



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

September 10, 2015

Mr. Robert Braun
President and Chief Nuclear Officer
PSEG Nuclear LLC-N09
P. O. Box 236
Hancocks Bridge, NJ 08038

SUBJECT: HOPE CREEK GENERATING STATION – INTERIM STAFF RESPONSE TO
REEVALUATED FLOOD HAZARDS SUBMITTED IN RESPONSE TO
10 CFR 50.54(f) INFORMATION REQUEST – FLOOD-CAUSING MECHANISM
REEVALUATION (TAC NO. MF3789)

Dear Mr. Braun:

The purpose of this letter is to provide a summary of the U.S. Nuclear Regulatory Commission (NRC) staff's assessment of the re-evaluated flood-causing mechanisms described in the March 12, 2014 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML14071A511) flood hazard reevaluation report (FHRR) submitted by PSEG Nuclear LLC (PSEG, the licensee) for Hope Creek Generating Station (Hope Creek), as well as supplemental information resulting from requests for additional information and audits.

By letter dated March 12, 2012, the NRC issued a request for information pursuant to Title 10 of the *Code of Federal Regulations*, Section 50.54(f) (hereafter referred to as the 50.54(f) letter) (ADAMS Accession No. ML12053A340). The request was issued as part of implementing lessons-learned from the accident at the Fukushima Dai-ichi nuclear power plant. Enclosure 2 to the 50.54(f) letter requested licensees to re-evaluate flood-causing mechanisms using present-day methodologies and guidance. Concurrently, with the reevaluation of flooding hazards, licensees were required to develop and implement mitigating strategies in accordance with NRC Order EA-12-049, "Requirements for Mitigation Strategies for Beyond-Design-Basis External Events" (ADAMS Accession No. ML12054A735). On March 30, 2015, the Commission provided Staff Requirements Memoranda (SRM) (ADAMS Accession No. ML15089A236) to COMSECY-14-0037, "Integration of Mitigating Strategies for Beyond-Design-Basis External Events and the Reevaluation of Flooding Hazards," dated November 21, 2014 (ADAMS Accession No. ML14309A256), affirming that licensees need to address the reevaluated flooding hazards within their mitigating strategies for beyond-design-basis external events.

The NRC staff has reviewed the information submitted by the licensee and has summarized the results of the review in the tables provided as an Enclosure to this letter. Table 1 provides the current design-basis flood hazard mechanisms. Table 2 provides the reevaluated flood hazard mechanisms; however, the reevaluated flood hazard mechanisms bounded by the current design-basis (Table 1) are not included.

The NRC staff has concluded that the licensee's reevaluated flood hazards information, as summarized in the Enclosure, is suitable for the assessment of mitigating strategies developed in response to Order EA-12-049 (i.e., defines the mitigating strategies flood hazard information described in guidance documents currently being finalized by the industry and NRC staff) for Hope Creek. Further, the staff has concluded that the licensee's reevaluated flood hazard information is a suitable input for other assessments associated with Near-Term Task Force Recommendation 2.1 "Flooding". The NRC staff plans to issue a staff assessment documenting the basis for these conclusions at a later time.

In addition, NEI 12-06 "Diverse and Flexible Coping Strategies (FLEX) Implementation Guide" is currently being revised. This revision will include a methodology to perform a Mitigating Strategies Assessment (MSA) with respect to the reevaluated flood hazards. Once this methodology is endorsed by the NRC, flood event duration parameters and applicable flood associated effects should be considered as part of the Hope Creek MSA. The NRC staff will evaluate the flood event duration parameters (including warning time and period of inundation) and flood-related associated effects developed by the licensee during the NRC staff's review of the MSA.


As stated above, Table 2 of the enclosure to this letter describes the reevaluated flood hazards that exceed the current design-basis. In order to complete its response to the information requested by Enclosure 2 to the 50.54(f) letter, the licensee is expected to submit an integrated assessment or a focused evaluation, as appropriate, to address these reevaluated flood hazards, as described in the NRC letter, "Coordination of Request for Information Regarding Flooding Hazard Reevaluation and Mitigating Strategies for Beyond-Design-Basis External Events" (ADAMS Accession No. ML15174A257). This letter describes the changes in the NRC's approach to the flood hazard reevaluations that were approved by the Commission in its SRM to COMSECY-15-0019, "Closure Plan for the Reevaluation of Flooding Hazards for Operating Nuclear Power Plants" (ADAMS Accession No. ML15209A682).

R. Braun

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If you have any questions, please contact me at (301) 415-6197 or e-mail at tekia.govan@nrc.gov.

Sincerely,

A handwritten signature in black ink, appearing to read "Tekia V. Govan", with a long, sweeping horizontal line extending to the right.

Tekia V. Govan, Project Manager
Hazards Management Branch
Japan Lessons-Learned Division
Office of Nuclear Reactor Regulation

Docket No. 50-354

Enclosure:
Summary of Results of Flooding
Hazard Re-Evaluation Report

cc w/encl: Distribution via Listserv

ENCLOSURE:

SUMMARY TABLES OF
REEVALUATED FLOOD HAZARD LEVELS

Table 1. Current Design Basis Flood Hazards for Use in the MSA

Mechanism	Stillwater Elevation	Waves/Runup	Design Basis Hazard Elevation	Reference
Local Intense Precipitation	12.1 ft NAVD88	Minimal	12.1 ft NAVD88	FHRR Table 3-1
Streams and Rivers				
Powerblock/Service Water Intake Structure	7.5 ft NAVD88	12.5 ft	20.0 ft NAVD88	FHRR Section 1.2.2
Failure of Dams and Onsite Water Control/Storage Structures				
Multiple Failures of Connonsville, Pepacton and Tocks Island Dams	12.7 ft NAVD88	12.8 ft	25.5 ft NAVD88	FHRR Section 1.2.3
Storm Surge				
Powerblock A	24.0 ft NAVD88	6.2 ft	30.2 ft NAVD88	FHRR Table 3-1
Powerblock B	24.0 ft NAVD88	10.6 ft	34.6 ft NAVD88	FHRR Section 1.2.4
Service Water Intake Structure	24.0 ft NAVD88	20.6 ft	44.6 ft NAVD88	FHRR Section 1.2.4
Seiche				
	No Impact on the Site Identified	No Impact on the Site Identified	No Impact on the Site Identified	FHRR Section 1.2.5
Tsunami				
	5.2 ft NAVD88	12.1 ft	17.3 ft NAVD88	FHRR Section 1.2.6
Ice-Induced Flooding				
	No Impact on the Site Identified	No Impact on the Site Identified	No Impact on the Site Identified	FHRR Section 1.2.7

Table 1. Current Design Basis Flood Hazards for Use in the MSA

Mechanism	Stillwater Elevation	Waves/ Runup	Design Basis Hazard Elevation	Reference
Channel Migrations/Diversions	No Impact on the Site Identified	No Impact on the Site Identified	No Impact on the Site Identified	FHRR Section 1.2.8

Note 1: Reported values are rounded to the nearest one-tenth of a foot.

Note 2: Based on the NRC staff's independent (deterministic) hazard assessment using present-day regulatory guidance and methodologies of storm surge, the staff concludes that the site's current design basis remains bounded. For this reason, the staff concludes it is appropriate to utilize the current design basis storm surge elevation in conjunction with the mitigating strategies assessment.

Table 2. Reevaluated Flood Hazards for Flood-Causing Mechanisms for Use in the MSA

Mechanism	Stillwater Elevation	Waves/ Runup	Reevaluated Hazard Elevation	Reference
Local Intense Precipitation	12.8 ft NAVD88	Minimal	12.8 ft NAVD88	FHRR Section 3.1

Note 1: The licensee is expected to develop flood event duration parameters and applicable flood associated effects to conduct the MSA. The staff will evaluate the flood event duration parameters (including warning time and period of inundation) and flood associated effects during its review of the MSA.

Note 2: Reevaluated hazard mechanisms bounded by the current design basis (see Table 1) are not included in this table.

Note 3: Reported values are rounded to the nearest one-tenth of a foot.

R. Braun

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If you have any questions, please contact me at (301) 415-6197 or e-mail at tekia.govan@nrc.gov.

Sincerely,

/RA/

Tekia V. Govan, Project Manager
Hazards Management Branch
Japan Lessons-Learned Division
Office of Nuclear Reactor Regulation

Docket No. 50-354

Enclosure:
Summary of Results of Flooding
Hazard Re-Evaluation Report

cc w/encl: Distribution via Listserv

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ADAMS Accession No.: PKG ML15238B660; LTR: ML15238B655; ENCL: ML15243A502 *via email

OFFICE	NRR/JLD/JHMB/PM	NRR/JLD/LA	NRO/DSEA/RHM1/TR*	NRO/DSEA/RHM1/TL*
NAME	TGovan	SLent	MLee	KErwin
DATE	09/02/15	09/01/15	09/09/15	09/09/15
OFFICE	NRR/JLD/JHMB/BC	NRR/JLD/JHMB/PM		
NAME	MShams	TGovan		
DATE	09/10/15	09/10/15		

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