

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR: 8709110350 DOC. DATE: 87/09/01 NOTARIZED: NO DOCKET #
 FACIL: 50-000 Generic Docket 05000000
 50-269 Oconee Nuclear Station, Unit 1, Duke Power Co. 05000269
 50-270 Oconee Nuclear Station, Unit 2, Duke Power Co. 05000270
 50-289 Three Mile Island Nuclear Station, Unit 1, General Pu 05000289
 50-369 William B. McGuire Nuclear Station, Unit 1, Duke Powe 05000369
 50-370 William B. McGuire Nuclear Station, Unit 2, Duke Powe 05000370
 50-413 Catawba Nuclear Station, Unit 1, Duke Power Co. 05000413
 50-414 Catawba Nuclear Station, Unit 2, Duke Power Co. 05000414

AUTH. NAME AUTHOR AFFILIATION
 TUCKER, H. B. Duke Power Co.
 RECIP. NAME RECIPIENT AFFILIATION
 Document Control Branch (Document Control Desk)

SUBJECT: Discusses containment integrated leak rate testing & imposition of backfit by virtue of change in staff position. Interpretations of 10CFR50, App J should be subjected to regulatory analysis.

DISTRIBUTION CODE: A017D COPIES RECEIVED: LTR 1 ENCL 0 SIZE: 6
 TITLE: OR Submittal: Append J Containment Leak Rate Testing

NOTES: AEOD/Ornstein: 1cy. 05000269
 AEOD/Ornstein: 1cy. 05000270
 R. Conte: 1cy. AEOD/ORNSTEIN: 1cy. 05000289
 LPDR 2cys AMDTS to FSAR. ASLB 1cy. 05000413
 LPDR 2cys AMDTS to FSAR. ASLB 1cy. 05000414

RECIPIENT ID CODE/NAME	COPIES LTTR ENCL	RECIPIENT ID CODE/NAME	COPIES LTTR ENCL
PD2-3 LA	1 0	PD1-4 LA	1 0
PD2-3 PD	5 5	PD1-4 PD	5 5
PASTIS, H	1 1	EDISON, G	1 1
HOOD, D	1 1	JABBOUR, K	1 1
INTERNAL: ACRS	10 10	ARM/DAF/LFMB	1 0
<u>NRR/DEST/PSB</u>	1 1	OGC/HDS2	1 1
<u>REG FILE</u> 01	1 1	RES DEPY GI	1 1
RES TELFORD, J	1 1	RES/DE/SSEB	1 1
RES/DRAA/SAIB	1 1	RES/DRPS/RPSIB	1 1
EXTERNAL: LPDR	4 4	NRC PDR	1 1
NSIC	1 1		
NOTES:	4 4		

TOTAL NUMBER OF COPIES REQUIRED: LTTR 45 ENCL 42

DUKE POWER COMPANY

P.O. BOX 33189
CHARLOTTE, N.C. 28242

HAL B. TUCKER
VICE PRESIDENT
NUCLEAR PRODUCTION

TELEPHONE
(704) 373-4531

September 1, 1987

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Subject: Oconee Nuclear Station
Docket Nos. 50-269, -270, -289
McGuire Nuclear Station
Docket Nos. 50-369, -370
Catawba Nuclear Station
Docket Nos. 50-413, -414
Containment Integrated Leak Rate Testing
Imposition of Backfit by Virtue of
Change in Staff Position

Gentlemen:

SUMMARY

In recent months the NRC Staff has adopted various interpretations of 10CFR50 Appendix J which appear to be contrary to previous Staff Positions and which do not appear to be soundly based upon the text of Appendix J. Two items of particular concern to Duke Power are 1) the Staff's reluctance to allow short-duration tests using the Mass-Plot analysis method, and 2) the Staff's requirement that as-found (Type B and C) leakage be included in Type A test results. The purpose of this letter is to assert Duke's position that these interpretations constitute backfit requirements, as defined by 10CFR50.109, and as such should be subjected to the Regulatory Analysis required by §50.109.

DISCUSSION

In 1973, 10CFR50 Appendix J went into effect, referencing ANSI N45.4-1972 (Leakage-Rate Testing of Containment Structures for Nuclear Reactors) as the operative method for performing Leak-Rate Testing at nuclear stations. Since that time Duke has performed leak-rate testing according to Appendix J, with some exemptions.

As time and technology have progressed, the NRC Staff has supplemented the requirements of Appendix J through Staff Positions, both tacit and explicit, which become backfitted requirements when enforced. For example in 1977 the Staff, absent any revision to Appendix J, began requiring that Integrated Leak Rate Test (ILRT) acceptance criteria include a 95 percent Upper Confidence Limit (UCL), rather than actual measured leakage. Inspection of Appendix J and ANSI N45.4-1972 reveals no mention of UCL. This requirement was presumably based on the then-draft revision to the standard, which was issued in 1981 as ANSI/ANS-56.8. Clearly, had the backfit rule been in effect in 1977 the imposition of the more stringent UCL requirement would have been considered as such.

8709110350 870901
PDR ADDOCK 05000269
PDR

A017
1/0

Another example of a changed Staff position as it affects Appendix J testing relates to the method used to analyze ILRT data. ANSI N45.4-1972 requires that either the "total time" or the "point-to-point" method be used to analyze ILRT data. However, starting in 1976 the Staff endorsed, again absent any revision to Appendix J, the Mass-Plot (Mass-Point) method. As late as 1986 (reference: Staff Review of Leak-rate Methodology, Deputy Director, NRR to Director, Div. of Inspection Programs, IE, April 1, 1986) the Staff has recognized the acceptability of the Mass-Plot method. On August 1, 1986 Duke was notified by the Staff that Mass-Plot was not an acceptable analysis method because Mass-Plot was not provided for in Appendix J. Duke had already obtained exemptions to allow use of Mass-Plot at the McGuire and Catawba stations, and subsequently obtained an exemption for Oconee.

The preceding two examples illustrate instances in which "Staff Position" has achieved virtually regulatory status without being accorded the due process required for rulemaking. Fortunately, neither has to date resulted in significant adverse impact. There are, however, two issues related to ILRTs which loom as significant contributors to increased probability of test failure and increased outage time. These issues are short-duration Mass-Plot testing and inclusion of as-found leakage from Type B and C testing in Type A test results.

Appendix J, through ANSI-N45.4, requires that ILRTs be performed for a 24-hour period, except "if it can be demonstrated to the satisfaction of those responsible for the acceptance of the containment structure that the leakage rate can be accurately determined during a shorter test period, the agreed-upon shorter period may be used." (ANSI-N45.4, paragraph 7.6). There are two important points in this exception. First, the exception is made without reference to the method of test analysis contained in the standard. Second, the exception refers to "those responsible for the acceptance of the containment structure." The NRC Staff, in its April 1, 1986 review of leak rate testing methodology stated that the only test of less than 24 hours which is acceptable to the NRC is the method specified in the Bechtel Corporation Topical, BN-TOP-1 (the total time and point-to-point methods). The Staff further contends that "those responsible for acceptance of the containment structure" refers to the NRC. In Inspection Report Number 50-269/86-13 (June 30, 1986) the Staff states "ANSI-N45.4, paragraph 7.6 requires a 24-hour test unless a test of shorter duration has been agreed upon by NRC." The Report also states "The Region believes that the second sentence [of paragraph 7.6] is simply a statement of the obvious; specifically, that with the review and approval of the regulatory body that initially approved the rule, in this case the NRC, an acceptable alternative to the requirements of that rule may be implemented."

It is not obvious at all that "those responsible for the acceptance of the containment structure" and "the regulatory body that approved the rule" are one and the same. The licensee is responsible for the operation, maintenance, and overall quality of the nuclear station. Acceptance of the containment structure is one aspect of the licensee's responsibility to assure that the health and safety of the public is not endangered. The ability to verify this acceptability in less than 24 hours, using Mass-Plot, has been explicitly recognized in the revised standard ANSI/ANS-56.8-1981, "Containment System Leakage Testing Requirements." It is implicit in the current standard, N45.4-1972, that if the acceptance criteria are met, the test duration is irrelevant.

The recent unwillingness of the Staff to accept short duration Mass-Plot testing represents a change in Staff position. The table below lists two Inspection Reports in which the NRC Inspector witnessed and, by signing the report with no related violations or identified items, tacitly approved short-duration tests using Mass-Plot.

<u>REPORT NO.</u>	<u>REPORT DATE</u>	<u>TEST DURATION</u>	<u>MINIMUM TEST DURATION PER PROCEDURE</u>
50-270/83-35	Dec. 15, 1983	8 hours 25 min.	6 hours
50-287/81-04	April 6, 1981	10 hours 45 min.	Not in procedure

Note that the inspector(s) reviewed at least one procedure which specified a minimum 6-hour test.

As noted, the provision in the standard to allow short duration testing did not make reference to any of the analysis methods included in the standard. In addition to the total time and point-to-point methods (two options in the broader category of the absolute method) the Standard describes testing by the Reference-Vessel Method. The Reference-Vessel Method is distinctly different from the absolute method; nevertheless, the Standard, in paragraph 7.10, reaffirms that "If it can be demonstrated to the satisfaction of those responsible for the acceptance of the containment that the leakage rate can be accurately determined during a shorter test period, the agreed upon shorter period may be used". It follows that if two diverse methods have the capability to satisfactorily determine leakage rates in less than 24 hours, then the duration of the test is not as critical a parameter as the April, 1986 Staff position seems to indicate.

In accordance with the backfit policy guidance presented in Chapter 0514 of the NRC Manual, the elements of imposition of a backfit are satisfied. The Staff's original position of acceptance of short duration testing is documented in the above inspection reports. The Staff's new position is documented in the April, 1986 Staff review of leakrate methodology. Actual imposition of the backfit has been accomplished by the need to perform 24-hour tests or risk ILRT failure. The previous Staff position had been in effect since the inception of Appendix J.

The issue of as-found leakage has already proven significant at McGuire Nuclear Station, where one unit was ruled to have failed an ILRT by virtue of having not included as-found leakage in Type A test results. The Staff's position, established in Information Notice 85-71, was identified too close in time to a scheduled ILRT to allow resolution in a timely manner. Information Notice 85-71 was dated August 22, 1985; before the effective date of 10 CFR 50.109 (October 21, 1985). However, the backfit policy identified in Generic Letter 84-08, which is similar to 50.109, is applicable to this issue.

Appendix J appears to have been predicated upon the intent of assuring containment integrity for the period of operation following the test. The acceptance criteria for the test require that the measured leakage (LTM) rate be 75% of the allowable leakage (LT) rate. Thus, assuming a nominal degradation (25%) of containment integrity over the subsequent period, the leakage will still be within the allowable limits at the conclusion of the period.

The inclusion of as-found leakage in Type A test results thus causes a double-counting of normal (anticipated) containment integrity degradation. The .75 La acceptance criterion of Appendix J concedes an acceptable amount of expected degradation. To further require that the actual degradation of the containment (i.e., the as-found leakage) result in a total leakage of less than .75 La, rather than La, is beyond the intent of the rule. Any other interpretation must refute the definition of La as an acceptable leakage and .75 La as an acceptance criterion.

The periodic testing and maintenance which is performed only serves to reinforce the probability of sound containment integrity. The Staff apparently accepted this philosophy for many years; as indicated by the fact the NRC inspectors have reviewed test procedures and witnessed ILRTs, and have not (until recently) identified any deviations or violations. There appears to be a shift in the philosophy of the intent of Appendix J from ensuring future operability to verification of past operability. It may be valid that future operability may best be expected based upon past history of containment leakage. It may also be that this new philosophy of verifying past operability results in penalization for utilities' valve maintenance programs by increasing the time required to perform Type B and C testing, increasing the possibility of Type A test failure, and thus increasing the Type A test frequency. In fact, the valve and penetration maintenance program will accomplish the same goal as the ILRT program, without subjecting the utility to the economic risks (i.e., increased outage time) associated with ILRT failure.

An example of the Staff's previous position that Type C as-found leakages need not be included in ILRT test results can be found in Information Report 50-269/80-06, transmitted to Duke by letter, R.C. Lewis to W.O. Parker, dated March 20, 1980. The inspector noted that 14 of 60 penetrations were not aligned as required for ILRT. As a result, the inspector required that the as-found leakage from those 14 valves be included in the ILRT results. As-found leakage from the 46 valves which were properly aligned were not required to be included. This specific exclusion of valves from the as-found testing requirement serves as documentation of a previous Staff position that as found valve leakage need not be included in ILRT results. Again, the Staff position presented in Information Notice 85-71 represents a departure from that position and should receive the appropriate analysis to determine that there is a substantial increase in the overall protection of the public health and safety from the imposition of this backfit.

CONCLUSION

The Staff's original positions on short duration Mass-Plot Testing and as-found leakage are documented in the various inspection reports referenced elsewhere in this letter. By approving the inspection reports with no applicable violations or deviations identified, the Staff has tacitly approved short-duration Mass-Plot Testing and ILRTs without as-found leakage. More recent Staff documents, notably the April 1, 1986 Staff review of leakage methodology and Inspection Notice 85-71, change Staff position in such a way as to place additional burden on Licensees without demonstrating that a significant increase in overall protection of public health and safety will be achieved. Duke considers that these issues are backfits and should receive the appropriate regulatory analysis. This analysis should compare the cost of the increased outage time associated with extended leak rate

1982
 1983
 1984
 1985
 1986
 1987
 1988
 1989
 1990
 1991
 1992
 1993
 1994
 1995
 1996
 1997
 1998
 1999
 2000
 2001
 2002
 2003
 2004
 2005
 2006
 2007
 2008
 2009
 2010
 2011
 2012
 2013
 2014
 2015
 2016
 2017
 2018
 2019
 2020
 2021
 2022
 2023
 2024
 2025
 2026
 2027
 2028
 2029
 2030
 2031
 2032
 2033
 2034
 2035
 2036
 2037
 2038
 2039
 2040
 2041
 2042
 2043
 2044
 2045
 2046
 2047
 2048
 2049
 2050
 2051
 2052
 2053
 2054
 2055
 2056
 2057
 2058
 2059
 2060
 2061
 2062
 2063
 2064
 2065
 2066
 2067
 2068
 2069
 2070
 2071
 2072
 2073
 2074
 2075
 2076
 2077
 2078
 2079
 2080
 2081
 2082
 2083
 2084
 2085
 2086
 2087
 2088
 2089
 2090
 2091
 2092
 2093
 2094
 2095
 2096
 2097
 2098
 2099
 2100
 2101
 2102
 2103
 2104
 2105
 2106
 2107
 2108
 2109
 2110
 2111
 2112
 2113
 2114
 2115
 2116
 2117
 2118
 2119
 2120
 2121
 2122
 2123
 2124
 2125
 2126
 2127
 2128
 2129
 2130
 2131
 2132
 2133
 2134
 2135
 2136
 2137
 2138
 2139
 2140
 2141
 2142
 2143
 2144
 2145
 2146
 2147
 2148
 2149
 2150
 2151
 2152
 2153
 2154
 2155
 2156
 2157
 2158
 2159
 2160
 2161
 2162
 2163
 2164
 2165
 2166
 2167
 2168
 2169
 2170
 2171
 2172
 2173
 2174
 2175
 2176
 2177
 2178
 2179
 2180
 2181
 2182
 2183
 2184
 2185
 2186
 2187
 2188
 2189
 2190
 2191
 2192
 2193
 2194
 2195
 2196
 2197
 2198
 2199
 2200
 2201
 2202
 2203
 2204
 2205
 2206
 2207
 2208
 2209
 2210
 2211
 2212
 2213
 2214
 2215
 2216
 2217
 2218
 2219
 2220
 2221
 2222
 2223
 2224
 2225
 2226
 2227
 2228
 2229
 2230
 2231
 2232
 2233
 2234
 2235
 2236
 2237
 2238
 2239
 2240
 2241
 2242
 2243
 2244
 2245
 2246
 2247
 2248
 2249
 2250
 2251
 2252
 2253
 2254
 2255
 2256
 2257
 2258
 2259
 2260
 2261
 2262
 2263
 2264
 2265
 2266
 2267
 2268
 2269
 2270
 2271
 2272
 2273
 2274
 2275
 2276
 2277
 2278
 2279
 2280
 2281
 2282
 2283
 2284
 2285
 2286
 2287
 2288
 2289
 2290
 2291
 2292
 2293
 2294
 2295
 2296
 2297
 2298
 2299
 2300
 2301
 2302
 2303
 2304
 2305
 2306
 2307
 2308
 2309
 2310
 2311
 2312
 2313
 2314
 2315
 2316
 2317
 2318
 2319
 2320
 2321
 2322
 2323
 2324
 2325
 2326
 2327
 2328
 2329
 2330
 2331
 2332
 2333
 2334
 2335
 2336
 2337
 2338
 2339
 2340
 2341
 2342
 2343
 2344
 2345
 2346
 2347
 2348
 2349
 2350
 2351
 2352
 2353
 2354
 2355
 2356
 2357
 2358
 2359
 2360
 2361
 2362
 2363
 2364
 2365
 2366
 2367
 2368
 2369
 2370
 2371
 2372
 2373
 2374
 2375
 2376
 2377
 2378
 2379
 2380
 2381
 2382
 2383
 2384
 2385
 2386
 2387
 2388
 2389
 2390
 2391
 2392
 2393
 2394
 2395
 2396
 2397
 2398
 2399
 2400
 2401
 2402
 2403
 2404
 2405
 2406
 2407
 2408
 2409
 2410
 2411
 2412
 2413
 2414
 2415
 2416
 2417
 2418
 2419
 2420
 2421
 2422
 2423
 2424
 2425
 2426
 2427
 2428
 2429
 2430
 2431
 2432
 2433
 2434
 2435
 2436

Hal B. Tucker

SAG/84/jgc

xc: Dr. J. Nelson Grace, Regional Administrator
U.S. Nuclear Regulatory Commission - Region II
101 Marietta Street, Suite 2900
Atlanta, Georgia 30323

Executive Director for Operations
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Dr. K.N. Jabbour, Project Manager
Office of Nuclear Reactor Regulation
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Ms. Helen Pastis, Project Manager
Office of Nuclear Reactor Regulation
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Mr. Darl Hood, Project Manager
Office of Nuclear Reactor Regulation
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Mr. P.K. Van Doorn
NRC Resident Inspector
Catawba Nuclear Station

Mr. W.T. Orders
NRC Resident Inspector
McGuire Nuclear Station

Mr. J.C. Bryant
NRC Resident Inspector
Oconee Nuclear Station

USNRC-DS

1987 SEP 11 A 9:27