



**UNITED STATES  
NUCLEAR REGULATORY COMMISSION**  
WASHINGTON, D.C. 20555-0001

November 30, 2015

Mr. Bryan C. Hanson  
Senior Vice President  
Exelon Generation Company, LLC  
President and Chief Nuclear Officer (CNO)  
Exelon Nuclear  
4300 Winfield Road  
Warrenville, IL 60555

**SUBJECT: BYRON STATION, UNIT NOS. 1 AND 2 - ISSUANCE OF AMENDMENTS RE:  
ULTIMATE HEAT SINK TO REFLECT CURRENT DESIGN BASIS FLOOD  
LEVEL (CAC NOS. MF5320 and MF5321)**

Dear Mr. Hanson:

The U.S. Nuclear Regulatory Commission (NRC or Commission) has issued the enclosed Amendment No. 193 to Renewed Facility Operating License No. NPF-37 and Amendment No. 193 to Renewed Facility Operating License No. NPF-66 for the Byron Station, Unit Nos. 1 and 2, respectively. The amendments are in response to your application dated November 24, 2014 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML14328A800).

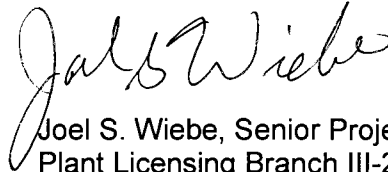
The amendments revise Condition I and Surveillance Requirement (SR) 3.7.9.3 associated with technical specification (TS), Section 3.7.9, "Ultimate Heat Sink (UHS)." The changes reflect the current design basis flood level and ensure the operability of the service water makeup pumps to meet TS 3.7.9 limiting condition for operation requirement.

B. Hanson

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A copy of the Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly *Federal Register* notice.

Sincerely,

A handwritten signature in black ink that reads "Joel S. Wiebe". The signature is written in a cursive style with a large, looping initial "J".

Joel S. Wiebe, Senior Project Manager  
Plant Licensing Branch III-2  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Docket Nos. STN 50-454 and STN 50-455

Enclosures:

1. Amendment No. 193 to NPF-37
2. Amendment No. 193 to NPF-66
3. Safety Evaluation

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

EXELON GENERATION COMPANY, LLC

DOCKET NO. STN 50-454

BYRON STATION, UNIT NO. 1

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 193  
Renewed License No. NPF-37

1. The U.S. Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Exelon Generation Company, LLC (the licensee) dated November 24, 2014, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Renewed Facility Operating License No. NPF-37 is hereby amended to read as follows.

(2) Technical Specifications

The Technical Specifications contained in Appendix A as revised through Amendment No. 193 and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated into this renewed license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 90 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

A handwritten signature in black ink, appearing to read "J. C. Poole" with a stylized flourish at the end.

Justin C. Poole, Acting Chief  
Plant Licensing Branch III-2  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical  
Specifications and Renewed Facility Operating License

Date of Issuance: November 30, 2015



**UNITED STATES  
NUCLEAR REGULATORY COMMISSION**  
WASHINGTON, D.C. 20555-0001

EXELON GENERATION COMPANY, LLC

DOCKET NO. STN 50-455

BYRON STATION, UNIT NO. 2

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 193  
Renewed License No. NPF-66

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Exelon Generation Company, LLC (the licensee) dated November 24, 2014, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Renewed Facility Operating License No. NPF-66 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A (NUREG-1113), as revised through Amendment No. 193 and the Environmental Protection Plan contained in Appendix B, both of which were attached to Renewed License No. NPF-37, dated November 19, 2015, are hereby incorporated into this renewed license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 90 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

A handwritten signature in black ink, appearing to read 'J. Poole', is written over the typed name below.

Justin C. Poole, Acting Chief  
Plant Licensing Branch III-2  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical  
Specifications and Renewed Facility Operating License

Date of Issuance: November 30, 2015

ATTACHMENT TO LICENSE AMENDMENT NOS. 193 AND 193  
RENEWED FACILITY OPERATING LICENSE NOS. NPF-37 AND NPF-66  
DOCKET NOS. STN 50-454 AND STN 50-455

Replace the following pages of the Renewed Facility Operating License and Appendix A Technical Specifications with the attached pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Remove

License NPF-37  
Page 3

License NPF-66  
Page 3

TSs  
3.7.9 – 4  
3.7.9 – 5

Insert

License NPF-37  
Page 3

License NPF-66  
Page 3

TSs  
3.7.9 – 4  
3.7.9 – 5

- (2) Pursuant to the Act and 10 CFR Part 70, to receive, possess and use at any time special nuclear material as reactor fuel, in accordance with the limitations for storage and amounts required for reactor operation, as described in the Updated Final Safety Analysis Report, as supplemented and amended;
- (3) Pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess, and use at any time any byproduct, source and special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;
- (4) Pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess, and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components; and
- (5) Pursuant to the Act and 10 CFR Parts 30, 40 and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.

C. The renewed license shall be deemed to contain and is subject to the conditions specified in the Commission's regulations set forth in 10 CFR Chapter I and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

(1) Maximum Power Level

The licensee is authorized to operate the facility at reactor core power levels not in excess of 3645 megawatts thermal (100 percent rated power) in accordance with the conditions specified herein.

(2) Technical Specifications

The Technical Specifications contained in Appendix A as revised through Amendment No. 193 and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated into this renewed license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

(3) Deleted.

(4) Deleted.

Renewed License No. NPF-37  
Amendment No. 193



- (2) Pursuant to the Act and 10 CFR Part 70, to receive; possess and use at any time special nuclear material as reactor fuel, in accordance with the limitations for storage and amounts required for reactor operation, as described in the Updated Final Safety Analysis Report, as supplemented and amended;
  - (3) Pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess, and use at any time any byproduct, source and special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;
  - (4) Pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess, and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components; and
  - (5) Pursuant to the Act and 10 CFR Parts 30, 40 and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.
- C. The renewed license shall be deemed to contain and is subject to the conditions specified in the Commission's regulations set forth in 10 CFR Chapter I and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:
- (1) Maximum Power Level  
  
The licensee is authorized to operate the facility at reactor core power levels not in excess of 3645 megawatts thermal (100 percent rated power) in accordance with the conditions specified herein.
  - (2) Technical Specifications  
  
The Technical Specifications contained in Appendix A (NUREG-1113), as revised through Amendment No. 193 and the Environmental Protection Plan contained in Appendix B, both of which were attached to Renewed License No. NPF-37, dated November 19, 2015, are hereby incorporated into this renewed license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
<p>H. Rock River water level <math>\leq</math> 670.6 ft Mean Sea Level (MSL).</p>	<p>H.1 Verify Rock River water level is <math>&gt;</math> 664.7 ft MSL and flow <math>\geq</math> 700 cubic feet per second (cfs).</p>	<p>1 hour <u>AND</u> Once per 12 hours thereafter</p>
<p>I. Required Action of Condition H not met.  <u>OR</u>  Rock River water level forecast to exceed 698.68 ft MSL by the National Weather Service (NWS).  <u>OR</u>  Tornado Watch issued by the NWS that includes the Byron site.</p>	<p>I.1 Verify basin level for each tower is <math>\geq</math> 90%.  <u>AND</u>  I.2 Verify OPERABILITY of at least one deep well pump.  <u>AND</u>  I.3 Verify OPERABILITY of both deep well pumps.</p>	<p>1 hour <u>AND</u> Once per 2 hours thereafter  1 hour  72 hours</p>
<p>J. Required Action and associated Completion Time of Condition A, B, C, E, F, G, or I not met.  <u>OR</u>  UHS inoperable for reasons other than Condition A, B, C, D, E, F, G, H, or I.</p>	<p>J.1 Be in MODE 3.  <u>AND</u>  J.2 Be in MODE 5.</p>	<p>6 hours  36 hours</p>

SURVEILLANCE REQUIREMENTS

SURVEILLANCE		FREQUENCY
SR 3.7.9.1	Verify water level in each SXCT basin is $\geq 60\%$ .	In accordance with the Surveillance Frequency Control Program
SR 3.7.9.2	Verify SXCT fan requirements in Table 3.7.9-1 or Table 3.7.9-2 are met.	In accordance with the Surveillance Frequency Control Program
SR 3.7.9.3	Verify river water level is $> 670.6$ ft MSL and $\leq 698.68$ ft MSL.	In accordance with the Surveillance Frequency Control Program
SR 3.7.9.4	Operate each required SXCT fan on high speed for $\geq 15$ minutes.	In accordance with the Surveillance Frequency Control Program
SR 3.7.9.5	Verify each SX makeup manual, power operated, and automatic valve in the flow path that is not locked, sealed, or otherwise secured in the open position, is in the correct position.	In accordance with the Surveillance Frequency Control Program
SR 3.7.9.6	Verify that each SX makeup pump starts on a simulated or actual low basin level signal and operates for $\geq 30$ minutes.	In accordance with the Surveillance Frequency Control Program

(continued)



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
RELATED TO AMENDMENT NO. 193 TO RENEWED FACILITY OPERATING LICENSE NO.  
NPF-37 AND AMENDMENT NO. 193 TO RENEWED FACILITY OPERATING LICENSE NO.  
NPF-66 EXELON GENERATION COMPANY, LLC  
BYRON STATION, UNIT NOS. 1 AND 2  
DOCKET NOS. STN 50-454 AND STN 50-455

1.0 INTRODUCTION

By letter to the U.S. Nuclear Regulatory Commission (NRC or Commission) dated November 24, 2014 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML14328A800), Exelon Generation Company, LLC (the licensee) requested changes to the renewed facility operating licenses for Byron Station, (Byron) Unit Nos. 1 and 2. The changes revise Condition I and surveillance requirement (SR) 3.7.9.3 associated with Technical Specification (TS) Section 3.7.9, "Ultimate Heat Sink (UHS)." The changes reflect the current design basis flood level and ensure the operability of the service water (SW) makeup pumps to meet TS 3.7.9, limiting condition for operation (LCO) requirement.

2.0 REGULATORY EVALUATION

In Section 4.1 of its submittal, Exelon Generation Company, LLC (the licensee) identified the applicable regulatory requirements related to its application.

Title 10 of the *Code of Federal Regulations* (10 CFR) Part 50, Appendix A, General Design Criterion (GDC) 2, "Design bases for protection against natural phenomena," requires that the portion of the UHS system that is important to safety be designed to withstand the effects of natural phenomena such as earthquakes, tornadoes, hurricanes, floods, tsunami, and seiches without loss of capability to perform their safety functions. GDC 2 also requires that the design bases for that system reflect appropriate consideration of the most severe of the natural phenomena that have been historically reported for the site and surrounding area, with sufficient margin for the limited accuracy, quantity, and period of time in which the historical data have been accumulated.

Part 50 of 10 CFR, Appendix A, GDC 44, "Cooling water," requires that a system be provided to transfer the combined heat load from structures, systems, and components (SSCs) important to safety to an UHS under normal operating and accident conditions. GDC 44 also requires that suitable redundancy in components and features, and suitable interconnections, leak detection, and isolation capabilities of the UHS system be provided

to assure that for onsite electric power system operation (assuming offsite power is not available) and for offsite electric power system operation (assuming onsite power is not available), the system safety function can be accomplished, assuming a single failure.

The U.S. Nuclear Regulatory Commission (NRC or Commission) regulatory requirements related to the contents of TS, set forth in 10 CFR 50.36, require that the TS LCOs are consistent with assumed values of the initial conditions in the licensee's safety analyses. Section 50.36(c)(2)(i) states that limiting conditions for operation are the lowest functional capability or performance levels of equipment required for safe operation of the facility. Under 10 CFR 50.36(c), TSs are required, in part, to include LCOs and SRs. When a LCO of a nuclear reactor is not met, the licensee shall shut down the reactor or follow any remedial action permitted by the TSs until the condition can be met.

The NRC Administrative Letter 98-10, "Dispositioning of Technical Specification That Are Insufficient to Assure Plant Safety," dated December 29, 1998 (ADAMS Accession No. ML031110108), covers the discovery of TSs that are insufficient to assure plant safety. The discovery of an improper or inadequate TS value or required action is considered a degraded or nonconforming condition as defined in Generic Letter 91-18, "Resolutions of Degraded and Nonconforming Conditions," dated October 8, 1997 (ADAMS Accession No. ML031200704). Imposing administrative controls in response to improper or inadequate TS is considered an acceptable short-term corrective action. The NRC staff expects that, following the imposition of administrative controls, an amendment to the TS, with appropriate justification and schedule, will be submitted in a timely fashion. Once an amendment correcting the TS is approved, the licensee must update the final safety analysis report (UFSAR), as necessary, to comply with 10 CFR 50.71(e).

### 3.0 TECHNICAL EVALUATION

The UHS provides a heat sink for removing process and operating heat from safety-related components during a transient or accident, as well as during normal operation. The UHS is a common system to Byron Station, Units 1 and 2, and consists of two essential service water mechanical draft cooling towers (SXCT) and a makeup system to the cooling towers. The normal makeup to the SXCT is provided by the non-safety, non-engineered safety feature, circulating water system. Two diesel-driven makeup pumps (SW makeup pumps) take suction from the Rock River to provide the safety-related source of makeup to the UHS. The SW makeup pumps auto start on low level in their associated cooling tower basin. The two diesel-driven SW makeup pumps are located in the River Screen House (RSH) at a nominal floor elevation of 702 feet and are enclosed by a 4-foot high wall.

A backup makeup source is provided by two non-safety, seismically qualified, deep well pumps. The deep wells provide a source of makeup water to the SXCT basins in the event of a flood more severe than the combined event flood (CEF) on the Rock River. The CEF is defined as a flood on the Rock River having  $1 \times 10^{-6}$  annual probability of exceedance at a 90 percent confidence level.

The licensee proposed to change the current TS 3.7.9, Condition I, which specifies a value of 702.0 feet mean sea level (MSL) for the Rock River water level. Currently, the Required Actions

to verify adequate initial basin level and OPERABILITY of the deep well pumps would be taken when the Rock River water level was forecasted to exceed 702.0 feet. This value is not conservative since it is 3.32 feet above the CEF level of 698.68 feet MSL used in the design of the RSH.

The NRC staff reviewed the proposed change as well as the current plant flood analysis. UFSAR Sections 2.4.3, "Probable Maximum Flood (PMF) on the Rock River" and 3.4.1 "Flood Protection," describe the significant and maximum wave effects of a coincident 40-mph overland wind. These were superimposed on the combined flood water level at the RSH. The wave runups, including setups, were calculated based on a water condition and are 2.77 feet and 4.71 feet for the significant and maximum waves, respectively. To prevent damage to the safety related SW makeup pump equipment due to flood and/or maximum wave runup, the floor elevation is established at 702 feet and a 4-foot high concrete wall encloses the area where the SW makeup pump equipment is located.

The RSH and the safety-related SW makeup pumps are designed and licensed to operate for the CEF river level of 698.68 feet. The RSH structure and the SW makeup pumps may be adversely affected by a higher river level since superimposing the maximum wave runup value on the current TS value of 702.0 feet would result in the maximum wave's elevation of 706.71 feet. Because this elevation exceeds the 4-foot high wall enclosing the safety-related equipment at the RSH structure, the SW makeup pumps could be damaged.

The NRC staff finds that decreasing the TS 3.7.9, Condition I Rock River water level to 698.68 feet for initiating required actions associated with Condition I provides additional margin to protect the SW makeup pumps from flooding that would render them inoperable. The use of a maximum wave runup value on a Rock River level of 698.68 feet would result in a maximum wave elevation of 703.39 feet at the RSH, which is below the height of the wall protecting the safety-related SW makeup pump equipment. This more conservative water level will allow the operability of the deep well pumps to be verified prior to reaching a water level that has the potential to render SW makeup pumps inoperable.

The NRC staff determined that 10 CFR 50, Appendix A, GDC 2, "Design bases for protection against natural phenomena," is met because establishing a water level of 698.68 feet at which to require verification of adequate initial UHS basin level and OPERABILITY of the deep well pumps ensures that the safety related portion of the UHS system will withstand the effects of floods on the Rock River without loss of capability to perform safety functions. The deep well pumps are located approximately 200 feet above the river at plant elevation and are not affected by flooding on the Rock River.

The NRC staff determined that 10 CFR Part 50, Appendix A, GDC 44, "Cooling water," is met because the new level provides assurance that the UHS is able to transfer the combined heat load from SSCs important to safety under postulated flood conditions.

The NRC staff determined that 10 CFR, Section 50.36, is met because the TS 3.7.9 LCOs are consistent with assumed values of the flooding in the licensee's safety analyses, and that SR 3.7.9.3 will provide assurance the LCO is met.

Based on the above, the NRC staff finds the following TS changes acceptable.

Revising TS 3.7.9 to replace "702.0 ft" with "698.68 ft" as follows:

CONDITION

- I. Required Action of Condition H not met.

OR

Rock River water level forecast to exceed 698.68 ft MSL by the National Weather Service (NWS).

OR

Tornado Watch issued by the NWS that includes the Byron site.

The NRC staff also finds that replacing "702.0 ft" with "698.68 ft" in SR 3.7.9.3 such that the Revised SR 3.7.9.3 states, "Verify river water level is > 670.6 ft MSL and < 698.68 ft MSL," is acceptable because, as discussed above, it ensures that the safety-related portion of the UHS system will withstand the effects of flooding on the Rock River without loss of capability to perform safety functions.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Illinois State official was notified on October 2, 2015, of the proposed issuance of the amendment. The State official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

The amendments change requirements with respect to installation or use of a facility's component located within the restricted area as defined in 10 CFR Part 20 and changes a surveillance requirement. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration (80 FR 17088; March 31, 2015), and there has been no public comment on such finding. Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by

operation in the proposed manner, (2) there is reasonable assurance that such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: T. Sweat, NRR

Date of issuance: November 30, 2015



B. Hanson

- 2 -

A copy of the Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly *Federal Register* notice.

Sincerely,

*/RA/*

Joel S. Wiebe, Senior Project Manager  
Plant Licensing Branch III-2  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Docket Nos. STN 50-454 and STN 50-455

Enclosures:

1. Amendment No. 193 to NPF-37
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3. Safety Evaluation

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