

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

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 FACIL: 50-269 Oconee Nuclear Station, Unit 1, Duke Power Co. 05000269
 50-270 Oconee Nuclear Station, Unit 2, Duke Power Co. 05000270
 50-287 Oconee Nuclear Station, Unit 3, Duke Power Co. 05000287

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 DENTON, H.R. Office of Nuclear Reactor Regulation, Director
 STOLZ, J.F. Operating Reactors Branch 4

SUBJECT: Application for exemption from inservice insp requirements
 of Section XI of ASME Boiler & Pressure Vessel Code, w/Class
 III & I license fees.

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HAL B. TUCKER
VICE PRESIDENT
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August 30, 1982

Mr. Harold R. Denton, Director
Office of Nuclear Reactor Regulation
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Attention: Mr. John F. Stolz, Chief
Operating Reactors Branch No. 4

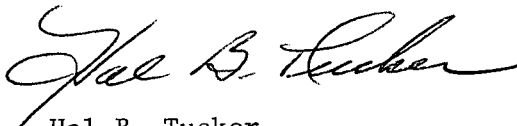
Subject: Oconee Nuclear Station
Docket Nos. 50-269, -270, -287

Dear Sir:

Pursuant to 10 CFR 50, §50.55a, please find attached a request for relief from the inservice inspection requirements of Section XI of the ASME Boiler and Pressure Vessel Code. The attached request concerns piping of the Reactor Building spray system and the Reactor Building sump lines.

This request for relief has been determined to consist of one Class III and two Class I license fees. Accordingly, please find attached a check in the amount of \$4,800.

Very truly yours,



Hal B. Tucker

JFN/php
Attachment

A047
w/ check
\$ 4,800

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PDR ADOCK 05000269
Q PDR

DUKE POWER COMPANY
OCONEE NUCLEAR STATION
Request for Relief from ASME Section XI
Volumetric Inspection Requirements

I. Component for Which Relief Is Requested:

a. Name and Number

Piping from valves BS-1 and BS-2 to containment spray nozzles.
Piping from reactor building emergency sump to valves LP-19 and LP-20. (Typical all three units)

b. Function

Containment Heat Removal

c. ASME Section III Code Class

Class 2

d. Valve Category

NA

II. ASME Code Section XI Requirement That Has Been Determined to Be Impractical:

ASME Boiler and Pressure Vessel Code Section XI, 1974 Edition, including 1975 Summer Addenda:

Paragraph 1WC-1220(a), which states that components may be exempted from examination if the design temperature is 200°F or less, and the design pressure is 275 psig or less.

Table 1WC-2520, examination category C-F requires volumetric examination of 12½ percent of total number of circumferential butt welds exceeding 4" NPS at structural discontinuities.

III. Basis for Requesting Relief:

1. Design conditions listed for the containment spray headers from valves BS-1 and BS-2 to the spray nozzles are 200 psig at 300°F. Design conditions for the Reactor Building emergency sump suction lines are building pressure at 300°F. The design pressures are, thus, within the limits for exemption under 1WC-1220(a). The 300°F design temperature was applied due to the expected maximum temperature of 250°F in the Reactor Building sump following a Design Basis Accident. The Reactor Building spray lines and Reactor Building sump lines have not been subjected to the design conditions noted and would only be operated in excess of 200°F for approximately 24 hours following a Design Basis Accident.

III. (cont'd)

2. The Reactor Building emergency sump lines are embedded in concrete and are inaccessible for the required examination.

The piping of the Reactor Building Spray System from valves BS-1 and BS-2, which runs vertically and adjacent to the reactor building wall to the spray nozzles at the building dome, makes the required examination difficult and dangerous to perform.

Therefore, we request that the subject piping be considered exempt from the requirements of Section XI, paragraph IWC-2520.

IV. Alternate Examination:

No volumetric or surface examination will be performed on this piping.

V. Implementation Schedule:

This request will apply to the first ten-year interval for Oconee Units 1, 2, and 3.