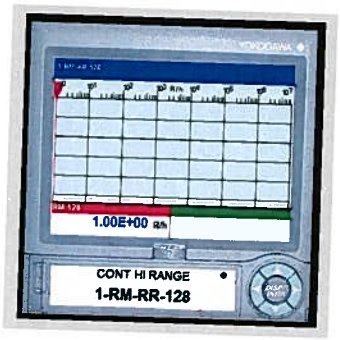
D. 1) "C"
   2) Inoperable

Question: 88

Initial Conditions:

- Refueling operations in progress for Unit 1.
- Annunciator 1-RMA-A1, Unit 1 CTMT HI RAD ALERT/TRBL alarms.
- Recorder traces for 1-RM-RR-127, and 1-RM-128 are shown below.





Current Conditions:

- The operator depresses the SAFE/RESET pushbutton to attempt to reset the alarm.
- There is no change from the initial conditions.

Which ONE of the following describes:

1) The actions that must be performed in accordance with 1-RMA-A1, Unit 1 CTMT HI RAD ALERT/TRBL?
2) The Technical specification basis for this Radiation Monitor?

A. 1) Stop refueling, notify HP, and evacuate Unit 1 containment.
   2) Provide information to the operators that will enable them to determine the likelihood of a gross breach of barriers to radioactive release.

B. 1) Review Technical Specifications and monitor activity using the redundant monitor.
   2) Provide information to the operators that will enable them to determine the likelihood of a gross breach of barriers to radioactive release.

C. 1) Stop refueling, notify HP, and evacuate Unit 1 containment.
   2) Provide a sensitive indication of primary system leakage.

D. 1) Review Technical Specifications and monitor activity using the redundant monitor.
   2) Provide a sensitive indication of primary system leakage.

Question: 89

Tech Specs Section 4.6, Emergency Power System Periodic Testing, states that a minimum fuel oil storage of 35,000 gallons shall be maintained on-site.

Concerning the basis for this capacity, which ONE of the following states:

1) The number of EDG(s) running at full power assumed?
2) The time elapsed before the fuel supply is exhausted?

A. 1) Two (2).
   2) 1 Day.

B. 1) Two (2).
   2) 7 Days.

C. 1) One (1).
   2) 1 Day.

D. 1) One (1).
   2) 7 Days.

Question: 90

Given the following:

- Team is responding to a SGTR on Unit 2 "B" S/G following a sustained loss of off-site power.
- Team has transitioned to 2-E-3, Steam Generator Tube Rupture.
- Annunciator 2B-E6, IA LO HDR PRESS / IA COMPR1 TRBL has alarmed.
- Instrument air pressure on PI-IA-100 is 50 psig and lowering rapidly.
- Temporary Diesel Air compressor will not start.
- Annunciator 2D-C6, PRZR PWR RELIEF VV LO AIR PRESSURE is not LIT.

Team is ready to perform RCS cooldown.

Which ONE of the following correctly describes:

1) The initial actions for RCS cooldown in accordance with 2-E-3.
2) The post-SGTR procedure to use following initial cooldown.

A. 1) Cooldown by dumping steam from intact S/Gs via local operation of the S/G PORVs.
   2) 2-ES-3.1, POST SGTR COOLDOWN USING BACKFILL.

B. 1) Cooldown by dumping steam from intact S/Gs via local operation of the steam dumps.
   2) 2-ES-3.1, POST SGTR COOLDOWN USING BACKFILL.

C. 1) Cooldown by dumping steam from intact S/Gs via local operation of the S/G PORVs.
   2) 2-ES-3.2, POST SGTR COOLDOWN USING BLOWDOWN.

D. 1) Cooldown by dumping steam from intact S/Gs via local operation of the steam dumps.
   2) 2-ES-3.2, POST SGTR COOLDOWN USING BLOWDOWN.

Question: 91

Unit 1 has experienced a LBLOCA coincident with a loss of off-site power.

Initial Conditions:

- 4160 V Emergency bus 1H is de-energized due to a fault.
- CTMT pressure is 55 psia and slowly rising.
- 1-CS-P-1B, "B" CS pump tripped on start.
- The Team has performed 1-FR-Z.1, Response to Containment High Pressure, and returned to 1-E-1, Loss of Reactor or Secondary Coolant.

Current Conditions:

- CTMT Pressure 58 psia and stable.

Which ONE of the following states:

1) The procedure and Path Color in effect.
2) Based upon Current Conditions, does 1-FR-Z.1 need to be implemented again?

A. 1) FR-Z.1, Response to Containment High Pressure; RED Path.
   2) Yes.

B. 1) FR-Z.1, Response to Containment High Pressure; RED Path.
   2) No.

C. 1) FR-Z.1, Response to Containment High Pressure; ORANGE Path.
   2) Yes.

D. 1) FR-Z.1, Response to Containment High Pressure; ORANGE Path.
   2) No.

Question: 92

Given the following conditions:

- Unit 1 is at 100% power when a fire occurs in the Unit 1 Emergency Switchgear Room.
- The emergency switchgear Fire Suppression system could NOT be manually actuated.
- The team entered 1-FCA-4.00, Limiting ESGR Number 1 Fire.
- The team has placed the MS PRESS CONT VLV FIRE EMERG CLOSE switches to EMERG CLOSE position for the SG PORVs.
- The fire has resulted in a loss of all three transfer busses.
- Condenser Waterbox Vacuum breakers have been opened.

Which one of the following states:

1) The correct method available for controlling the RCS cooldown in accordance with 0-FCA-17.00.
2) The maximum amount of time for restoring the Fire Suppression system to a FUNCTIONAL status per the TRM.

(REFERENCE PROVIDED)

A. 1) Operate the SG PORVs from the MCR.
   2) 14 days.

B. 1) Operate the SG PORVs from the MCR.
   2) 7 days.

C. 1) Locally operate the SG PORVs using the air bottles.
   2) 14 days.

D. 1) Locally operate the SG PORVs using the air bottles.
   2) 7 days.

Question: 93

The Unit is operating at 100% when the following events occur:

- 1236 #2 EDG is started for Monthly Performance Test.
- 1330 Heavy black smoke is reported coming from #2 EDG Engine Control Cabinet.
- 1333 #2 EDG tripped.
- 1340 An alert is declared based on Tab HA2.1, Fire or explosion affecting the operability of structures, systems, or components required to establish or maintain Safe Shutdown.

Which ONE of the following identifies:

1) The **first** method used to notify Plant personnel to report to their Emergency Assembly Area(s) in accordance with EPIP-1.01.
2) The maximum time allowed for Security to report initial results of Accountability of Plant personnel.

A. 1) Contact Station Personnel with Beepers.
   2) 15 minutes.

B. 1) Contact Station Personnel with Beepers.
   2) 30 minutes.

C. 1) Station Emergency Manager announcement using Gai-tronics.
   2) 15 minutes.

D. 1) Station Emergency Manager announcement using Gai-tronics.
   2) 30 minutes.

Question: 94

Preparations are in progress to conduct a Reactor startup following a refueling outage for Unit 2. The operations and support personnel have assembled to conduct a Pre-job briefing for 2-NPT-RX-008, Startup Physics Testing (ICCE II).

Which ONE of the following correctly describes the requirements per OP-AA-106, Infrequently Conducted or Complex Evolutions, Attachment 3 – Management Expectations Briefing checklist, for covering the expectations regarding "**the need for open communication**"?

1) The individual must have a position of _____ or above.
2) The individual must be qualified at a minimum as _____.

A. 1) Test Coordinator
   2) Current Reactor Engineer

B. 1) Test Coordinator
   2) Senior Reactor Operator, past or present

C. 1) Second Line Supervisor
   2) Senior Reactor Operator, past or present

D. 1) Second Line Supervisor
   2) Current Reactor Engineer

Question: 95

Initial Conditions:

- The reactor is operating at 100% power.
- An RCS Leak Rate is in progress in accordance with 1-OPT-RC-10.0, Reactor Coolant Leakage – Computer Calculated.

Current Conditions:

- The Reactor Trips due to a loose wire in the "A" reactor trip breaker.
- The Team is stabilizing the Unit in accordance with ES-0.1, Reactor Trip Response.

Which of the following identifies:

1) In accordance with OP-AP-105, Post Trip Review, the minimum authority who must authorize Reactor Restart following completion of recovery actions is the _____.

2) The RCS Leakrate is required to be completed _____ in accordance with TS-4.13 Basis.

A. 1) Manager – Nuclear Operations.
   2) Shiftly.

B. 1) Manager – Nuclear Operations.
   2) Daily.

C. 1) Site Vice President.
   2) Shiftly.

D. 1) Site Vice President.
   2) Daily.

Question: 96

Given the following:

- Unit 1 and Unit 2 are at 100%.
- Post Maintenance Testing (PMT) is in progress following Emergency Diesel Generator No. 1 Battery replacement.
- 1-EPT-0106-03, Emergency Diesel No. 1 Battery Service Test has just been turned in for review as UNSAT.
- All other PMT items have been completed as Sat.

Which ONE of the following correctly answers the questions regarding returning Emergency Diesel No. 1 to service per VPAP 2003, Post Maintenance Testing:

1) Can Emergency Diesel No. 1 be returned to service without performing rework, with an Engineering justification that the test failure does not affect operability?
2) Which department is responsible for making the final determination of operability?

A. 1) Yes.
   2) Engineering.

B. 1) Yes.
   2) Operations.

C. 1) No.
   2) Engineering.

D. 1) No.
   2) Operations.

Question: 97

A licensed operator commences the release of a waste gas decay tank in accordance with OP-23.2.4, Release of Waste Gas Decay Tank 1B. Initial Hydrogen concentration in WGDT 1B is 67.3%. A spike results in the process vent radiation monitor, 1-GW-RI-130, entering into ACCIDENT mode. The release is subsequently isolated in accordance with 0-RMA-C7, Process Vent Normal Range Gas Alert/HI, and activity returns to pre-event levels.

Which ONE of the following states:

1) The maximum release rate from a WGDT 1B is based on _____.
2) The procedure used to return the radiation monitor to Normal Mode.

A. 1) the release permit
   2) I&C resets the RM using an Instrument Maintenance Procedure.

B. 1) the Hydrogen concentration
   2) I&C resets the RM using an Instrument Maintenance Procedure.

C. 1) release permit
   2) Operations resets the RM using 0-RMA-C7.

D. 1) the Hydrogen concentration
   2) Operations resets the RM using 0-RMA-C7.

Question: 98

Given the following conditions:

- A General Emergency has been declared.
- An individual has been injured in the Auxiliary Building penetration area.
- Dose rates in the area are 140 Rem/hr.
- It will take a maximum of 10 minutes to remove the individual from the area.

Which ONE of the following correctly describes the EPIP 4.04 requirements for:

1) Solicitation of a volunteer to rescue the individual.
2) Use of an RWP.

A. 1) Anticipated dose rates do NOT require the SEM to solicit volunteers to remove the injured worker.
2) An RWP is NOT required provided the SEM authorizes the activity.

B. 1) Due to anticipated dose for rescue workers, the SEM can assign ONLY volunteers to remove the injured worker.
2) An RWP is NOT required provided the SEM authorizes the activity.

C. 1) Anticipated dose rates do NOT require the SEM to solicit volunteers to remove the injured worker.
2) The SEM may NOT waive the requirement for an RWP.

D. 1) Due to anticipated dose for rescue workers, the SEM can assign ONLY volunteers to remove the injured worker.
2) The SEM may NOT waive the requirement for an RWP.

Question: 99

Initial Conditions:

- Unit 1 was initially at 100% power.
- "A" Train ICCM plasma display is INOPERABLE.

Current Conditions:

- A severe thunderstorm has caused a Loss of Off-Site Power (LOOP).
- Storm generated missiles have destroyed the following tanks.
  - 1 & 2-CN-TK-2 (Normal CN Storage Tanks).
  - 1-CN-TK-1 (Emergency CN Storage Tank).
  - 1-FP-TK-1A & 1B (Fire Protection Tanks).
  - At the completion of 1-ES-0.1, Reactor Trip Response, the operating team determines that a Natural Circulation Cooldown must be performed at a rate that will cause Reactor Vessel Head void formation.

Which ONE of the following states the required procedural transition for Unit 1?

A. Go directly to 1-ES-0.3, Natural Circulation Cooldown with Steam Void in Rx Vessel.

B. Go directly to 1-ES-0.4, Natural Circulation Cooldown with Steam Void in Rx Vessel Without RVLIS.

C. Go to 1-ES-0.2, Natural Circulation Cooldown. Initiate RCS cooldown, then transition to 1-ES-0.3, Natural Circulation Cooldown with Steam Void in Rx Vessel.

D. Go to 1-ES-0.2, Natural Circulation Cooldown. Initiate RCS cooldown, then transition to 1-ES-0.4, Natural Circulation Cooldown with Steam Void in Rx Vessel Without RVLIS.

Question: 100

Given the following sequence of events:

- Unit 1 tripped from 100% power due to "A" SG fault in Unit 1 Safeguards.
- A Main Steam Safety Valve has stuck open on "B" and "C" S/G on the reactor trip.
- The TDAFW pump, 1-FW-P-2, tripped on startup.
- SG narrow-range levels are off-scale low.
- "A" and "B" MDAFW pump trip and lock-out.
- The Team has transitioned to 1-E-2, Faulted Steam Generator Isolation.
- The Team has completed Step 1 when the STA reports a red-path exists on Heat Sink CSF status tree.

Which ONE of the following describes:

1) The first source of AFW directed to be restored in accordance with 1-FR-H.1, Response to Loss of Heat Sink.
2) Sequence of procedure transitions for this event.

A. 1) AFW from Unit 2 via the cross-tie.
   2) 1-FR-H.1, Response to Loss of Heat Sink; 1-ECA-2.1, Uncontrolled Depressurization of All S/Gs.

B. 1) Unit 1 Main Feed Water.
   2) 1-FR-H.1, Response to Loss of Heat Sink; 1-ECA-2.1, Uncontrolled Depressurization of All S/Gs.

C. 1) Unit 1 Main Feed Water.
   2) 1-FR-H.1, Response to Loss of Heat Sink; 1-E-2, Faulted S/G Isolation; 1-ECA-2.1, Uncontrolled Depressurization of All S/Gs.

D. 1) AFW from Unit 2 via the cross-tie.
   2) 1-FR-H.1, Response to Loss of Heat Sink; 1-E-2, Faulted S/G Isolation; 1-ECA-2.1, Uncontrolled Depressurization of All S/Gs.

SRO EXAM

LIST OF ATTACHMENTS

Attachment #	Attachment Description
1	TS Table 3.7-1, Page 3.7-12
2	TS Figure 3.12-3
3	TS Figure 3.8-1
4	TRM, section 3.7, Plant Systems (3.7.1 - 3.7.6)

Question #	Answer
SRO	
76	A
77	D
78	C
79	A
80	C
81	C
82	A
83	B
84	B
85	C
86	D
87	D
88	B
89	D
90	A
91	D
92	C
93	D
94	C
95	D
96	B
97	A
98	A
99	C
100	D