

**CATEGORY 1**

**REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)**

ACCESSION NBR:9701140204    DOC.DATE: 97/01/08    NOTARIZED: NO  
 FACIL:50-270 Oconee Nuclear Station, Unit 2, Duke Power Co.  
 AUTH.NAME                    AUTHOR AFFILIATION  
 WILKIE,L.V.                  Duke Power Co.  
 HAMPTON,J.W.                Duke Power Co.  
 RECIP.NAME                  RECIPIENT AFFILIATION

DOCKET #  
05000270

SUBJECT: LER 96-006-00:on 961211,containment isolation valves  
 determined technically inoperable.Valves reassembled.W/  
 960108 ltr.

DISTRIBUTION CODE: IE22T    COPIES RECEIVED:LTR 1 ENCL 1 SIZE: 3  
 TITLE: 50.73/50.9 Licensee Event Report (LER), Incident Rpt, etc.

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**DUKE POWER**

January 8, 1997

U.S. Nuclear Regulatory Commission  
Document Control Desk  
Washington, D.C. 20555

Subject: Oconee Nuclear Station Unit  
Docket Nos. 50-269, -270, -287  
Licensee Event Report 270/96-06

Gentlemen:

Pursuant to 10 CFR 50.73 Sections (a) (1) and (d),  
attached is Licensee Event Report 270/96-06, concerning  
the past inoperability of containment isolation valves.

Duke's evaluation of this event is continuing and this  
report will be supplemented to address root cause and  
other LER requirements.

This report is being submitted in accordance with 10 CFR  
50.73 (a) (2) (ii).

Very truly yours,

J. W. Hampton, Vice President  
Oconee Nuclear Site

/fts

Attachment

140048  
9701140204 970108  
PDR ADOCK 05000270  
S PDR

Document Control Desk  
January 8, 1997

cc: Mr. Luis A. Reyes  
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Atlanta, GA 30323

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Mr. M. A. Scott  
NRC Resident Inspector  
Oconee Nuclear Station

### LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) **Oconee Nuclear Station, Unit Two** DOCKET NUMBER (2) **05000 270** PAGE (3) **1 OF 1**

TITLE (4) **CONTAINMENT ISOLATION VALVES TECHNICALLY INOPERABLE**

| EVENT DATE (5) |     |      | LER NUMBER (6) |                   |                 | REPORT DATE (7) |     |      | OTHER FACILITIES INVOLVED (8) |                  |
|----------------|-----|------|----------------|-------------------|-----------------|-----------------|-----|------|-------------------------------|------------------|
| MONTH          | DAY | YEAR | YEAR           | SEQUENTIAL NUMBER | REVISION NUMBER | MONTH           | DAY | YEAR | FACILITY NAME                 | DOCKET NUMBER(S) |
| 12             | 11  | 96   | 96             | 06                | 00              | 01              | 08  | 97   | Oconee, Unit Three            | 05000 287        |
|                |     |      |                |                   |                 |                 |     |      |                               | 05000            |

OPERATING MODE (9) **N** THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR (Check one or more of the following) (11)

|                           |                   |                   |                      |  |
|---------------------------|-------------------|-------------------|----------------------|--|
| POWER LEVEL (10) <b>0</b> | 20.402(b)         | 20.405(c)         | 50.73(a)(2)(iv)      | 73.71(b)   |
|                           | 20.405(a)(1)(i)   | 50.36(c)(1)       | 50.73(a)(2)(v)       | 73.71(c)   |
|                           | 20.405(a)(1)(ii)  | 50.36(c)(2)       | 50.73(a)(2)(vii)     | OTHER (Specify in Abstract below and in Text, NRC Form 366A) |
|                           | 20.405(a)(1)(iii) | 50.73(a)(2)(i)    | 50.73(a)(2)(viii)(A) |  |
|                           | 20.405(a)(1)(iv)  | X 50.73(a)(2)(ii) | 50.73(a)(2)(viii)(B) |  |
|                           | 20.405(a)(1)(v)   | 50.73(a)(2)(iii)  | 50.73(a)(2)(x)       |  |

LICENSEE CONTACT FOR THIS LER (12)

| NAME                                   | TELEPHONE NUMBER         |
|--|--------------------------|
| Lanny V. Wilkie, Safety Review Manager | AREA CODE (864) 885-3518 |

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

| CAUSE | SYSTEM | COMPONENT | MANUFACTURER | REPORTABLE TO NPRDS | CAUSE | SYSTEM | COMPONENT | MANUFACTURER | REPORTABLE TO NPRDS |
|-------|--------|-----------|--------------|---------------------|-------|--------|-----------|--------------|---------------------|
|       |        |           |              |                     |       |        |           |              |                     |
|       |        |           |              |                     |       |        |           |              |                     |

SUPPLEMENTAL REPORT EXPECTED (14)

|   |    |                               |       |     |      |
|---|----|-------------------------------|-------|-----|------|
| X YES (If yes, complete EXPECTED SUBMISSION DATE) | NO | EXPECTED SUBMISSION DATE (15) | MONTH | DAY | YEAR |
|   |    |                               | 02    | 28  | 97   |

ABSTRACT (Limit to 1400 spaces, i.e. approximately fifteen single-space typewritten lines) (16)

In December 1995, an inspection of Generic Letter 89-10 implementation identified hard seat contact on Anchor/Darling double disc gate valves as an issue. A Duke calculation using EPRI motor operated valves methodology was approved on July 22, 1996 and concluded that the valve's disc wedges have a preferred direction. During the current shutdown of all three Oconee Nuclear Station units, an inspection of these type valves identified four containment isolation valves that had the wedges installed in the non-preferred direction:

| Unit | Valve    | Installed    | Inspected     |
|------|----------|--------------|---------------|
| 2    | 2HP-20   | Mar. 5, 1992 | Nov. 26, 1996 |
|      | 2FDW-103 | Apr. 1, 1988 | Jan. 04, 1997 |
|      | 2FDW-104 | Apr. 1, 1988 | Jan. 04, 1997 |
| 3    | 3FDW-103 | Dec. 9, 1989 | Nov. 12, 1996 |

These valves were determined to be technically inoperable from the date of installation. They have been reassembled with the disc lower wedges in the preferred direction.

Duke's evaluation of this event is continuing and this report will be supplemented to address root cause and other LER content requirements.