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DENTON,H.R. Office of Nuclear Reactor Regulation, Director
STOLZ,J.F. Operating Reactors Branch 4

SUBJECT: Forwards response to Franklin Research Ctr draft technical evaluation rept re control of heavy loads,supplementing 821008 response to NRC 820218 request.

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November 5, 1982

Mr. Harold R. Denton, Director
Office of Nuclear Reactor Regulation
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Attention: Mr. John F. Stolz, Chief
Operating Reactors Branch No. 4

Subject: Oconee Nuclear Station
Docket Nos. 50-269, -270, -287

Dear Sir:

In response to your letter of February 18, 1982, please find attached the remainder of our response to the Draft Technical Evaluation Report (DTER) by Franklin Research Center on control of heavy loads at Oconee Nuclear Station. This submittal supplements my letter of October 8, 1982 and completes our response to your request.

Very truly yours,

Hal B. Tucker

JFN/php
Attachment

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OCONEE NUCLEAR STATION
Response to FRC-TER
Control of Heavy Loads
(NUREG-0612)

Second Submittal

Introduction

This report provides additional information on the load handling systems at Oconee Nuclear Station as requested by the NRC in their letter dated February 18, 1982, concerning the Franklin Research Center Technical Evaluation Report (FRC-TER).

Section 5.1 (General Provisions for Load Handling)

Guideline 1 Recommendation

Clarify procedures for the handling of deviations from established load paths.

Response

Clarifications of procedures will not be necessary due to the fact that established load paths provide the only path with sufficient clearance for a heavy load. If it became necessary, a procedure would be written for the deviation. Laydown areas have already been designated by Design Engineering for load consideration on the turbine floor. This should be sufficient justification for deviation for the turbine parts, because if not placed properly the floor is not able to withstand the load.

Guideline 5 Recommendation a

Verify that sling selection and marking are based on the sum of the maximum static and dynamic loads.

Response

Slings are being checked to ensure that tags are attached and show load capacity. Riggers Handbook and ANSI 30.2 are given as guides for load capacities.

Guideline 5 Recommendation b

Verify that slings restricted in use to certain cranes are clearly marked to so indicate.

Response

No slings are restricted to use on a particular crane. Some slings and lifting devices are designated for certain lifts to be made on equipment. These slings are stored on nearby racks and are not generally used for any other lifts.

Guideline 7 Recommendation a

Evaluate crane design at the Oconee Nuclear Station for compliance with the 13 items identified in the FRC evaluation of CMAA-70.

Supplemental Response to Item 2 - Longitudinal Stiffeners

This submittal covers the two items which were not addressed in "Item 2" of our previous response to the NRC on the FRC-TER.

These items/questions are:

Are longitudinal stiffeners used on crane bridge girders located in accordance with applicable sections of CMAA-70?

Are longitudinal stiffeners used on bridge girders sized in accordance with applicable sections of CMAA-70?

Location of Longitudinal Stiffeners

Longitudinal stiffeners used on crane girders at Oconee Nuclear Station lie in close proximity to their required locations. A small variation in stiffener location does not significantly affect the capacity of a girder. Therefore, the locations of longitudinal stiffeners used on the subject cranes are adequate.

Size of Longitudinal Stiffeners

Longitudinal stiffeners used on Turbine Aisle, Auxiliary Turbine Aisle and Heater Bay cranes at Oconee Nuclear Station were sized in accordance with the AISE No. 6, 1949 "Specification for Electric Overhead Traveling Cranes for Steel Mill Service". This Specification was the accepted Industry Standard at the time our cranes were built and provides an adequate margin of safety in stiffener design.

Longitudinal stiffeners found on the Polar Cranes at Oconee meet the moment of inertia requirements set forth in CMAA-70.

Section 5.3 (Interim Protection Measures)

Response to Interim Measure 2 and 6 Recommendations

Oconee Nuclear Station has previously implemented most of the provisions of these interim measures. The last remaining provision (concerning inspections of load bearing components) is being implemented at this time.