UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION II
101 MARIETTA STREET, N.W.
ATLANTA, GEORGIA 30303

Report Nos.: 50-269/78-34, 50-270/78-32 and 50-287/78-34

Docket Nos.: 50-269, 50-270 and 50-287

License Nos.: DPR-38, DPR-47 and DPR-55

Licensee: Duke Power Company
422 South Church Street
Charlotte, North Carolina 28242

Facility Name: Oconee Units 1, 2 and 3

Inspection at: Oconee Site, Seneca, South Carolina

Inspection conducted: December 6-8, 1978

Inspector: J. E. Ouzts

Approved by: _____ 12/18/78
P. J. Kellogg, Chief Date
Nuclear Support Section No. 2
Reactor Operations and Nuclear Support Branch

Inspection Summary

Inspection on December 6-8, 1978 (Report Nos. 50-269/78-34, 50-270/78-32 and 50-287/78-34)
Areas Inspected: Routine, unannounced inspection of plant procedures for Units 1, 2 and 3. The inspection involved 15 inspector hours on site by one NRC inspector.
Result: Of the area inspected, no items of noncompliance or deviations were identified.

RII Report Nos. 50-269/78-34,
50-270/78-32 and 50-287/78-34 I-1

DETAILS I

Prepared by: J. E. Ouzts 12/18/78
J. E. Ouzts, Reactor Inspector Date
Nuclear Support Section No. 2
Reactor Operations and Nuclear
Support Branch

Dates of Inspection: December 6-8, 1978

Reviewed by: P. H. Kellogg 12/18/78
P. H. Kellogg, Chief Date
Nuclear Support Section No. 2
Reactor Operations and Nuclear
Support Branch

1. Persons Contacted

*J. E. Smith, Station Manager
*R. T. Bond, Licensing Project Engineer
*R. M. Koehler, Superintendent Technical Services
*K. J. Brackett, Station Senior QA Engineer
*J. D. Norton, Corporate QA Engineer
*D. J. Vito, Assitant Engineer
Four storage vault and file room clerks were also contacted.

*Denotes those attending exit interview.

2. Licensee Action on Previous Inspection Findings

No inspected

3. Unresolved Items

Unresolved items are matters about which more information is required in order to ascertain whether they are acceptable items, items of noncompliance or deviations. No unresolved items were identified during this inspection.

4. Exit Interview

The inspector met with licensee representatives (denoted in paragraph 1) at the conclusion of the inspection on December 6-8, 1978. The inspector summarized the scope and findings of the inspection as discussed in paragraph 5. The licensee acknowledged the inspector's findings as discussed in these details.

5. Plant Procedures Verification

The inspector conducted a review of the following plant procedures:

Operating Procedures

-OP-0-A-1205-01- Preparation for Refueling

-OP-3-A-1102-02- Reactor Trip Recovery

-OP-1-A-1106-01- Turbine Generator

-OP-0-A-1510-07- Control Rod Cluster Configuration Loading and Shipment

-OP-1-A-1104-02- High Pressure Injection System Operational Procedure

-OP-2-A-1103-05- Pressurizer Operation

-OP-3-A-1103-06- Reactor Coolant Pump Operation

-IP-0-A-0301-03T- Power Range Calibration at Power

-IP-2-A-0305-03A- Reactor Protection System - Channel A - On Line

Abnormal (Alarm)

-2/1701/41- Control Rod Drive Sequence Fault

-2/1702/01- High Pressure Loop "A" Emergency Flow Low

-2/1704/16- Reactor Building Emergency Hatch Inner/Outer Door Open

-2/1702/39- Reactor Coolant Pressure High/Low

-2/1702/38- Letdown Tank Pressure High/Low

-2/1702/20- Reactor Coolant Pressure Low

-2/1701/08- N15 High Flux Trip

-2/1701/07- Channel A - Reactor Projection System/Flux Trip

Emergency Procedures

-EP-0-A-1800-7- Steam Generator Tube Failure

-EP-0-A-1800-29- Loss of Instrument Air

-EP-0-A-1800-26- Loss of Component Cooling

-EP-A-1800-23- Loss of Containment Integrity

-EP-1-A-1500-02- Fire (PIE)

Maintenance Procedures

-MP-0-A-1300-07- High Pressure Injection Pump Removal and Replacement

-MP-0-A-1400-03- Removal and Installation of Type "A" Electrical Penetrations

-MP-0-A-1800-17- Disassembly of Letdown Cooler and Test for Leaks

-MP-0-A-3000-21- Pressurizer Safety Valve Setpoint Testing

-1P-0-A-0305-05A- Reactor Protection System Channel "A" Reactor Building High Pressure Trip

Administrative Procedures

-3.7.1- Containment Access Control During Refueling and Major Maintenance

-3.1.1- Tagging and Safety

-3.1.7- Access to Containment

Temporary Procedures

-TO/0/A/1107/03- Temporary Drumming Procedure for Evaporator Bottoms

-TO/0/A/3001/01- Disconnect DC Output Cables on Exide 600 Ampere Battery Charger

The above procedures were reviewed to verify that:

- Reviews, approvals and changes covering the activities were in accordance with Technical Specifications.
- Temporary Procedures were not in conflict with Technical Specifications
- Where required procedure changes were made to reflect changes required by selected technical specification revisions
- Safety reviews were made and recorded in conformance with 10 CFR 50.59(a) and (b)
- Procedure contents were in accordance with Technical Specification requirements

- Selected procedure contents were adequate to control required safety related operations.

Safety-related systems and components which may be exposed to a freezing environment were reviewed with the licensee. He reported no systems or components which were exposed to freezing that were not protected by heating. In addition prior to freezing weather a checkoff procedure is put into use to check the operability of these systems and components.

The inspector used one or more of the following acceptance criteria for evaluating the above items used in the procedure review.

- Regulatory Guide 1.33
- ANSI N 18.7 (1976)
- ANSI N 45.2.9(1) (1971)
- VEPCO Topical Report (17.0)
- Administrative Policy Manual - (3.1.4) (3.1.5) (3.1.6) (4.1) and (4.3)
- Corporate Quality Assurance Program
- Inspector Judgement

Within the areas inspected no items of noncompliance or deviations were identified.