

# UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D.C. 20555-0001

August 4, 2015

Mr. K. Henderson Site Vice President Catawba Nuclear Station Duke Energy Carolinas, LLC 4800 Concord Road York, SC 29745

SUBJECT:

CATAWBA NUCLEAR STATION, UNIT 1 – REQUEST FOR ADDITIONAL

**INFORMATION REGARDING RELIEF REQUEST 15-CN-001** 

(TAC NO. MF5875)

Dear Mr. Henderson:

By letter dated March 19, 2015 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML15082A074), Duke Energy Carolinas (Duke) submitted a relief request for the Catawba Nuclear Station, Unit 1. The proposed relief would allow the implementation of an alternate repair method for a steam leak. Duke has provided supplements to the submittal dated May 4, 2015, and July 16, 2015 (ADAMS Accession Nos. ML15127A170 and ML15201A499, respectively).

The U.S. Nuclear Regulatory Commission staff has reviewed the licensee's submittal and determined that additional information is needed in order to complete our review. The enclosed document describes this request for additional information (RAI). On July 30, 2015, the Duke staff indicated that a response to the RAI would be provided within 30 days of the date of this letter.

If you have any questions, please call me at 301-415-2481.

Sincerely,

G. Edward Miller, Project Manager

Plant Licensing Branch II-1

Division of Operating Reactor Licensing Office of Nuclear Reactor Regulation

Docket No. 50-413

Enclosure:

Request for Additional Information

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## REQUEST FOR ADDITIONAL INFORMATION REGARDING

## CATAWBA NUCLEAR STATION, UNIT 1

### RELIEF REQUEST 15-CN-001

## ALTERNATE REPAIR FOR FLEX HOSE STEAM LEAK

#### **DOCKET NO. 50-413**

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- 1. The request stated that it is proposing an alternative to the requirements of ASME Code Section XI, Appendix IX, Article IX-1000, General, (c) (2), in order to use a mechanical clamping device on a system that forms part of the containment boundary. The licensee also stated that the clamping device will be designed to Appendix IX, Article IX-3000. The NRC requests that Duke verify that it will also meet all of the other requirements of Appendix IX.
- The request stated that the injection valve to be installed is ASME Code Class 1. The NRC requests that the applicant describe the code class of the other materials that will be used.
- 3. The request stated that sealant will be injected into the piping between the root valves. The NRC requests that the licensee provide the temperature rating of the sealant and verify that the sealant does not contain contaminates, such as fluorides or chlorides, that could be detrimental to stainless steel pressure boundary components.
- 4. The request stated that the clamping device is designed to Appendix IX, Article IX-3000, The NRC requests that Duke verify that the piping system, with the clamp installed and sealant injected, has been analyzed for and meets all normal operating loads, seismic loads and accident conditions.<sup>1</sup>

<sup>1</sup> Question 4 was identified as a result of the discussion of the draft RAIs.

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G. Edward Miller, Project Manager Plant Licensing Branch II-1 Division of Operating Reactor Licensing Office of Nuclear Reactor Regulation

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