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SUBJECT: Requests NRC approval for util to use recent Rev 3 to CROV computer code (approved by NRC on 950420).

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**DUKE POWER**

May 17, 1995

U. S. Nuclear Regulatory Commission  
Washington, D. C. 20555

Attention: Document Control Desk

Subject: Oconee Nuclear Station  
Docket Numbers 50-269, -270, and 287  
McGuire Nuclear Station  
Docket Numbers 50-369 and -370  
Catawba Nuclear Station  
Docket Numbers 50-413 and -414  
Request for Approval by Duke Power Company of CROV

On March 7-10, 1995, the NRC audited Duke Power Company's use of the TACO3 computer code (Reference 1) and the Fuel Rod Gas Pressure Criterion (Ref. 2), both of which are NRC-approved methodologies developed by B&W Fuel Company (BWFC). The NRC concluded in their Safety Evaluation (Ref. 3) that Duke has the technical capability to perform TACO3 analyses for reload licensing applications for Duke's Oconee, McGuire, and Catawba Nuclear Stations. TACO3 serves as input to the CROV computer code to analyze fuel cladding performance. During the audit it was noted that a revision to the CROV computer code was under review, and was expected to be approved in the near future. The suggestion was made by one of the auditors that the NRC be notified when Duke was prepared to transition to the revised CROV computer code. The NRC could then determine whether: a) another audit or other demonstration of Duke's use of the code was appropriate, or b) the NRC could accept Duke's use of the revised CROV code without further review.

The purpose of this letter is to request approval for use by Duke Power Company of the recent revision (Revision 3) to the CROV computer code (Reference 4), which was approved by the NRC on April 20, 1995. Duke currently uses Revision 2 of the code (Reference 5) in conjunction with the fuel rod creep collapse methodology approved in DPC-NE-1002A (Reference 6, for Oconee Nuclear Station) and DPC-NE-2001P-A (Reference 7, for McGuire and Catawba Nuclear Stations). As part of the TACO3 audit, the NRC reviewed Duke's workplace procedure which prescribes requirements for

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U. S. Nuclear Regulatory Commission

May 17, 1995

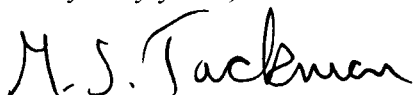
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applying an externally licensed methodology and/or computer code. In accordance with this workplace procedure, a methodology report will be developed by July 31, 1995 which describes how the revised CROV computer code and BWFC creep collapse methodology will be used by Duke. Duke will not make any changes to the NRC-approved code and methodology of Reference 4.

Duke plans to begin using the revised CROV code for reload licensing analyses for all three nuclear stations beginning with Oconee 1 Cycle 17, which is scheduled to start up on December 3, 1995. A creep collapse analysis will be completed by July 31, 1995. In keeping with this schedule, it is requested that the NRC notify Duke within 30 days (June 16, 1995) whether any additional review or audit is considered necessary by the NRC to accept Duke's use of the CROV computer code.

If there are any questions, please call Scott Gewehr at (704) 382-7581.

Very truly yours,



M. S. Tuckman

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Oconee Nuclear Station

## References

1. BAW-10162P-A, TACO3 - Fuel Pin Thermal Analysis Code, B&W Fuel Company, November, 1989
2. BAW10183P, Fuel Rod Gas Pressure Criterion, B&W Fuel Company, August 1991
3. Safety Evaluation, H. N. Berkow to M. S. Tuckman, Duke Power's Use of TACO3 and the Fuel Rod Gas Pressure Criterion for the Oconee, McGuire, and Catawba Nuclear Stations
4. BAW-10084, Rev. 3, Program to Determine In-Reactor Performance of B&W Fuels-Cladding Creep Collapse, Babcock & Wilcox, July, 1991
5. BAW-10084, Rev. 2, Program to Determine In-Reactor Performance of B&W Fuels-Cladding Creep Collapse, Babcock & Wilcox, October 1978
6. DPC-NE-1001A, Duke Power Company Oconee Nuclear Station Reload Design Methodology, October, 1985
7. DPC-NE-2001P-A, Fuel Mechanical Reload Methodology for Mark-BW Fuel, October, 1990