

# CATEGORY 1

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR: 9703070207	DOC. DATE: 97/03/03	NOTARIZED: NO	DOCKET #
FACIL: 50-269	Oconee Nuclear Station, Unit 1, Duke Power Co.		05000269
50-270	Oconee Nuclear Station, Unit 2, Duke Power Co.		05000270
50-287	Oconee Nuclear Station, Unit 3, Duke Power Co.		05000287

AUTH. NAME	AUTHOR AFFILIATION
HAMPTON, J.W.	Duke Power Co.
RECIP. NAME	RECIPIENT AFFILIATION
	Document Control Branch (Document Control Desk)

SUBJECT: Forwards Rev 1 to Request for Relief ONS-009, including editorial change, deleting weld identification numbers. W/one oversize drawing.

DISTRIBUTION CODE: A047D COPIES RECEIVED: LTR 1 ENCL 1 SIZE: 5  
TITLE: OR Submittal: Inservice/Testing/Relief from ASME Code - GL-89-04

NOTES:

	RECIPIENT ID CODE/NAME	COPIES LTR ENCL	RECIPIENT ID CODE/NAME	COPIES LTR ENCL
	PD2-2 LA	1 1	PD2-2 PD	1 1
	LABARGE, D	1 1		
INTERNAL:	ACRS	1 1	AEOD/SPD/RAB	1 1
	<u>FILE CENTER 01</u>	1 1	NRR/DE/EMEB	1 1
	NUDOCS-ABSTRACT	1 1	OGC/HDS2	1 0
	RES/DET/EIB	1 1	RES/DET/EMMEB	1 1
EXTERNAL:	LITCO ANDERSON	1 1	NOAC	1 1
	NRC PDR	1 1		

*Drawings Available in Central Files*

NOTE TO ALL "RIDS" RECIPIENTS:  
PLEASE HELP US TO REDUCE WASTE! CONTACT THE DOCUMENT CONTROL DESK,  
ROOM OWFN 5D-5 (EXT. 415-2083) TO ELIMINATE YOUR NAME FROM  
DISTRIBUTION LISTS FOR DOCUMENTS YOU DON'T NEED!

TOTAL NUMBER OF COPIES REQUIRED: LTR 14 ENCL 13

*NAB*

C  
A  
T  
E  
G  
O  
R  
Y  
  
1  
  
D  
O  
C  
U  
M  
E  
N  
T

Duke Power Company  
Oconee Nuclear Site  
P.O. Box 1439  
Seneca, SC 29679

J. W. HAMPTON  
Vice President  
(864)885-3499 Office  
(864)885-3564 Fax



**DUKE POWER**

March 3, 1997

U.S. Nuclear Regulatory Commission  
Attention Document Control Desk  
Washington, DC 20555

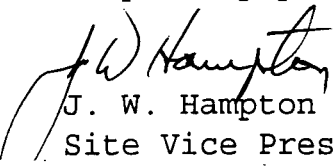
Subject: Duke Power Company  
Oconee Nuclear Station, Units 1, 2, and 3  
Docket No. 50-269, -270, -287  
Third Ten Year Inservice Inspection Interval  
Request for Relief ONS-009

Request for Relief ONS-009 was previously submitted to the NRC in a Duke letter dated June 14, 1994. This request for relief addresses the High Pressure Injection System letdown coolers for Units 1, 2, and 3. The NRC approved this request for relief in a Safety Evaluation Report (SER) to Duke dated November 15, 1995.

Please find attached Revision 1 to Request for Relief ONS-009, which includes an editorial change which deletes the weld identification numbers. The weld identification numbers are redundant to the code item numbers. Weld identification numbers are subject to change due to replacement of like-for-like components. This editorial revision does not affect the basis for relief granted in the November 15, 1995, SER since letdown cooler joint configuration and examination accessibility have not been affected.

If there are any questions or further information is needed you may contact D. A. Nix at (864) 885-3634.

Very truly yours,

  
J. W. Hampton  
Site Vice President

9703070207 970303  
PDR ADOCK 05000269  
G PDR

Attachment

070067



*Drawing Available in Central Files*

*A0471*

U. S. Nuclear Regulatory Commission

March 3, 1997

Page 2

xc (w/attch):           Mr. D. E. LaBarge  
                          Project Manager  
                          Office of Nuclear Reactor Regulation  
                          U. S. Nuclear Regulatory Commission  
                          Washington, DC 20555

                          Mr. L. A. Reyes  
                          Regional Administrator, Region II  
                          U. S. Nuclear Regulatory Commission

xc (w/o attch):         Mr. M. A. Scott  
                          Senior NRC Resident Inspector  
                          Oconee Nuclear Station

                          Mr. Max Batavia  
                          Bureau of Radiological Health  
                          SC Dept. of Health & Environmental Control  
                          2600 Bull St.  
                          Columbia, SC 29201

DUKE POWER COMPANY  
Request for Relief From  
Inservice Inspection Requirement

Station: Oconee

Unit: 1, 2 & 3

Requesting Department: Nuclear Generation

Reference Code: ASME Section XI, 1989 Edition , no addenda

I. Component for which exemption is requested:

a. Name and Identification Number:

Letdown Cooler Nozzles ( Inside Radius Section) for Units 1, 2 & 3  
OM-201-3107 (Attachment "A"). The following item numbers are affected:

<u>Oconee 1</u>	<u>Oconee 2</u>	<u>Oconee 3</u>
<u>Item No.</u>	<u>Item No.</u>	<u>Item No.</u>
B03.160.001	B03.160.001	B03.160.001
B03.160.002	B03.160.002	B03.160.002
B03.160.003	B03.160.003	B03.160.003
B03.160.004	B03.160.004	B03.160.004

b. Function:

The Letdown Cooler reduces the temperature of the letdown flow from the  
Reactor Coolant System to a temperature suitable for demineralization.

c. ASME Section XI Code Class:  
Class 1

d. Construction Code and Class (If Applicable):

N/A

e. Valve Category (If Applicable):

N/A

II. Reference Code Requirement that has been determined to be impractical:

Table IWB-2500, Examination Category B-D, Item Number B03.160. Table requires  
that an inside radius volumetric examination be performed on heat exchanger nozzles.

III. Basis for Requesting Relief:

Due to the size and geometry of the nozzle inside radius on the Letdown Coolers, we have been unable to perform a meaningful (i.e. unable to get sound into the area of interest) volumetric examination.

IV. Alternate Examination:

Perform the volumetric examination on the weld volume, as required by ASME Section XI, Table IWB-2500-1, Examination Category B-D, Item Number B03.150. This will provide adequate Assurance of the welded connection. The alternate proposed inservice testing will provide an acceptable level of quality and safety and ensures the level of public health and safety is not reduced.

V. Implementation Schedule:

Oconee 1		Oconee 2		Oconee 3	
<u>Item No.</u>	<u>RFO</u>	<u>Item No.</u>	<u>RFO</u>	<u>Item No.</u>	<u>RFO</u>
B03.160.001	16	B03.160.001	15	B03.160.001	17
B03.160.002	16	B03.160.002	15	B03.160.002	17
B03.160.003	20	B03.160.003	20	B03.160.003	21
B03.160.004	20	B03.160.004	20	B03.160.004	21

Evaluated By: RM Rouse Date 2/10/97

Reviewed By: J. Barlow Date 2/11/97