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SUBJECT: Provides confirmation of status of investigation re "Stress Corrosion Cracking in Type 410 Stainless Steel Fastners in Check Valves,per NRC Bulletin 89-02.

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June 20, 1991

U. S. Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, DC 20555

Subject: Oconee Nuclear Station, Unit 3
Docket Nos. 50-269, -270, and -287
NRC Bulletin No. 89-02
Stress Corrosion Cracking in Type 410 Stainless Steel Fasteners in Check Valves
Unit 3 Final Response

Gentlemen:

Mr. C. E. Rossi's (NRC/ONRR) July 19, 1989 letter (Bulletin 89-02) concerned stress corrosion cracking of high-hardness type 410 Stainless Steel internal preloaded bolting in Anchor Darling Model S350W swing check valves or valves of similar design. The purpose of this bulletin was to request identification, disassembly, and inspection of certain types of swing check valves which may contain type 410 Stainless Steel (SS) bolting material; and if the Type 410 SS bolting material is of sufficiently high hardness that it is susceptible to stress corrosion cracking (SCC), or has failed, to take appropriate actions.

Pursuant to Reporting Requirement No. 2 of the Bulletin, this letter provides confirmation of the status of our investigation into this bulletin at Oconee Nuclear Station Unit 3.

Although Oconee Nuclear Station Unit 3 does not have any Anchor Darling Model S350 swing check valves, a design review identified eleven valves of a similar design. These valves were all manufactured by Velan and are located as follows:

Application	Location and Size
3 HP-105, 109, 113	High Pressure Injection Pump discharge,3 in.
3 LP-55, 57	Decay heat Cooler discharge to High Pressure Injection pump suction, 3 in.
3 BS-7, 9	Decay heat Cooler discharge to Building Spray pump suction, 3 in.
3 HP-188, 194	High Pressure Injection Pump discharge,4 in.
3 CC-76, 77	Component Cooling Water to CRDM's, 3 in.

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Nine of the eleven applicable (i.e., subject to the Bulletin) check valves were disassembled and inspected during the past Unit 3 refueling outage (which ended on March 30, 1991). The internal 410 Stainless Steel fasteners were dye penetrant tested using ASME Section XI criteria. Although no cracks or unacceptable indications were found, the fasteners were replaced, with A 193 Grade B8 material as recommended by the vendor.

The other two check valves, 3 HP-188 and 194, were not disassembled during the outage since the station has included these applications in their valve replacement plan. No further action regarding this Bulletin is planned for Unit 3.

In conclusion, the results of our completed investigation revealed no broken or cracked 410 Stainless Steel Fasteners on Oconee Nuclear Station Units 1,2 or 3 check valve internals inspected. This report closes out all Duke Power action regarding Oconee Nuclear Station and NRC Bulletin 89-02.

I declare under penalty of perjury that the statements set forth herein are true and correct to the best of my knowledge. Should there be any questions concerning this matter or if additional information is required, please contact S. G. Benesole at (704) 373-2101.

Very truly yours,

M. J. Tuckman

M. S. Tuckman

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