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 AUTH. NAME: PARKER, W.O. AUTHOR AFFILIATION: Duke Power Co.
 RECIP. NAME: DENTON, H.R. RECIPIENT AFFILIATION: Office of Nuclear Reactor Regulation, Director
 REID, R.W. Operating Reactors Branch 4

SUBJECT: Application for amend to Licenses DPR-38, DPR-47 & DPR-55, changing Tech Specs & license conditions per NUREG-0578, in response to NRC 800702 request. Discussion of applicability of STS & Class I & III fees encl.

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S.L.

DUKE POWER COMPANY

POWER BUILDING

422 SOUTH CHURCH STREET, CHARLOTTE, N. C. 28242

WILLIAM O. PARKER, JR.
VICE PRESIDENT
STEAM PRODUCTION

October 2, 1980

TELEPHONE: AREA 704
373-4083

Mr. Harold R. Denton, Director
Office of Nuclear Reactor Regulation
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Attention: Mr. R. W. Reid, Chief
Operating Reactors Branch No. 4

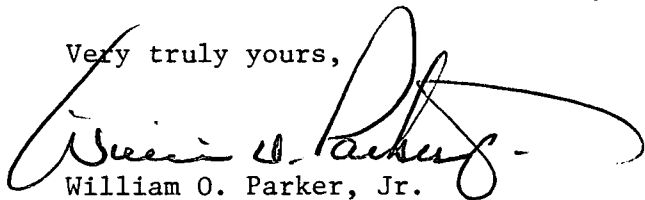
Subject: Oconee Nuclear Station
Docket Nos. 50-269, -270, -287

Dear Sir:

In a letter dated July 2, 1980, the NRC Staff requested that applicable specifications and license conditions be submitted as a proposed license amendment for Staff approval. In response to this request please find attached a "Discussion of Applicability of Model Standard Technical Specifications" (Attachment 1) and a "Proposed Technical Specification Revision and Facility Operating License Condition Amendment to Incorporate NUREG-0578 Category A Requirements" (Attachment 2).

Pursuant to 10CFR 170, §170.22 this proposed amendment is considered to consist of one Class III and two Class I license amendments. Therefore, please find a check in the amount of \$4,800.

Very truly yours,


William O. Parker, Jr.

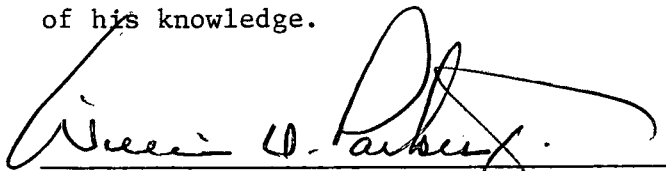
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Attachments

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\$ 4800.00

8010090422

Mr. Harold R. Denton, Director
October 2, 1980
Page Two

WILLIAM O. PARKER, JR., being duly sworn, states that he is Vice President of Duke Power Company; that he is authorized on the part of said Company to sign and file with the Nuclear Regulatory Commission this request for amendment of the Oconee Nuclear Station Technical Specifications, Appendix A to Facility Operating Licenses DPR-38, DPR-47 and DPR-55; and that all statements and matters set forth therein are true and correct to the best of his knowledge.



William O. Parker, Jr., Vice President

Subscribed and sworn to before me this 2nd day of October, 1980.



Notary Public

My Commission Expires:

Feb 15, 1982

ATTACHMENT 1

DUKE POWER COMPANY
OCONEE NUCLEAR STATION

Response to NRC Letter of July 2, 1980

Discussion of Applicability of Model Standard Technical Specifications

INSTRUMENTATION 3/4.3.2 ENGINEERED SAFETY FEATURE ACTUATION SYSTEM

INSTRUMENTATION

1. The operability requirements for Engineered Safety Feature (ESF) instrumentation are contained in Oconee Specification 3.5.1. The proposed revision to this specification, which reflects the diverse containment isolation feature, is attached.
2. The emergency feedwater initiation features are contained in the attached proposed revision to Oconee Specification 3.4.
3. The trip setpoints for ESF instrumentation are contained in Oconee Specification 3.5.3. The proposed revision to this specification, which reflects the diverse containment isolation feature, is attached.
4. Emergency feedwater pumps are actuated by pressure switches which sense low control oil pressure and low pump discharge pressure when the main feedwater pumps trip. It is considered that the establishment of trip setpoints in the Technical Specifications in this instance is unnecessary. Thus, no changes to the Oconee Technical Specifications are provided in response to this item.
5. The surveillance requirements for ESF instrumentation are contained in Oconee Specification 4.1, Table 4.1-1. The proposed revision to this specification, which reflects the diverse containment isolation feature, is attached.
6. Emergency feedwater pump automatic start and automatic valve actuation functional test surveillance requirements are contained in the attached proposed revision to Specification 4.1, Table 4.1-2.
7. The ESF instrument response times are provided in the Oconee FSAR and have been reflected in existing surveillance procedures. It is considered that inclusion of specific response times in Technical Specifications is unnecessary and thus, no changes to the Oconee Specifications are proposed.

INSTRUMENTATION 3/4.3.3.6 ACCIDENT MONITORING INSTRUMENTATION

1. Pressurizer Water Level - Surveillance requirements for this instrument are currently contained in Oconee Specification 4.1, Table 4.1-1, Item 26 and are consistent with the STS surveillance requirements.

In the event the pressurizer water level instrument system, or a part thereof, becomes inoperable, station operating procedures, as required by Oconee Specification 6.4.1, specify corrective actions to be taken to assure safe operation of the unit.

In consideration of the above, no changes to the Oconee Technical Specifications are deemed necessary in response to this item.

2. Emergency Feedwater Flow Rate - Operability requirements for this instrument are provided in the attached proposed revision to Oconee Specification 3.4.

Surveillance requirements for this instrument are provided in the attached proposed revision to Oconee Specification 4.1.

3. RCS Subcooling Margin Monitor - Operability requirements for this instrument are provided in the attached proposed revision to Oconee Specification 3.1.12.

Surveillance requirements for this instrument are provided in the attached proposed revision to Oconee Specification 4.1. The input devices to this instrument, reactor coolant outlet temperature and pressure, are currently included in Specification 4.1 and are tested monthly and calibrated each refueling. In consideration of the above, the inclusion of a functional test of the subcooling monitor is sufficient to assure satisfactory operation of this instrument.

4. PORV and Safety Valve Position Indicators - Surveillance requirements for these instruments are provided in the attached proposed revision to Oconee Specification 4.1. In the event a position indicator becomes inoperable, station operating procedures, as required by Oconee Specification 6.4.1, specify corrective actions to be taken to assure safe operation of the unit.

REACTOR COOLANT SYSTEM 3/4.4.3 RELIEF VALVES

1. Power operated relief valves (PORV's) and the associated block valves will be determined to be operable in accordance with the valve testing requirements of the inservice test program required by 10CFR 50, §50.55a and ASME Boiler and Pressure Vessel Code, Section XI.
2. The pressurizer power-operated relief valve (PORV) in each Oconee unit is a DC solenoid-operated pilot valve. The PORV block valve in each Oconee unit is an AC motor-operated valve. The power supplies for the PORV's and their associated block valves are therefore independent and diverse. Power is available to the PORV's solenoids from the 125-volt DC instrument and control battery power system. Battery chargers are provided and are powered from safety-grade MCC's which are capable of being powered from both the offsite power system and the onsite emergency power system. Power is available to the block valves through non-load-shed load centers which are capable of being powered from both the offsite power system and the onsite emergency power system. No manual transfer of motive or control power for these valves is required. These power systems are further described in Section 8 of the Oconee FSAR.
3. In consideration of the above, no changes to the Oconee Technical Specifications are deemed necessary in response to this item.

REACTOR COOLANT SYSTEM 3/4.4.4 PRESSURIZER

1. The requirement to periodically check pressurizer level currently exists in Oconee Technical Specification 4.1, Table 4.1-1, Item 26. Pressurizer level instrumentation is checked each shift.
2. The Station Operating Procedures required by Specification 6.4.1 specify the required pressurizer water level.
3. The pressurizer heaters for each Oconee unit are supplied from four non-safety-related motor control centers (MCCs). These MCCs are in turn powered via load centers from the 4160-volt Engineered Safeguard (ESG) buses. The MCCs and their associated load centers are divided among the three 4160-volt

ESG buses such that the loss of one entire 4160-volt switchgear will not preclude the capability to supply sufficient pressurizer heaters to maintain natural circulation under hot standby conditions.

The MCCs and their associated load centers are not automatically shed from the 4160-volt ESG buses under either blackout of LOCA conditions. Therefore, power is available to the heaters from either the offsite power system or the onsite emergency power system and the transferring of heater loads or reapplying of heater loads is not required. The emergency power sources available to each Oconee unit including their capacity and capability are described in Section 8.2.3 of the Oconee FSAR.

4. In consideration of the above, no changes to the Oconee Technical Specifications are deemed necessary in response to this item.

CONTAINMENT SYSTEMS 3/4.6.4 CONTAINMENT ISOLATION VALVES

1. All containment isolation valves are verified operable in accordance with the valve testing requirements of the Inservice Test Program required by 10CFR 50, §50.55a and ASME Boiler and Pressure Vessel Code, Section XI.
2. Oconee Specification 3.6.3 currently contains the action requirements in the event that a containment isolation valve becomes inoperable.
3. The specific types of valves listed in STS Table 3.6-1 are currently included in the Oconee Pump and Valve Inservice Test Program submitted for Staff approval May 15, 1979.
4. In consideration of the above, no changes to the Oconee Technical Specifications are deemed necessary in response to this item.

PLANT SYSTEMS 3/4.7.1.2 AUXILIARY FEEDWATER SYSTEM

1. Oconee emergency feedwater system operability requirements are contained in proposed revision to Oconee Specification 3.4 which is attached.
2. Emergency feedwater pumps and automatic valves are determined operable in accordance with Oconee Specification 4.9 and the testing requirements of the Inservice Test Program required by 10CFR 50, §50.55a and ASME Boiler and Pressure Vessel Code, Section IX.
3. Functional testing of automatic valve actuation and emergency feedwater pump starting is contained in proposed revision to Oconee Specification 4.9 which is attached.

ADMINISTRATIVE CONTROLS 6.3 UNIT STAFF QUALIFICATIONS

1. The staff organization requirements are contained in Oconee Specification 6.1.1. The proposed revision to this specification which reflects the Shift Technical Advisor functions is attached.