

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR: 8712070402 DOC. DATE: 87/12/01 NOTARIZED: NO DOCKET #
 FACIL: 50-269 Oconee Nuclear Station, Unit 1, Duke Power Co. 05000269
 50-270 Oconee Nuclear Station, Unit 2, Duke Power Co. 05000270
 50-287 Oconee Nuclear Station, Unit 3, Duke Power Co. 05000287

AUTH. NAME AUTHOR AFFILIATION
 TUCKER, H. B. Duke Power Co.
 RECIP. NAME RECIPIENT AFFILIATION
 Document Control Branch (Document Control Desk)

SUBJECT: Forwards clarification to Tech Spec 4.5.2 providing addl assurances that testing of valves associated w/reactor bldg cooling sys is per requirements specified in subsection IWV of Section IX of ASME boiler & pressure vessel code.

DISTRIBUTION CODE: A047D COPIES RECEIVED: LTR 1 ENCL 1 SIZE: 2+5
 TITLE: OR Submittal: Inservice Inspection/Testing

NOTES: AEOD/Ornstein:1cy. 05000269
 AEOD/Ornstein:1cy. 05000270
 AEOD/Ornstein:1cy. 05000287

	RECIPIENT ID CODE/NAME	COPIES LTTR ENCL	RECIPIENT ID CODE/NAME	COPIES LTTR ENCL
	PD2-3 LA	1 0	PD2-3 PD	5 5
	PASTIS, H	1 1		
INTERNAL:	AEOD/DOA	1 1	AEOD/DSP/TPAB	1 1
	ARM/DAF/LFMB	1 0	NRR/DEST/MEB	1 1
	NRR/DEST/MTB	1 1	NRR/RMAS/ILRB	1 1
	OGC/HD52	1 0	<u>REG FILE</u> 01	1 1
	RES/DE/EIB	1 1		
EXTERNAL:	LPDR	1 1	NRC PDR	1 1
	NSIC	1 1		
NOTES:		1 1		

DUKE POWER COMPANY

P.O. BOX 33189
CHARLOTTE, N.C. 28242

HAL B. TUCKER
VICE PRESIDENT
NUCLEAR PRODUCTION

TELEPHONE
(704) 373-4531

December 1, 1987

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Subject: Oconee Nuclear Station
Docket Nos. 50-269, -270, -287
Reactor Building Cooling Systems

Gentlemen:

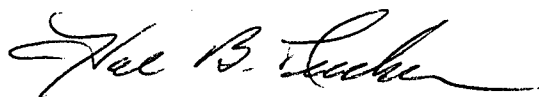
By letter dated February 6, 1986 Duke provided a proposed amendment to the Oconee Facility Operating License and revision to the Oconee Nuclear Station Technical Specifications (T.S.). The proposal consisted of a change to T.S. 4.5.2.1 Reactor Building Spray System. By letter dated August 20, 1986 Duke provided a supplement to the February 6, 1986 submittal which revised the No Significant Hazards Consideration (NSHC) evaluation.

Per subsequent conversations with the NRC Staff, it was requested that a clarification to T.S. 4.5.2 be added concerning testing of Reactor Building Cooling System valves. Accordingly, please find attached a supplement to the February 6, 1986 amendment request (Attachment 1) and appropriate revisions to the NSHC (Attachment 2) provided by the August 20, 1986 Duke letter.

The proposed clarification provides additional assurances that testing of valves associated with the Reactor Building Cooling System is in accordance with the requirements specified in subsection IWV of Section XI of the ASME Boiler and Pressure Vessel Code. The regulatory requirement for the implementation of Section XI by a licensee is provided by 10CFR 50.55a.

Inasmuch as the request for these proposed changes are a supplement to a previous Technical Specification amendment request, no license fees are required.

Very truly yours,



Hal B. Tucker

PJN/249/jgc

Attachment

8712070402 871201
PDR ADOCK 05000269
PDR

AD47
1/1

Document Control Desk
December 1, 1987
Page 2

xc: Dr. J. Nelson Grace
Regional Administrator
U.S. Nuclear Regulatory
Commission - Region II
101 Marietta St. NW
Suite 2900 - Atlanta, GA 30323

Ms. Helen Pastis
Office of Nuclear Reactor Regulation
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Mr. Heyward Shealy, Chief
Bureau of Radiological Health
S.C. Department of Health and
Environmental Control
2600 Bull Street
Columbus, S.C. 29201

Mr. J.C. Bryant
NRC Resident Inspector
Oconee Nuclear Station