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 REID, R. W. Operating Reactors Branch 4

SUBJECT: Forwards supplemental info to 800131 ltr re NUREG-0578 Items 2.1, 3b, 5c, 6a, 8a & 8c. Provides info on instrumentation for detection of inadequate core cooling, capability to install hydrogen purge unit & inplant iodine monitoring.

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NOTES: ~~M. CUNNINGHAM - ALL AMENDMENTS TO FSAR + CHANGES TO TECH SPECS~~

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DUKE POWER COMPANY

POWER BUILDING

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WILLIAM O. PARKER, JR.
VICE PRESIDENT
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February 8, 1980

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Mr. Harold R. Denton, Director
Office of Nuclear Reactor Regulation
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

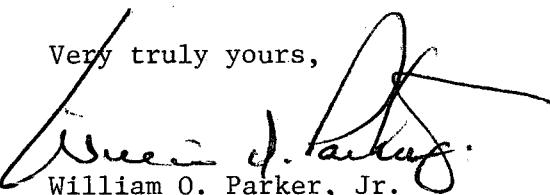
Attention: Mr. R. W. Reid, Chief
Operating Reactors Branch No. 4

Re: Oconee Nuclear Station
Docket Nos. 50-269, -270, -287

Dear Sir:

Please refer to my letter of January 31, 1980. Attached is supplemental information for NUREG-0578 items 2.1.3b, 2.1.5c, 2.1.6a, 2.1.8a, and 2.1.8c.

Very truly yours,


William O. Parker, Jr.

RLG:scs

Attachment

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DUKE POWER COMPANY
OCONEE NUCLEAR STATION

NUREG-0578 Supplemental Response

2.1.3b Instrumentation for Detection of Inadequate Core Cooling

In a letter dated November 21, 1979, Duke provided Operator Aid Computer (OAC) system availability data. This submittal was based on the best available information at the time. This data did not take into account unit operation which when factored in, provides the OAC availability data for 1979 for each unit when power is above 15% FP (turbine on line) is:

Unit 1	99.44%
Unit 2	99.72%
Unit 3	99.94%

It is considered that these values are typical for previous years. Computer maintenance is scheduled during unit outages rather than during unit operation.

2.1.5c Capability to Install Hydrogen Purge Unit

The existing procedure for installation of the hydrogen purge unit at Oconee has been reviewed. Areas in which preplanning can be accomplished are being identified. These include shielding requirements and personnel access requirements. A portion of this effort is being conducted in conjunction with item 2.1.6B, "Design Review of Plant Shielding." Upon identification of these shielding requirements interim methods for shielding can be established.

2.1.6a Integrity of Systems Outside Containment Likely to Contain Radioactive Materials

In response to this item a review of the Oconee design was conducted. Systems were identified which would or could, without deliberate operator action, contain highly radioactive fluids. The systems identified the allowable leakage rates, and basis of these rates are as follows:

<u>System</u>	<u>Leak Rate</u>	<u>Basis</u>
1. High Pressure Injection (High Pressure Injection Recirculation)	1 GPM	Contiguous with RCS whose allowable leak- age is 1 GPM (Techni- cal Specification 3.1.6)
a. Makeup		
b. Letdown		

	<u>System</u>	<u>Leak Rate</u>	<u>Basis</u>
	c. Recirculation		
	d. Purification Demineralizers		
	e. Deborating Demineralizers		
	f. attached portions of chemical addition and sampling		
2.	Low Pressure Injection (Residual Heat Removal)	2 GPH	Technical Specification 4.5.4
3.	Reactor Building Spray (Containment Spray Recirculation)	1 GPH	Contiguous with LPI System
4.	Coolant Storage a. Coolant Treatment	1 GPM	Contiguous with RCS and HPI
5.	Liquid Waste Disposal	3 gpm	Note 1
6.	Gaseous Waste Disposal	34 cfm	Note 1

NOTE 1: These allowable leakage rates have been established based on operating experience with these systems. Efforts will be made to reduce identified leakage to as low as practicable and subsequently reducing the allowable leakage rates.

Procedures are available at the station to perform leakage tests on the above systems. They have revised to reflect the above criteria and to delete the parenthetical expression "depending on the size and nature of the leak."

2.1.8a Post-Accident Sampling Capability

The existing sampling procedure has been revised to specify required actions if a primary sample is required and a nuclear accident has occurred and fuel damage is suspected. The procedure includes notification and involvement of health physics personnel, determination of radiation levels and use of shielding as required.

This procedure is available onsite for NRC review.

2.1.8c Improved Inplant Iodine Monitoring

Sample cartridges are used at Oconee. A determination of airborne iodine level can be made by utilization of a portable gamma monitor. A 'go' or 'no go' level has been established that allows the appropriate personnel to determine iodine concentration.