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 Document Control Branch (Document Control Desk)

SUBJECT: Forwards addl info re implementation of TMI Action Item  
 II.K.3.5, "Automatic Trip of Reactor Coolant Pumps," per  
 860721 & 1020 ltrs.

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November 30, 1987

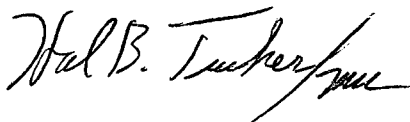
U. S. Nuclear Regulatory Commission  
Attention: Document Control Desk  
Washington, D. C. 20555

Subject: Oconee Nuclear Station  
Docket Nos. 50-269, -270, -287  
Generic Letter 86-05  
Implementation of TMI Action Item II.K.3.5,  
"Automatic Trip of Reactor Coolant Pumps"

Dear Sir:

By letter dated May 29, 1986, the NRC transmitted Generic Letter 86-05 to all licensees with Babcock and Wilcox (B&W) designed Nuclear Steam Supply Systems. Enclosed with the Generic Letter was the NRC's Safety Evaluation (SE) regarding the B&W Owners Group submittals on TMI Action Item II.K.3.5, which requested B&W licensees provide additional information. By letters dated July 21, 1986 and October 20, 1986, Duke power provided the requested information for the Oconee Nuclear Station. In a subsequent conference call with members of the NRC Staff and their consultants, Duke Power provided additional information and agreed to submit this information in a letter to the NRC. Accordingly, please find attached the additional information.

Very truly yours,

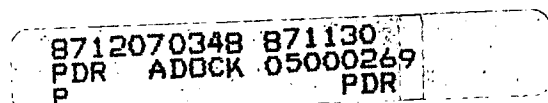


Hal B. Tucker

Attachment

PFG/1040/sbn

Attachment



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U. S. Nuclear Regulatory Commission  
November 30, 1987  
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Oconee Nuclear Station

Ms. Helen Pastis  
Office of Nuclear Reactor Regulation  
U. S. Nuclear Regulatory Commission  
Washington, D. C. 20555

Duke Power Company  
Oconee Nuclear Station  
Attachment

Request:

Identify the value (indicated) of the subcooling margin at which the operator is instructed to trip the pumps.

Response:

The operator is instructed to trip the reactor coolant pumps at 0°F subcooled margin. This setpoint is documented in the following emergency procedures:

Oconee Unit 1 - EP/1/A/1800/01  
Oconee Unit 2 - EP/2/A/1800/01  
Oconee Unit 3 - EP/3/A/1800/01

Request:

Duke's October 20, 1986 letter indicates replacement of the Motorola 56PH pressure transmitters with Rosemount 1153GD9 units. Describe the purpose for the replacement and verify installation. The stated uncertainty of the Rosemount unit (250 psi) is greater than the Motorola (225 psi) for adverse containment condition. Verify that the subcooled margin curves account for the difference in uncertainty.

Response:

The Motorola 56PH wide range RCS pressure transmitters were replaced with Rosemount 1153GD9 transmitters during the course of routine instrumentation maintenance. The primary reason for the change was due to the difficulty in obtaining replacement parts for the Motorola transmitters. The Rosemount transmitters have been installed at all three Oconee Units and the affected calibration procedures have been appropriately revised. The instrument uncertainties for the Rosemount transmitters provided in the October 10, 1986 submittal were preliminary numbers based on a review of the vendor specifications. A detailed uncertainty analysis for these transmitters has been completed and documented in accordance with Duke QA procedures. The resultant measurement uncertainties for wide range RCS pressure are as follows:

<u>Instrument</u>	<u>Range</u>	<u>Containment Conditions</u>	
		<u>Normal</u>	<u>Adverse</u>
Rosemount 1153GD9	0-2500 psig	75 psig	220 psig

Since these uncertainties are less than the 120 psi and 225 psi wide range RCS pressure uncertainties currently in the subcooled margin curves, the current curves conservatively account for wide range RCS pressure measurement uncertainty.

Request:

Verify that the instrument channels for determining subcooled margin are independent (e.g., diverse power supplies, cable separation).

Response:

The instrument channels for determining subcooled margin are independent. A detailed discussion of the design is provided by a Duke letter dated June 8, 1983 submitted in response to a NRC request for additional information.