



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

October 19, 2015

Mr. Mano Nazar
President and Chief Nuclear Officer
Nuclear Division
NextEra Energy
P.O. Box 14000
Juno Beach, Florida 33408-0420

SUBJECT: ST. LUCIE PLANT, UNIT 1 AND UNIT 2 – CORRECTION OF AMENDMENT NOS. 226 AND 176 REGARDING TECHNICAL SPECIFICATION 6.8.4.h, "CONTAINMENT LEAKAGE RATE TESTING PROGRAM" (TAC NOS. MF4694, MF4695, MF6797, AMD MF6798)

Dear Mr. Nazar:

On August 27, 2015, the Nuclear Regulatory Commission (NRC) staff issued Amendment Nos. 226 and 176, respectively (Agencywide Documents Access and Management System (ADAMS) Accession No. ML15195A655) to St. Lucie Units 1 and 2. The amendment revised Technical Specifications 6.8.4.h, "Containment Leakage Rate Testing Program," to allow extension of the 10-year frequency of the containment Type A test to a 15-year frequency. Subsequent to the issuance, the NRC staff noted that an administrative error was made while processing the amendments, as described in the paragraph below.

To support the frequency extension, the licensee's original August 26, 2014, application (ADAMS Accession No. ML14241A496) referenced Nuclear Energy Institute (NEI) Topical Report NEI 94-01, Revision 3-A, "Industry Guideline for Implementing Performance-Based Option of 10 CFR [*Code of Federal Regulations*] Part 50, Appendix J." However, by letter dated January 14, 2015 (ADAMS Accession No. 15029A496), the licensee revised the reference to Revision 2-A of the same topical report. In issuing the amendment, the NRC staff inadvertently stated Revision 3-A instead of Revision 2-A.

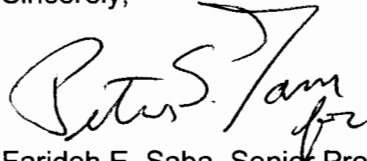
We are issuing this letter to correct the error in accordance with the guidance stated in SECY-96-238, dated January 16, 1997 (ADAMS Legacy Library Accession No. ML9611250030), which was approved by the Commission on December 17, 1996, via a Staff Requirements Memorandum (ADAMS Accession No. ML003754054). Enclosed are the corrected TS pages 6-15b; please use these pages to replace corresponding pages issued by Amendment Nos. 226 and 176 for the units. This correction does not change any of the conclusions in the safety evaluation associated with the amendments.

M. Nazar

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If you have any questions regarding this matter, please call me at (301) 415-1447.

Sincerely,

A handwritten signature in black ink, appearing to read "Farideh E. Saba". The signature is stylized and cursive, with a large initial "F" and "S".

Farideh E. Saba, Senior Project Manager
Plant Licensing Branch II-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket Nos. 50-335 and 50-389

Enclosures:
Corrected TS Page 6-15b for each unit

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ADMINISTRATIVE CONTROLS

- (2) conform to the guidance of Appendix I to 10 CFR Part 50, and
- (3) include the following:

- 1) Monitoring, sampling, analysis, and reporting of radiation and radionuclides in the environment in accordance with the methodology and parameters in the ODCM.
- 2) A Land Use Census to ensure that changes in the use of areas at and beyond the SITE BOUNDARY are identified and that modifications to the monitoring program are made if required by the results of this census, and
- 3) Participation in a Interlaboratory Comparison Program to ensure that independent checks on the precision and accuracy of the measurements of radioactive materials in environmental sample matrices are performed as part of the quality assurance program for environmental monitoring.

h. Containment Leakage Rate Testing Program

A program to implement the leakage rate testing of the containment as required by 10 CFR 50.54(o) and 10 CFR 50 Appendix J, Option B, as modified by approved exemptions. This program is in accordance with the guidelines contained in NEI 94-01, Revision 2-A, "Industry Guideline for Implementing Performance-Based Option of 10 CFR Part 50, Appendix J," except that the next Type A test performed after the December 8, 2005 Type A test shall be performed no later than December 8, 2020.

The peak calculated containment internal pressure for the design basis loss of coolant accident P_a , is 42.8 psig. The containment design pressure is 44 psig.

The maximum allowed containment leakage rate, L_a , at P_a , shall be 0.50% of containment air weight per day.

Leakage rate acceptance criteria are:

- a. Containment leakage rate acceptance criterion is $\leq 1.0 L_a$. During the first unit startup following testing in accordance with this program, the leakage rate acceptance criteria are $< 0.60 L_a$ for the Type B and C tests, $\leq 0.75 L_a$ for Type A tests, and $\leq 0.096 L_a$ for secondary containment bypass leakage paths.
- b. Air lock testing acceptance criteria are:
 - 1) Overall air lock leakage rate is $\leq 0.05 L_a$ when tested at $\geq P_a$.
 - 2) For the personnel air lock door seal, leakage rate is $< 0.01 L_a$ when pressurized to $\geq 1.0 P_a$.
 - 3) For the emergency air lock door seal, leakage rate is $< 0.01 L_a$ when pressurized to ≥ 10 psig.

ADMINISTRATIVE CONTROLS

than 8 days in gaseous effluents released from each unit to areas beyond the SITE BOUNDARY conforming to Appendix I to 10 CFR Part 50,

- 10) Limitations on the annual dose or dose commitment to any MEMBER OF THE PUBLIC, beyond the site boundary, due to releases of radioactivity and to radiation from uranium fuel cycle sources conforming to 40 CFR Part 190.

The provisions of Specifications 4.0.2 and 4.0.3 are applicable to the Radioactive Effluent Controls Program surveillance frequency.

g. Radiological Environmental Monitoring Program

A program shall be provided to monitor the radiation and radionuclides in the environs of the plant. The program shall provide (1) representative measurements of radioactivity in the highest potential exposure pathways, and (2) verification of the accuracy of the effluent monitoring program and modeling of the environmental exposure pathways. The program shall (1) be contained in the ODCM, (2) conform to the guidance of Appendix I to 10 CFR Part 50, and (3) include the following:

- 1) Monitoring, sampling, analysis, and reporting of radiation and radionuclides in the environment in accordance with the methodology and parameters in the ODCM.
- 2) A Land Use Census to ensure that changes in the use of areas at and beyond the SITE BOUNDARY are identified and that modifications to the monitoring program are made if required by the results of this census, and
- 3) Participation in a Interlaboratory Comparison Program to ensure that independent checks on the precision and accuracy of the measurements of radioactive materials in environmental sample matrices are performed as part of the quality assurance program for environmental monitoring.

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The peak calculated containment internal pressure for the design basis loss of coolant accident P_a , is 43.48 psig. The containment design pressure is 44 psig.

The maximum allowed containment leakage rate, L_a , at P_a , shall be 0.50% of containment air weight per day.

M. Nazar

- 2 -

If you have any questions regarding this matter, please call me at (301) 415-1447.

Sincerely,

/RA by P. Tam for F. Saba/

Farideh E. Saba, Senior Project Manager
Plant Licensing Branch II-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket Nos. 50-335 and 50-389

Enclosure:
Corrected TS Page 6-15b for each unit

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ADAMS Accession No: ML15280A377 (Pkg.); ML15280A392 (Letter); ML15280A422 (TS)NRR-058

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