

September 8, 1986

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DOCKET NO(S). 50-269, 50-270 and 50-287

Mr. Hal B. Tucker
Vice President - Nuclear Production
Duke Power Company
P.O. Box 33189
422 South Church Street
Charlotte, North Carolina 28242
SUBJECT: OCONEE NUCLEAR STATION, UNITS NOSS 1, 2 AND 3

The following documents concerning our review of the subject facility are transmitted for your information.

- Notice of Receipt of Application, dated _____.
- Draft/Final Environmental Statment, dated _____.
- Notice of Availability of Draft/Final Environmental Statement, dated _____.
- Safety Evaluation Report, or Supplement No. _____, dated _____.
- Notice of Hearing on Application for Construction Permit, dated _____.
- Notice of Consideration of Issuance of Facility Operating License, dated _____.
- Monthly Notice; Applications and Amendments to Operating Licenses Involving no Significant Hazards Considerations, dated _____.
- Application and Safety Analysis Report, Volume _____.
- Amendment No. _____ to Application/SAR dated _____.
- Construction Permit No. CPPR- _____, Amendment No. _____ dated _____.
- Facility Operating License No. _____, Amendment No. _____, dated _____.
- Order Extending Construction Completion Date, dated _____.

Other (Specify) Notice of Consideration of Issuance of Amendments to Facility Operating Licenses and Proposed No Significant Hazards Consideration Determination and Opportunity for Hearing concerning amendment application dated June 30, 1986, as superseded September 2, 1986, for Oconee 2 Cycle 9 operation.

1st C. McCracken
Office of Nuclear Reactor Regulation
Helen N. Pastis, Project Manager
PWR Project Directorate #6
Division of PWR Licensing-B

Enclosures:
As stated

cc: w/enclosure:
See next page

OFFICE	PBD#6 RIngram	PBD#6 HPastis;cf	PBD#6 SWest	PBD#6 CMcCracken	PBD#6 GEdison	PBD#6 JStolz
SURNAME	RIngram	HPastis;cf	SWest	CMcCracken	GEdison	JStolz
DATE	9/8/86	9/8/86	9/8/86	9/8/86	9/8/86	9/8/86

September 8, 1986

DOCKET NO. 50-269, 50-270, and 50-287

MEMORANDUM FOR: Rules and Procedures Branch
Division of Rules and Records
Office of Administration

FROM: Office of Nuclear Reactor Regulation

SUBJECT: OCONEE NUCLEAR STATION, UNITS NOS. 1, 2 AND 3

One signed original of the *Federal Register* Notice identified below is enclosed for your transmittal to the Office of the Federal Register for publication. Additional conformed copies (5) of the Notice are enclosed for your use.

- Notice of Receipt of Application for Construction Permit(s) and Operating License(s).
- Notice of Receipt of Partial Application for Construction Permit(s) and Facility License(s); Time for Submission of Views on Antitrust Matters.
- Notice of Consideration of Issuance of Amendment to Facility Operating License.*
- Notice of Receipt of Application for Facility License(s); Notice of Availability of Applicant's Environmental Report; and Notice of Consideration of Issuance of Facility License(s) and Notice of Opportunity for Hearing.
- Notice of Availability of NRC Draft/Final Environmental Statement.
- Notice of Limited Work Authorization.
- Notice of Availability of Safety Evaluation Report.
- Notice of Issuance of Construction Permit(s).
- Notice of Issuance of Facility Operating License(s) or Amendment(s).
- Order.
- Exemption.
- Notice of Granting Exemption.
- Environmental Assessment.
- Notice of Preparation of Environmental Assessment.
- Other: *Please insert date on the 5th page 1st paragraph of this notice for a 30-day intervention period, and call Caryn on extension 27288 to inform her of the date inserted. Referenced documents have been provided PDR.

Division of PWR Licensing 9B
Office of Nuclear Reactor Regulation

Enclosure:
As stated

Contact: j
Phone:

NOTE: Expedited handling is requested. Publication in the FEDERAL REGISTER no later than 9/11/86 is required to provide a 30-day notice period as required by the Commission's regulations and to avoid adverse impact on plant operations.

OFFICE									
SURNAME									PBD#6
DATE	9/8/86								RIngram /86

Mr. H. B. Tucker
Duke Power Company

Oconee Nuclear Station
Units Nos. 1, 2 and 3

cc:

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Charlotte, North Carolina 28242

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Babcock & Wilcox
Nuclear Power Generation Division
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Bethesda, Maryland 20814

Manager, LIS
NUS Corporation
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Clearwater, Florida 33515

Senior Resident Inspector
U.S. Nuclear Regulatory Commission
Route 2, Box 610
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Regional Administrator
U.S. Nuclear Regulatory Commission
101 Marietta Street, N.W.
Suite 3100
Atlanta, Georgia 30303

Mr. Heyward G. Shealy, Chief
Bureau of Radiological Health
South Carolina Department of Health
and Environmental Control
2600 Bull Street
Columbia, South Carolina 29201

Office of Intergovernmental Relations
116 West Jones Street
Raleigh, North Carolina 27603

Honorable James M. Phinney
County Supervisor of Oconee County
Walhalla, South Carolina 29621

UNITED STATES NUCLEAR REGULATORY COMMISSIONDUKE POWER COMPANYDOCKETS NOS. 50-269, 50-270 AND 50-287NOTICE OF CONSIDERATION OF ISSUANCE OF AMENDMENTS TO
FACILITY OPERATING LICENSES AND PROPOSED NO SIGNIFICANT HAZARDS
CONSIDERATION DETERMINATION AND OPPORTUNITY FOR HEARING

The U.S. Nuclear Regulatory Commission (the Commission) is considering issuance of amendments to Facility Operating Licenses Nos. DPR-38, DPR-47 and DPR-55, issued to Duke Power Company (the licensee), for operation of the Oconee Nuclear Station, Units Nos. 1, 2 and 3, located in Oconee County, South Carolina.

In accordance with the licensee's application dated June 30, 1986, as superseded in its entirety September 2, 1986, the proposed amendments would revise the Station's common Technical Specifications (TSs) to support the operation of Oconee Unit 2 at full rated power during the upcoming Cycle 9. The proposed amendment request changes the following areas:

1. Core Protection Safety Limits (TS 2.1);
2. Protective System Maximum Allowable Setpoints (TS 2.3);
3. Rod Position Limits (TS 3.5.2); and
4. Power Imbalance Limits (TS 3.5.2).

To support the license amendment request for operation of Oconee Unit 2, Cycle 9, the licensee submitted, as an attachment to the application, a Duke Power Company (DPC) Report, DPC-RD-2007, "Oconee Unit 2, Cycle 9 Reload Report,"

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dated June 1986. A summary of the Cycle 9 operating parameters is included in the report, along with safety analyses.

During the refueling outage, 117 fuel assemblies will be reinserted similar to those previously used, and 60 fuel assemblies will be discharged and replaced with new, but substantially similar, assemblies of the Mark BZ type. As in the previous cycle, Cycle 9 will utilize gray (less-absorbing) axial power shaping rods (APSRs) instead of the previously used black (highly-absorbing) APSRs. The use of the Mark BZ fuel assemblies and the gray APSRs was approved by the Commission's staff for use at Oconee Unit 1 during Cycle 9, in amendments dated November 23, 1984.

Before issuance of the proposed license amendments, the Commission will have made findings required by the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations.

The Commission has made a proposed determination that the amendment request involves no significant hazards consideration. Under the Commission's regulations in 10 CFR 50.92, this means that operation of the facility in accordance with the proposed amendments would not (1) involve a significant increase in the probability or consequences of an accident previously evaluated; or (2) create the possibility of a new or different kind of accident from any accident previously evaluated; or (3) involve a significant reduction in a margin of safety.

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The Commission has provided guidance concerning the application of the standards in 10 CFR 50.92 by providing certain examples (51 FR 7750).

Example (iii) of the types of amendments not likely to involve significant hazards considerations is an amendment to reflect a core reload where:

- (1) no fuel assemblies significantly different from those found previously acceptable to the Commission for a previous core at the facility in question are involved;
- (2) no significant changes are made to the acceptance criteria for the Technical Specifications;
- (3) the analytical methods used to demonstrate conformance with the Technical Specifications and regulations are not significantly changed; and
- (4) the NRC has previously found such methods acceptable.

This particular reload involves the reinsertion of 117 fuel assemblies of a type previously approved and used and the insertion of 60 fuel assemblies of the Mark BZ type. The Mark BZ fuel assemblies are the same as previously approved and used assemblies in terms of fuel rods, end grid, end fittings, and guide tubes and differ only slightly from previously approved assemblies in the use of Zircaloy spacer grids rather than Inconel Intermediate Spacer grids. Thus, this core reload involves the use of fuel assemblies that are not significantly different from those found previously acceptable to the Commission for a previous core at this facility. The

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request for amendment changes the TSS to reflect new operating limits based on the fuel and control rods to be inserted into the core. These parameters are based on the new physics of the core and fall within the acceptance criteria.

In the analyses supporting this reload, there have been no significant changes in the acceptance criteria for the Technical Specifications, the analytical methods used to demonstrate conformance with the Technical Specifications and the regulations were not significantly changed, and those analytical methods have been previously found acceptable. Thus, this reload and the proposed license amendments reflecting it appear to be encompassed by example (iii) of amendments not likely to involve a significant hazards consideration. On this basis, the Commission proposes to determine that these amendments do not involve significant hazards considerations.

The Commission is seeking public comments on this proposed determination. Any comments received within 30 days after the date of publication of this notice will be considered in making any final determination. The Commission will not normally make a final determination unless it receives a request for a hearing.

Written comments may be submitted to the Rules and Procedures Branch, Division of Rules and Records, Office of Administration, U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, and should cite the publication date and page number of this FEDERAL REGISTER notice. Copies of comments received may be examined at the NRC Public Document Room, 1717 H Street, N.W., Washington, D.C.

By October 14, 1986 , the licensee may file a request for a hearing with respect to issuance of the amendments to the subject facility operating licenses and any person whose interest may be affected by this proceeding and who wishes to participate as a party in the proceeding must file a written petition for leave to intervene. Request for a hearing and petitions for leave to intervene shall be filed in accordance with the Commission's "Rules of Practice for Domestic Licensing Proceedings" in 10 CFR Part 2. If a request for a hearing or petition for leave to intervene is filed by the above date, the Commission or an Atomic Safety and Licensing Board, designated by the Commission or by the Chairman of the Atomic Safety and Licensing Board Panel, will rule on the request and/or petition and the Secretary or the designated Atomic Safety and Licensing Board will issue a notice of hearing or an appropriate order.

As required by 10 CFR §2.714, a petition for leave to intervene shall set forth with particularity the interest of the petitioner in the proceeding and how that interest may be affected by the results of the proceeding. The petition should specifically explain the reasons why intervention should be permitted with particular reference to the following factors: (1) the nature of the petitioner's right under the Act to be made a party to the proceeding; (2) the nature and extent of the petitioner's property, financial, or other interest in the proceeding; and (3) the possible effect of any order which may be entered in the proceeding on the petitioner's interest. The petition should also identify the specific aspect(s) of the subject matter of the proceeding

as to which petitioner wishes to intervene. Any person who has filed a petition for leave to intervene or who has been admitted as a party may amend the petition without requesting leave of the Board up to fifteen (15) days prior to the first prehearing conference scheduled in the proceeding, but such an amended petition must satisfy the specificity requirements described above.

Not later than fifteen (15) days prior to the first prehearing conference scheduled in the proceeding, a petitioner shall file a supplement to the petition to intervene which must include a list of the contentions which are sought to be litigated in the matter, and the bases for each contention set forth with reasonable specificity. Contentions shall be limited to matters within the scope of the amendments under consideration. A petitioner who fails to file such a supplement which satisfies these requirements with respect to at least one contention will not be permitted to participate as a party.

Those permitted to intervene become parties to the proceeding, subject to any limitations in the order granting leave to intervene, and have the opportunity to participate fully in the conduct of the hearing, including the opportunity to present evidence and cross-examine witnesses.

If a hearing is requested, the Commission will make a final determination on the issue of no significant hazards consideration. The final determination will serve to decide when the hearing is held.

If the final determination is that the amendment request involves no significant hazards consideration, the Commission may issue the amendments and make them effective, notwithstanding the request for a hearing. Any hearing held would take place after issuance of the amendments.

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If the final determination is that the amendments involve a significant hazards consideration, any hearing held would take place before the issuance of any amendments.

Normally, the Commission will not issue the amendments until the expiration of the 30-day notice period. However, should circumstances change during the notice period such that failure to act in a timely way would result, for example, in derating or shutdown of the facility, the Commission may issue the license amendments before the expiration of the 30-day notice period, provided that its final determination is that the amendments involve no significant hazards consideration. The final determination will consider all public and State comments received. Should the Commission take this action, it will publish a notice of issuance and provide for opportunity for a hearing after issuance. The Commission expects that the need to take this action will occur very infrequently.

A request for a hearing or a petition for leave to intervene must be filed with the Secretary of the Commission, U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Docketing and Service Branch, or may be delivered to the Commission's Public Document Room, 1717 H Street, N.W., Washington, D.C., by the above date. Where petitions are filed during the last ten (10) days of the notice period, it is requested that the petitioner promptly so inform the Commission by a toll-free telephone call to Western Union at (800) 325-6000 (in Missouri (800) 342-6700). The Western Union operator should be given Datagram Identification Number 3737 and the following

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message addressed to John F. Stolz: petitioner's name and telephone number; date petition was mailed; plant name; and publication date and page number of this FEDERAL REGISTER notice. A copy of the petition should also be sent to the Office of the General Counsel-Bethesda, U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, and to J. Michael McGarry, III, Bishop, Liberman, Cook, Purcell and Reynolds, 1200 17th Street, N.W., Washington, D.C. 20036, attorney for the licensee.

Nontimely filings of petitions for leave to intervene, amended petitions, supplemental petitions and/or requests for hearing will not be entertained absent a determination by the Commission, the presiding officer or the Atomic Safety and Licensing Board, that the petition and/or request should be granted based upon a balancing of the factors specified in 10 CFR 2.714(a)(1)(i)-(v) and 2.714(d).

For further details with respect to this action, see the application for amendments dated June 30, 1986, as superseded in its entirety September 2, 1986, which is available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D.C., and at the Oconee County Library, 501 West Southbroad Street, Walhalla, South Carolina.

Dated at Bethesda, Maryland, this 8th day of September 1986.

FOR THE NUCLEAR REGULATORY COMMISSION



Gordon E. Edison, Acting Director
PWR Project Directorate #6
Division of PWR Licensing-B

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September 17, 1986

DOCKET NO(S). 50-269, 50-270, 50-287,
50-302, and 50-346

SUBJECT: OCONEE 1, 2 AND 3, CRYSTAL RIVER UNIT 3 AND DAVIS-BESSE UNIT 1

The following documents concerning our review of the subject facility are transmitted for your information.

- Notice of Receipt of Application, dated _____.
- Draft/Final Environmental Statment, dated _____.
- Notice of Availability of Draft/Final Environmental Statement, dated _____.
- Safety Evaluation Report, or Supplement No. _____, dated _____.
- Notice of Hearing on Application for Construction Permit, dated _____.
- Notice of Consideration of Issuance of Facility Operating License, dated _____.
- ^{Biweekly} ~~Monthly~~ Notice; Applications and Amendments to Operating Licenses Involving no Significant Hazards Considerations, dated September 10, 1986.
- Application and Safety Analysis Report, Volume _____.
- Amendment No. _____ to Application/SAR dated _____.
- Construction Permit No. CPPR- _____, Amendment No. _____ dated _____.
- Facility Operating License No. _____, Amendment No. _____, dated _____.
- Order Extending Construction Completion Date, dated _____.
- Other (Specify) _____

Office of Nuclear Reactor Regulation
PWR Project Directorate #6, PWR-B

Enclosures:
As stated

cc: w/enclosures:
See next page

OFFICE	PBD#6						
SURNAME	RIngram;cf						
DATE	9/16 /86						

Mr. H. B. Tucker
Duke Power Company

Oconee Nuclear Station
Units Nos. 1, 2 and 3

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Manager, LIS
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2600 Bull Street
Columbia, South Carolina 29201

Office of Intergovernmental Relations
116 West Jones Street
Raleigh, North Carolina 27603

Honorable James M. Phinney
County Supervisor of Oconee County
Walhalla, South Carolina 29621

Mr. W. S. Wilgus
Florida Power Corporation

Crystal River Unit No. 3 Nuclear
Generating Plant

cc:

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Senior Vice President
and General Counsel
Florida Power Corporation
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Office of Planning and Budget
Executive Office of the Governor
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Nuclear Plant Manager
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Mr. F. Alex Griffin, Chairman
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Babcock & Wilcox
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Resident Inspector
U.S. Nuclear Regulatory Commission
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Regional Administrator, Region II
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Atlanta, Georgia 30303

Mr. Allan Schubert, Manager
Public Health Physicist
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State of Florida
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Attorney General
Department of Legal Affairs
The Capitol
Tallahassee, Florida 32304

Mr. J. Williams
Toledo Edison Company

Davis-Besse Nuclear Power Station
Unit No. 1

cc:

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Program Director
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Ohio Department of Industrial Relations
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Mr. Harold Kohn, Staff Scientist
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President, Board of
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Regional Administrator, Region III
U.S. Nuclear Regulatory Commission
799 Roosevelt Road
Glen Ellyn, Illinois 60137

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RIngram

October 15, 1986

DOCKET NO(S). 50-269, 50-270, 50-287

See attached list of addressees

SUBJECT: OCONEE NUCLEAR STATION, UNITS NOS. 1, 2 AND 3

The following documents concerning our review of the subject facility are transmitted for your information.

- Notice of Receipt of Application, dated _____.
- Draft/Final Environmental Statment, dated _____.
- Notice of Availability of Draft/Final Environmental Statement, dated _____.
- Safety Evaluation Report, or Supplement No. _____, dated _____.
- Notice of Hearing on Application for Construction Permit, dated _____.
- Notice of Consideration of Issuance of Facility Operating License, dated _____.
- Monthly Notice; Applications and Amendments to Operating Licenses Involving no Significant Hazards Considerations, dated _____.
- Application and Safety Analysis Report, Volume _____.
- Amendment No. _____ to Application/SAR dated _____.
- Construction Permit No. CPPR- _____, Amendment No. _____ dated _____.
- Facility Operating License No. _____, Amendment No. _____, dated _____.
- Order Extending Construction Completion Date, dated _____.
- Other (Specify) Monthly Operating Reports for June 1986., July amd August, 1986

Office of Nuclear Reactor Regulation
PWR Project Directorate #6, PWR-B

Enclosures:
As stated

cc: w/enclosures:
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SURNAME	RIngram;cf						
DATE	8/11/86						

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October 16, 1986 -

DOCKET NO(S). 50-269, 50-270
and 50-287, 50-302, 50-312, 50-289
50-313 and 50-346

SUBJECT: OGONEE 1, 2 AND 3, CRYSTAL RIVER UNIT 3, ARKANSAS UNIT 1, RANCHO SECO, TMI-1
AND DAVIS-BESSE UNIT 1

The following documents concerning our review of the subject facility are transmitted for your information.

- Notice of Receipt of Application, dated _____.
- Draft/Final Environmental Statement, dated _____.
- Notice of Availability of Draft/Final Environmental Statement, dated _____.
- Safety Evaluation Report, or Supplement No. _____ dated _____.
- Environmental Assessment and Finding of No Significant Impact, dated _____.
- Notice of Consideration of Issuance of Facility Operating License or Amendment to Facility Operating License, dated _____.
- Bi-Weekly Notice; Applications and Amendments to Operating Licenses Involving No Significant Hazards Considerations, dated 10/8/86 [see page(s) 17, 18].
- Exemption, dated _____.
- Construction Permit No. CPPR-_____, Amendment No. _____ dated _____.
- Facility Operating License No. _____, Amendment No. _____ dated _____.
- Order Extending Construction Completion Date, dated _____.
- Monthly Operating Report for _____ transmitted by letter dated _____.
- Annual/Semi-Annual Report- _____
_____ transmitted by letter dated _____.

Enclosures:
As stated

Office of Nuclear Reactor Regulation
PWR Project Directorate #6
Division of PWR Licensing-B

cc:w/enclosureg:
See next page

OFFICE	PWR#6						
SURNAME	Ingram; eh						
DATE	10/20/86						

Bi-Weekly Notice; Application and Amendments to Operating Licenses Involving No Significant Hazards Considerations

I. Background

Pursuant to Pub. L. 97-415, the Nuclear Regulatory Commission (the Commission) is publishing this regular bi-weekly notice. Pub. L. 97-415 revised section 189 of the Atomic Energy Act of 1954, as amended (the Act), to require the Commission to publish notice of any amendments issued, or proposed to be issued, under a new provision of section 189 of the Act. This provision grants the Commission the authority to issue and make immediately effective any amendment to an operating license upon a determination by the Commission that such amendment involves no significant hazards consideration, notwithstanding the pendency before the Commission of a request for a hearing from any person.

This bi-weekly notice includes all amendments issued, or proposed to be issued, since the date of publication of the last bi-weekly notice which was published on September 24, 1986 (51 FR 33938), through September 29, 1986.

NOTICE OF CONSIDERATION OF ISSUANCE OF AMENDMENT TO FACILITY OPERATING LICENSE AND PROPOSED NO SIGNIFICANT HAZARDS CONSIDERATION DETERMINATION AND OPPORTUNITY FOR HEARING

The Commission has made a proposed determination that the following amendment requests involve no significant hazards consideration. Under the Commission's regulations in 10 CFR 50.92, this means that operation of the facility in accordance with the proposed amendments would not: (1) Involve a significant increase in the probability or consequences of an accident previously evaluated; or (2) create the possibility of a new or different kind of accident from any accident previously evaluated; or (3) involve a significant reduction in a margin of safety. The basis for this proposed determination for each amendment request is shown below.

The Commission is seeking public comments on this proposed determination. Any comments received within 30 days after the date of publication of this notice will be considered in making any final determination. The Commission will not normally make a final determination unless it receives a request for a hearing.

Comments should be addressed to the Rules and Procedures Branch, Division of Rules and Records, Office of

Administration, U.S. Nuclear Regulatory Commission, Washington, DC 20555.

By November 7, 1986, the licensee may file a request for a hearing with respect to issuance of the amendment to the subject facility operating license and any person whose interest may be affected by this proceeding and who wishes to participate as a party in the proceeding must file a written petition for leave to intervene. Requests for a hearing and petitions for leave to intervene shall be filed in accordance with the Commission's "Rules of Practice for Domestic Licensing Proceedings" in 10 CFR Part 2. If a request for a hearing or petition for leave to intervene is filed by the above date, the Commission or an Atomic Safety and Licensing Board, designated by the Commission or by the Chairman of the Atomic Safety and Licensing Board Panel, will rule on the request and/or petition and the Secretary or the designated Atomic Safety and Licensing Board will issue a notice of hearing or an appropriate order.

As required by 10 CFR 2.714, a petition for leave to intervene shall set forth with particularity the interest of the petitioner in the proceeding, and how that interest may be affected by the results of the proceeding. The petition should specifically explain the reasons why intervention should be permitted with particular reference to the following factors: (1) The nature of the petitioner's right under the Act to be made a party to the proceeding; (2) the nature and extent of the petitioner's property, financial, or other interest in the proceeding; and (3) the possible effect of any order which may be entered in the proceeding on the petitioner's interest. The petition should also identify the specific aspect(s) of the subject matter of the proceeding as to which petitioner wishes to intervene. Any person who has filed a petition for leave to intervene or who has been admitted as a party may amend the petition without requesting leave of the Board up to fifteen (15) days prior to the first prehearing conference scheduled in the proceeding, but such an amended petition must satisfy the specificity requirements described above.

Not later than fifteen (15) days prior to the first prehearing conference scheduled in the proceeding, a petitioner shall file a supplement to the petition to intervene which must include a list of the contentions which are sought to be litigated in the matter, and the bases for each contention set forth with reasonable specificity. Contentions shall be limited to matters within the scope of the amendment under consideration. A

petitioner who fails to file such a supplement which satisfies these requirements with respect to at least one contention will not be permitted to participate as a party.

Those permitted to intervene become parties to the proceeding, subject to any limitations in the order granting leave to intervene, and have the opportunity to participate fully in the conduct of the hearing, including the opportunity to present evidence and cross-examine witnesses.

If a hearing is requested, the Commission will make a final determination on the issue of no significant hazards consideration. The final determination will serve to decide when the hearing is held.

If the final determination is that the amendment request involves no significant hazards consideration, the Commission may issue the amendment and make it immediately effective, notwithstanding the request for a hearing. Any hearing held would take place after issuance of the amendment.

If the final determination is that the amendment involves a significant hazards consideration, any hearing held would take place before the issuance of any amendment.

Normally, the Commission will not issue the amendment until the expiration of the 30-day notice period. However, should circumstances change during the notice period such that failure to act in a timely way would result, for example, in derating or shutdown of the facility, the Commission may issue the license amendment before the expiration of the 30-day notice period, provided that its final determination is that the amendment involves no significant hazards consideration. The final determination will consider all public and State comments received before action is taken. Should the Commission take this action, it will publish a notice of issuance and provide for opportunity for a hearing after issuance. The Commission expects that the need to take this action will occur very infrequently.

A request for a hearing or a petition for leave to intervene must be filed with the Secretary of the Commission, U.S. Nuclear Regulatory Commission, Washington, DC, 20555, Attention: Docketing and Service Branch, or may be delivered to the Commission's Public Document Room, 1717 H Street, NW., Washington, DC, by the above date. Where petitions are filed during the last ten (10) days of the notice period, it is requested that the petitioner promptly so inform the Commission by a toll-free telephone call to Western Union at (800) 325-6000 (in Missouri (800) 342-6700).

The Western Union operator should be given Datagram Identification Number 3737 and the following message addressed to (*Branch Chief*): petitioner's name and telephone number; date petition was mailed; plant name; and publication date and page number of this Federal Register notice. A copy of the petition should also be sent to the Executive Legal Director, U.S. Nuclear Regulatory Commission, Washington, DC 20555, and to the attorney for the licensee.

Nontimely filings of petitions for leave to intervene, amended petitions, supplemental petitions and/or requests for hearing will not be entertained absent a determination by the Commission, the presiding officer or the presiding Atomic Safety and Licensing Board, that the petition and/or request should be granted based upon a balancing of factors specified in 10 CFR 2.714(a)(1)(i)-(v) and 2.714(d).

For further details with respect to this action, see the application for amendment which is available for public inspection at the Commission's Public Document Room, 1717 H Street, NW., Washington, DC, and at the local public document room for the particular facility involved.

Alabama Power Company, Docket Nos. 50-348 and 50-364, Joseph M. Farley Nuclear Plant, Unit Nos. 1 and 2, Houston County, Alabama

Date of amendments request: August 25, 1986.

Description of amendments request: The amendment would revise Technical Specifications (TS) Figure 2.1-1, Reactor Core Safety Limit Three Loops in Operation, which assumes a 5% steam generator (SG) tube plugging limit to a 10% tube plugging limit. Also, TS 3.2.2, Equation for Heat Flux Hot Channel Factor, $F_0(Z)$, Limiting Condition for Operation, would contain new q_c values of 2.32 vice 2.31 and 4.64 vice 4.62. The changes would be consistent with reanalyses performed in accordance with the Westinghouse 1981 ECCS Large Break Evaluation Model with BART and a generic assessment of model changes described in WCAP-6561-P-A, Addendum 3. To date the licensee reports 2.9% of SG tubes plugged on Unit 1 and 3.7% on Unit 2. This action is taken to add margin to the existing 5% SG tube plugging limit without risking any possible startup delays should more SG tubes require plugging during the October 1986 outage on Unit 1.

Basis for proposed no significant hazards consideration determination: To support the requested changes, the licensee provided an evaluation of the significant hazards consideration per 10

CFR 50.92. The licensee's analysis is restated as follows:

(1) The proposed changes will not increase the probability or consequences of an accident previously evaluated because the revised ECCS analysis provided in Attachment 3, which was performed to support these changes, has demonstrated that the acceptance criteria for 10 CFR 50.46 have been met. The proposed changes have also been demonstrated to have no impact on the non-LOCA DNB transients or RCS structural integrity. Therefore, the probability or consequences of an accident previously evaluated will not be increased.

(2) The proposed changes will not create the possibility of a new or different kind of accident from any accident previously evaluated because both changes consist of changes to assumptions in previously evaluated accidents. Additionally, the increase in steam generator tube plugging has been evaluated for impact on RCS average temperature, thermal design flow and secondary side pressure and determined to have no impact on current plant operating limits for these parameters. Furthermore, the increase in the steam generator tube plugging limit will have no effect on RCS structural integrity. Thus, these proposed changes will not create the possibility of a new or different kind of accident from any accident previously evaluated.

(3) The proposed changes will not involve a reduction in a margin of safety because RCS structural integrity is maintained and the revised ECCS analysis has demonstrated the requirements of 10 CFR 50.46 are met. Additionally, the calculated peak clad temperature from this revised analysis is even less than the existing analysis and provides additional margin to the limit of 2200°F. Therefore, these proposed changes will not involve a reduction in a margin of safety.

On the basis of the NRC staff's preliminary review of the licensee's analysis, we agree that the action is a no significant hazards consideration. The Commission examples (51 FR 7751) of actions not likely to involve a significant hazards consideration include example "(vi) A change which either may result in some increase to the probability or consequences of a previously-analyzed accident or may reduce in some way a safety margin, but where the results of the change are clearly within all acceptable criteria with respect to the system or component specified in the Standard Review Plan: for example, a change resulting from the application of a small refinement of a previously used calculational model or design method," which seems to fit this proposed change. The proposed change includes an analysis assuming the new 10% SG tube plugging limit. The analysis indicates that peak clad fuel temperatures would remain within the allowable limits for the large break LOCA analysis of 10 CFR Part 50, Appendix K. Further, the

accident analysis indicates that the Standard Review Plan criteria of section 15.8.5 would be met. The non-LOCA transients will have no impact on DNB since the 10% SG tube plugging will not decrease the coolant flow below the thermal design flow. The safety margin remains within peak clad temperature limits and is probably reduced due to the use of the BART code methodology. It is expected that the NRC staff safety evaluation will agree with the licensee's conclusions. Therefore, we propose to determine that the amendment does not involve a significant hazards consideration.

Local Public Document Room location: George S. Houston Memorial Library, 212 W. Burdeshaw Street, Dothan, Alabama 36303.

Attorney for licensee: Ernest L. Blake, Esquire, 1800 M Street, NW., Washington, DC 20036.

NRC Project Director: Lester S. Rubenstein.

Arkansas Power and Light Company, Docket No. 50-313, Arkansas Nuclear One, Unit No. 1, Pope County, Arkansas

Date of amendment request: September 10, 1986.

Description of amendment request: The amendment would:

(a) permit operation of Arkansas Nuclear One, Unit 1 (ANO-1) for Cycle 8 in accordance with the licensee's application for amendment dated September 10, 1986. The design cycle length would be 425 effective full power days (EFPD). The amendment would change Figure 3.2-1 to provide acceptable boron concentration levels slightly greater than current levels in order to assure cold shutdown capability required for Cycle 8 operation, change Figure 3.5.2-5 to provide acceptable maximum linear heat rates such that the maximum cladding temperature will not exceed 10 CFR 50, Appendix K Final Acceptance Criteria for Cycle 8 operation, change Figures 3.5.2-1(A-D), 3.5.2-2(A-D), and 3.5.2-3(A-D) to provide acceptable rod positions versus power level to ensure shutdown margin requirements of Specification 3.5.2.1 and power peaking criteria are met for Cycle 8 operation, change Figures 3.5.2-6(A-D) to provide acceptable Axial Power Shaping Rod (APSR) positions at any given power level for Cycle 8 operation, change Figures 3.5.2-4(A-D) to provide acceptable operational power imbalance setpoints at any given power level for Cycle 8 operation, and change Specifications 3.5.2.4 and 3.5.2.5 to remove the 92% full power hold requirement for equilibrium xenon.

(b) change Specification 4.7.1.1 to revise the acceptable insertion time for a tripped control rod from 1.46 seconds to the original 1.66 seconds. A penalty of 0.20 seconds was added to the tripped control rod acceptable insertion time criteria to offset a potential rod bow effect from irradiation growth of the fuel rods because bowing of the fuel rods may interfere with the insertion rate of a tripped control rod. The tripped control rod insertion time is measured during a refueling outage prior to restart, and the penalty was added to insure that even if rod bowing occurred during the operating cycle, the control rods would still insert quickly enough to maintain the departure from nucleate boiling ratio (DNBR) safety margins. In support of the increase in the acceptable control rod trip insertion time, the licensee referenced the Babcock and Wilcox (B&W) Topical Report BAW-10147P, "Fuel Rod Bowing in Babcock & Wilcox Fuel Designs", dated April, 1981. The NRC Safety Evaluation of BAW-10147P is dated February 15, 1983.

Basis for proposed no significant hazards consideration determination: The proposed changes have been reviewed against each of the criteria in 10 CFR 50.92, namely that the proposed changes would not:

- (1) Involve a significant increase in the probability or consequences of an accident previously evaluated; or
- (2) Create the possibility of a new or different kind of accident from any accident previously evaluated; or
- (3) Involve a significant reduction in a margin of safety.

With regard to (1) above for Item (b), 1.66 seconds is the amount of time assumed for a tripped control rod to insert in the analyses of the Final Safety Analysis Report (FSAR). Thus, the FSAR analyses remain applicable. Therefore, increasing the allowable tripped control rod insertion time from 1.46 seconds to the original 1.66 seconds does not increase the probability or consequences of an accident previously evaluated.

With regard to (2) above for Item (b), there are no changes to the configuration or operability of the control rods or control rod drive system. Also, the function of the control rods will not change. Therefore, increasing the allowable tripped control rod insertion time from 1.46 seconds to the original 1.66 seconds does not create the possibility of a new or different kind of accident from any accident previously evaluated.

With regard to (3) above for Item (b), the NRC Safety Evaluation of BAW-10147P concludes that rod bow due to irradiation growth is not a concern in

the B&W fuel assemblies utilized by the licensee, thus a rod bow penalty is not necessary. Therefore, increasing the allowable tripped control rod insertion time from 1.46 seconds to the original 1.66 seconds does not reduce a margin of safety.

For Item (a), the Commission has provided guidance concerning the application of the criteria in 10 CFR 50.92 by providing certain Examples (51 FR 7750). One of the examples (iii) of actions involving no significant hazards considerations is for a nuclear power reactor, a change resulting from a nuclear reactor core reloading, if no fuel assemblies significantly different from those found previously acceptable to the NRC for a previous core at the facility in question are involved. This assumes that no significant changes are made to the acceptance criteria for the technical specifications, that the analytical methods used to demonstrate conformance with the technical specifications and regulations are not significantly changed, and that NRC has previously found such methods acceptable.

The licensee has stated that the proposed amendment would permit operation for Cycle 8 with fuel that is not significantly different from that used in previous cycles. The mechanical design of the fuel assemblies in Cycle 8 is unchanged from Cycle 7. There are no significant changes in the nuclear design of Cycle 8. The thermal-hydraulic design evaluation remains bounded by the FSAR, and the thermal performance of the core during accidents and transients for the Cycle 8 reload remains within the bounds of previously accepted analyses. Also, there have been no significant changes in the acceptance criteria for the Technical Specifications.

On these bases, the Commission has made a proposed determination that the application for amendment involves no significant hazards considerations.

Local Public Document Room location: Tomlinson Library, Arkansas Tech University, Russellville, Arkansas 72801.

Attorney for licensee: Nicholas S. Reynolds, Bishop, Liberman, Cook, Purcell and Reynolds, 1200 17th Street, NW., Suite 700, Washington, DC 20036.

NRC Project Director: John F. Stolz.

Boston Edison Company, Docket No. 50-293, Pilgrim Nuclear Power Station, Plymouth County, Massachusetts

Date of amendment request: December 23, 1985, as revised September 15, 1986.

Description of amendment request: By letter dated December 23, 1985, the

licensee proposed an amendment to change the Technical Specifications relative to the licensee's Nuclear Safety Review and Audit Committee (NSRAC). The proposed amendment was previously noticed on February 12, 1986 (51 FR 5271). The original request has now been revised by substituting the title "Chief Operating Officer" for "Senior Vice-President, Nuclear." This change is being made to recognize that the position of Senior Vice-President, Nuclear has been eliminated and its authority and responsibilities have been transferred to a new position of higher authority, the Chief Operating Officer.

Basis for proposed no significant hazards consideration determination: This additional change to the Technical Specifications is administrative and does not physically affect plant related systems. Therefore, this change would not: (1) Involve a significant increase in the probability or consequences of an accident previously evaluated, or (2) create the possibility of a new or different kind of accident from any accident previously evaluated, or (3) involve a significant reduction in a margin of safety. Based on this finding, the staff has made an initial determination that the proposed amendment does not involve significant hazards considerations.

Local Public Document Room
location: Plymouth Public Library, 11 North Street, Plymouth, Massachusetts 02360.

Attorney for licensee: W.S. Stowe, Esq., Boston Edison Company, 800 Boylston Street, 36th Floor, Boston, Massachusetts 02199.

NRC Project Director: John A. Zwolinski.

Carolina Power & Light Company,
Dockets Nos. 50-325 and 50-324,
Brunswick Steam Electric Plant, Units 1 and 2, Brunswick County, North Carolina

Date of application for amendments:
August 29, 1986.

Description of amendment request:
The proposed amendment would change the Technical Specifications (TS) for Brunswick Steam Electric Plant, Units 1 and 2. The proposed change to TS Section 4.6.3.2 would permit the operability of primary containment isolation valves listed in TS Table 3.6.3-1 to be verified while the reactor is in operational conditions other than cold shutdown or refueling.

Technical Specification 4.6.3.2 requires that each primary containment isolation valve listed in TS Table 3.6.3-1 be demonstrated operable during COLD SHUTDOWN or REFUELING at least once every 18 months. This operability

test should verify that the valve actuates to the appropriate position upon receipt of a test signal. This requirement limits the operational flexibility of the plant for those valves capable of being tested during power operation. The Brunswick Updated Final Safety Analysis Report (FSAR), paragraph 7.3.1.1.9, states the following:

The primary containment isolation and NSSS Shutoff System is testable during reactor operation. Isolation valves can be tested to assure that they are capable of closing by operating manual switches in the Control Room and observing the position lights and any associated process effects.

The proposed revision to the Brunswick Technical Specifications would delete the phrase "during COLD SHUTDOWN or REFUELING" from Surveillance Requirement 4.6.3.2. This revision would allow primary containment isolation valves to be tested and demonstrated operable where such testing is feasible during power operation. The testing will normally be done in conjunction with logic system functional tests for the instrumentation associated with a given isolation valve.

Basis for proposed no significant hazards consideration determination: The Commission has provided standards for determining whether a significant hazards determination exists as stated in 10 CFR 50.92(c). A proposed amendment to an operating license involves no significant hazards considerations if operation of the facility in accordance with the proposed amendment would not: (1) Involve a significant increase in the probability or consequences of an accident previously evaluated, or (2) create the possibility of a new or different kind of accident from any accident previously evaluated, or (3) involve a significant reduction in a margin of safety.

The licensee has evaluated the proposed amendment against the standards in 10 CFR 50.92 and has determined the following:

1. The proposed amendment does not involve a significant increase in the probability or consequences of an accident previously evaluated. As noted in the Brunswick Updated FSAR, the Brunswick primary containment isolation system (including isolation valves) was designed to be testable during reactor operation. Therefore, the level of assurance of valve operability is not affected by conducting the testing during plant operation.

2. The proposed amendment does not create the possibility of a new or different kind of accident from any accident previously evaluated because

the proposed change does not affect the design of any safety systems. In addition, the performance of any safety functions is not affected since the surveillance testing is intended to actuate the containment isolation valves to their appropriate isolation position. Because the isolation valves are designed to be testable during plant operation, no new plant transients will be introduced by the proposed change.

3. The proposed amendment does not involve a significant decrease in a margin of safety. The testing of containment isolation valves with the unit in operation would allow the test conditions to more closely reflect the operating conditions under which the isolation valves are expected to perform their safety function. This can be especially important where thermal expansion and system pressures can affect valve performance. Therefore, the margin of safety may actually be increased if certain containment isolation valves are tested with the unit in operation.

Based on the above reasoning, the licensee has determined that the proposed amendment does not involve a significant hazards consideration.

The NRC staff has reviewed the licensee's no significant hazards consideration determination and agrees with the licensee's analysis. Based on this review, the staff therefore proposes to determine that the proposed amendment does not involve a significant hazards consideration.

Local Public Document Room
location: Auburn Public Library, 118 15th Street, Auburn, Nebraska 68305.

Attorney for licensee: Thomas A. Baxter, Esquire, Shaw, Pittman, Potts and Trowbridge, 1800 M Street, NW., Washington, DC 20036.

NRC Project Director: Daniel R. Muller.

Commonwealth Edison Company,
Docket No. STN-50-454, Byron Station,
Unit 1, Ogle County, Illinois

Date of amendment request: August 13, 1986.

Description of amendment request:
The amendment would revise Technical Specification Section 3/4.7.5 on pages 3/4 7-13 and 3/4 7-14 to replace "86% of total volume" with "50%" for the water level in the ultimate heat sink (UHS) cooling tower basin.

The minimum water volume in the basin is not being changed by this amendment. The licensee intends to replace the existing instrument with an instrument with greater range; therefore, 86% on the old instrument corresponds exactly with 50% on the new instrument.

The licensee wants to use an instrument with greater range so that the physical water level in the basin can be increased and still be read on the instrument. The increased water level, which is above the 100% reading of the existing instrument, is desirable because it provides more margin to the level at which the essential service water diesel driven pumps receive an auto start signal.

It is the staff's intention to apply this amendment to Byron Station, Unit 2, when it receives its operating license if the amendment is found acceptable for Byron Station, Unit 1.

Basis for proposed no significant hazards consideration determination: The staff has evaluated this proposed amendment and determined that it involves no significant hazards considerations. In accordance with the criteria of 10 CFR 50.92(c), the proposed amendment does not:

(1) Involve a significant increase in the probability or consequences of an accident previously evaluated because the proposed amendment does not alter the actual minimum water level of the Ultimate Heat Sink (UHS) cooling tower basin. The amendment merely revises the instrument indication in the control room for 873.75 feet Mean Sea Level (MSL) to read 50%. Changing the instrument indication to a different reference point does not increase the probability or consequences of a previously evaluated accident.

(2) Create the possibility of a new or different kind of accident from any accident previously evaluated because:

(a) the proposed amendment does not allow any new equipment or modes of operation which could initiate a new or different kind of accident from any previously evaluated because the actual minimum volume of water in the UHS cooling towers is not being changed. The change pertains only to instrumentation indication; therefore the possibility is unaltered.

(b) this is an administrative change which would merely change the control room indication for the UHS cooling towers to 50% when at 873.75 feet MSL. This change will allow operation above the minimum level without a constant high level alarm.

(3) Involve a significant reduction in the margin of safety, because there are no hardware changes associated with this proposed license amendment, nor in the manner that the UHS cooling towers are being operated. For these reasons, there is no reduction in the margin of safety as a result of the proposed license amendment.

Based on the preceding assessment, the staff proposes to determine that this

proposed amendment involves no significant hazards consideration.

Local Public Document Room
Location: Rockford Public Library, 215 N. Wyman Street, Rockford, Illinois 61103.

Attorney for licensee: Michael Miller, Isham, Lincoln and Beal, One First National Plaza, 42nd Floor, Chicago, Illinois 60603.

NRC Project Director: Vincent S. Noonan.

Commonwealth Edison Company,
Docket No. STN-50-454, Byron Station,
Unit 1, Ogle County, Illinois

Date of amendment request: August 27, 1986.

Description of amendment request: The amendment would revise Technical Specification section 3/4.8.2.1 on page 3/4 8-10; section 3/4.8.2.2 on page 3/4 8-13; and add a new section, section 3/4.8.2.1.3 on a new page, page 3/4 8-11a. These changes address operation of the D.C. crossties between Units 1 and 2 at Byron Station for two situations: (1) With both units operating and one battery charger failed, and (2) with one operating and the other unit shutdown with a battery and its associated battery charges out of service.

The staff intends to apply this amendment to Byron Station, Unit 2, when it receives its operating license if the amendment is found acceptable for Byron Station, Unit 1.

Basis for proposed no significant hazards consideration determination: The staff has evaluated this proposed amendment and determined that it involves no significant hazards considerations. According to 10 CFR 50.92(c), a proposed amendment to an operating license involves no significant hazards considerations if operation of the facility in accordance with the proposed amendment would not:

(1) Involve a significant increase in the probability or consequences of any accident previously evaluated; or

(2) Create the possibility of a new or different kind of accident from any accident previously evaluated; or

(3) Involve a significant reduction in a margin of safety.

This proposed amendment controls the use of the D.C. crosstie between opposite unit D.C. buses. Accidents previously evaluated assume a certain load profile on a D.C. bus. The D.C. bus loading, when using the crosstie, will be restricted so the capacity of the operating unit's battery will not be exceeded in the event of a single failure and simultaneous accident and loss of offsite power conditions. A single failure and simultaneous accident and loss of offsite power are the conditions assumed for a D.C. bus in previously

evaluated accidents. As a result, the probability or consequences of accidents previously evaluated are not changed by this proposed amendment.

The only new or different kind of accident which could be created by this proposed amendment would involve an interaction between the two D.C. buses which are crosstied. However, a breaker exists on either side of the crosstie which would isolate any potential short circuit from either unit. These breakers are coordinated with the D.C. bus main breaker to assure the crosstie will isolate from the affected D.C. bus before the battery would be isolated. All of these breakers are class 1E. For these reasons, a new or different kind of accident will not be created from this proposed amendment.

This proposed amendment will allow use of some margin in the capacity of the batteries which was allocated for future D.C. loads. However, no design margin in the batteries (i.e., aging or temperature correction factors) has been affected by this proposed amendment. Accordingly, no margin of safety has been reduced.

Based on the preceding assessment, the staff believes this proposed amendment involves no significant hazards considerations.

Local Public Document Room
Location: Rockford Public Library, 215 N. Wyman Street, Rockford, Illinois 61103.

Attorney for licensee: Michael Miller, Isham, Lincoln & Beal, One First National Plaza, 42nd Floor, Chicago, Illinois 60603.

NRC Project Director: Vincent S. Noonan.

Commonwealth Edison Company,
Docket No. STN 50-454, Byron Station,
Unit 1 Ogle County, Illinois

Date of application for amendment: September 10, 1986.

Description of amendment request: The amendment would revise several areas of Section 6.0, Administrative Controls, of the Technical Specifications. The changes have been requested to reflect a recent reorganization of the Byron Station management.

The staff intends to apply this amendment to Byron Station, Unit 2, when it receives its operating license if the amendment is found acceptable for Byron Station, Unit 1.

Basis for proposed no significant hazards consideration determination: The staff has evaluated this proposed amendment and determined that it involves no significant hazards considerations. In accordance with the

criteria set forth in 10 CFR 50.92(c), the proposed amendment does not:

(1) Involve a significant increase in the probability of consequences of an accident previously evaluated because the proposed amendment merely revises the Commonwealth Edison on-site and off-site organizational structure as found in the Byron Station Technical Specifications. This has no impact on plant design or operations; hence, the probability or consequences of previously evaluated accidents are unaltered.

(2) Create the possibility of a new or different kind of accident previously evaluated because the proposed amendment does not introduce any new equipment or modes of operation in Byron Station that could create the possibility of a new or different kind of accident from that which was previously evaluated.

(3) Involve a significant reduction in the margin of safety, because these changes are considered to be administrative. There are no changes being made to hardware or in the manner that plant systems are being operated as a result of this license amendment. Therefore, the margin of safety is not being compromised or changed.

Based on the preceding assessment, the staff believes this proposed amendment involves no significant hazards considerations.

Local Public Document Room
Location: Rockford Public Library, 215 N. Wyman Street, Rockford, Illinois 61103.

Attorney for licensee: Michael Miller, Isham, Lincoln and Beal, One First National Plaza, 42nd Floor, Chicago, Illinois 60603.

NRC Project Director: Vincent S. Noonan.

Commonwealth Edison Company,
Docket Nos. 50-237/249, Dresden Nuclear Power Station, Unit Nos. 2 and 3, Grundy County, Illinois

Date of amendment request: January 20, 1986 as supplemented by a letter dated July 29, 1986.

Description of amendment request:
The amendments proposed in the January 20, 1986 submittal primarily involved typographical errors, changes in nomenclature, sentence structure and references with the exception of a change for Dresden Unit 3 to allow post-maintenance testing of control rod drives in the refuel mode with low pressure cooling systems inoperable. This latter provision was approved for Dresden Unit 2 in Amendment 6 to DPR-19.

The July 29, 1986 submittal proposes additional changes of a similar nature including revisions to certain tables to reflect the results of minor appropriate plant modifications recently implemented.

Basis for proposed no significant hazards consideration determination:

The Commission has provided standards for determining whether a significant hazards determination exists as stated in 10 CFR 50.92(c).

The licensee has presented its determination of no significant hazards consideration in its submittals as follows:

Commonwealth Edison has evaluated the proposed Technical Specification amendment and determined that it does not represent a significant hazards consideration. Based on the criteria for defining a significant hazards consideration established in 10 CFR 50.92(c), operation of Dresden Units 2 and 3 in accordance with the proposed amendments will not:

(1) involve a significant increase in the probability or consequences of an accident previously evaluated because:

(a) The miscellaneous editorial, grammatical, reference changes are administrative in nature and do not allow any new operating practices or changes in equipment which could impact the probability or consequences of an accident.

(b) The provision to allow control rod drive testing with Low Pressure Cooling Systems inoperable includes restrictions that the reactor be in the REFUEL mode (following achievement of cold shutdown) and specifically prohibit any simultaneous work which has the potential to drain the reactor vessel. The latter provision ensures that the probability of a loss of coolant accident is not increased by this amendment. In addition, REFUEL mode interlocks prevent the withdrawal of more than one control rod thereby protecting against the possibility of making the reactor critical.

(c) The changes regarding the CRD return line valves reflect actions taken by Commonwealth Edison in response to NRC recommendations in NUREG-0619. As a result [of] thermal stress cracking in these lines, these lines had previously been isolated and on Unit 3, the line was recently removed. The proposed changes modify the Technical Specifications to reflect the current plant configuration. The CRD return lines have either been permanently isolated (Unit 3) or have the isolation valves closed (Unit 2) to ensure primary containment integrity.

(2) Create the possibility of a new or different kind of accident from any accident previously evaluated because

(a) The administrative changes do not allow any new equipment or operating procedures which could initiate or impact the scenario of an accident or operational event.

(b) Post maintenance testing of control rod drives is not a new activity and therefore does not introduce any new concerns regarding the initiation or progression of a transient event. This provision does not involve any new equipment, changes to

equipment, or significant changes to operating procedures and therefore cannot initiate any new events beyond those previously evaluated.

(c) The changes regarding the CRD line valves are conservative in that they reflect the removal or isolation of this line in response to NRC requirements.

(3) Involve a significant reduction in the margin of safety because the changes are either administrative and have no direct effect on operating limits or equipment availability or contain specific provisions to assure the margin of safety is not compromised as in the case of the control rod drive testing provision and the CRD return line valves (where removal/isolation of these lines provides additional protection against the thermal stress cracking concern).

In consideration of the above, Commonwealth Edison has determined that the proposed amendments do not represent a significant hazards consideration and request their approval under the provisions of 10 CFR 50.91(a)(4).

The staff has reviewed the licensee's no significant hazards consideration determination and the content of the licensee's submittals and agrees with the licensee's analysis. Therefore, based on this review, the staff has made a proposed determination that the requested amendments involve no significant hazards consideration.

Local Public Document Room
location: Morris Public Library, 604 Liberty Street, Morris, Illinois 60450.

Attorney for licensee: Mr. Michael I. Miller, Isham, Lincoln and Beale, Three First National Plaza, Suite 5200, Chicago, Illinois 60602.

NRC Project Director: John A. Zwolinski.

Connecticut Yankee Atomic Power Company, Docket No. 50-213, Haddam Neck Plant, Middlesex County, Connecticut

Date of amendment request: September 4, 1986.

Description of amendment request:
The proposed license amendment would amend Section 4.10.C of the technical specifications to incorporate inservice inspection surveillance requirements for the reactor coolant pump (RCP) flywheels consistent with the guidance found in Regulatory Guide 1.14, Revision 1. In particular, this proposed amendment would increase the frequency of pump flywheel inspections and ensure the examination of each RCP flywheel on a regular interval (approximately 3 years). The present inspection frequency requires only one RCP flywheel inservice inspection every second outage.

Basis for proposed no significant hazards consideration determination: The present technical specification requires that only one RCP flywheel be inspected every second refueling outage. At this frequency, it would take six (6) refueling outages (approximately 7 years) to complete the inspection of all 4 RCP flywheels and any individual RCP flywheel would be inspected every eight (8) refueling outages (approximately 9 years).

The proposed license amendment increases the inspection frequency such that each RCP flywheel will be inspected during an interval not to exceed three (3) years. In addition, the proposed change would require each RCP flywheel to receive an ultrasonic volumetric examination of the higher stress areas, i.e., bore and keyway areas, in lieu of the present requirement to do a visual and volumetric examination of only one flywheel every other outage.

The Commission has provided guidance concerning the application of standards in 10 CFR 50.92 by providing certain examples (51 FR 7750, March 6, 1986) of license changes involving no significant hazards consideration. The staff has reviewed the proposed change and concludes that it falls within the envelope of example (ii) in that the change would constitute an additional limitation, restriction or control not included in the current technical specifications. As described above, the proposed non-destructive testing requirements are in accordance with existing regulatory criteria not now required by the plant technical specifications.

Based on the above, the staff proposes to find that the requested license amendment involves no significant hazards considerations.

Local Public Document Room location: Russell Library, 123 Broad Street, Middletown, Connecticut 06457.

Attorney for licensee: Gerald Garfield, Esquire, Day, Berry and Howard, Counselors at Law, City Place, Hartford, Connecticut 06103-3499.

NRC Project Director: Christopher L. Grimes.

Consolidated Edison Company of New York, Docket No. 50-247, Indian Point Nuclear Generating Unit No. 2, Westchester County, New York

Date of amendment request: August 18, 1986.

Description of amendment request: The proposed amendment would revise the Technical Specifications to include provisions for automatic actuation of the reactor trip breakers shunt trip attachment consistent with Item 4.3 of

Generic Letter 83-28 concerning the generic implication of the Salem Anticipated Transient Without Scram (ATWS) event. The proposed changes are responsive to Generic Letter 85-09, entitled "Technical Specifications for Generic Letter 83-28, Item 4.3" and the June 22, 1984 Safety Evaluation for a modification to Indian Point Unit 2 to provide automatic actuation of the reactor trip breakers shunt trip attachment consistent with Item 4.3 of Generic Letter 83-28. The proposed amendment would also correct two typographical errors contained in the current Technical Specifications.

Basis for proposed no significant hazards consideration determination: The Commission has provided guidance concerning the application of the standards for a no significant hazards determination by providing certain examples (51 FR 7155). One of these examples (ii) of actions not likely to involve a significant hazards consideration relates to additional restrictions or controls not presently included in the Technical Specifications. Consistent with this example, the proposed changes with respect to reactor trip breakers provide new explicit LCO's and testing requirements consistent with the modified shunt trip, design, not previously included in the Technical Specifications.

The proposed changes correcting the typographical error are consistent with example (i) of the Commission's guidance. Example (i) relates to a purely administrative change to the technical specifications; for example a change to achieve consistency throughout the technical specifications; correction of an error, or a change in nomenclature. Based on the above the staff proposes to determine that the application does not involve a significant hazards consideration.

Local Public Document Room location: White Plains Public Library, 100 Martine Avenue, White Plains, New York, 10610.

Attorney for licensee: Brent L. Brandenburg, Esq., 4 Irving Place, New York, New York 10003.

NRC Project Director: Steven A. Varga.

Duke Power Company, et al., Docket Nos. 50-413 and 50-414, Catawba Nuclear Station, Units 1 and 2, York County, South Carolina

Date of amendment request: March 15, 1985 as supplemented August 7, 1985, November 8, 1985, March 7, 1986, April 14, 1986 and September 18, 1986.

Description of amendment request: The proposed amendments would revise: (1) Surveillance Requirement

4.6.5.3.Ib. to reduce the surveillance frequency for testing the ice condenser lower inlet doors from at least once per 3 months during the first year after the ice bed is initially fully-loaded and at least once per 6 months thereafter to at least once per 18 months; (2) Surveillance Requirements 4.6.5.3.Ib.3) and 4.6.5.3.Ib.4) to increase the inlet doors test sample to a least 50% of the doors in lieu of 25% and to ensure that all doors are tested at least once during two test intervals in lieu of four test intervals.

The testing and surveillance required to demonstrate operability of the ice condenser lower inlet doors are time consuming and require a unit shutdown. The licensee stated that scheduling a unit shutdown solely to carry out the testing and surveillance required is not considered appropriate because this surveillance has a limited safety significance due to the high reliability of the doors.

Basis for proposed no significant hazards consideration determination: The Commission has provided certain examples (51 FR 7744) of actions likely to involve no significant hazards considerations. The request involved in this case does not match any of those examples. However, the staff has reviewed the licensee's request for the above amendments and determined that should this request be implemented, it would not: (1) Involve a significant increase in the probability or consequences of an accident previously evaluated because the reduction in the surveillance frequency of the ice condenser lower inlet doors, the increase of the test sample and the change in test intervals would not significantly affect the operability of the doors. The surveillance records at Catawba show that the doors are highly reliable because a design change made to the door seals to prevent the doors from freezing was implemented at Catawba Units 1 and 2 prior to the issuance of their fuel loading licenses. Also, it would not (2) create the possibility of a new or different kind of accident from any accident previously evaluated because the changes would not affect the design and would not introduce new modes of operation of the facility. Finally, it would not (3) involve a significant reduction in a margin of safety because the surveillance records show that the ice condenser lower inlet doors are highly reliable as stated in item (1).

Accordingly, the Commission has determined that the above changes involve no significant hazards consideration.

Local Public Document Room

location: York County Library, 138 East Black Street, Rock Hill, South Carolina 29730.

Attorney for licensee: Mr. Albert Carr, Duke Power Company, 422 South Church Street, Charlotte, North Carolina 28242.

NRC Project Director: B.J. Youngblood.

Duke Power Company, et al., Docket Nos. 50-413 and 50-414, Catawba Nuclear Station, Units 1 and 2, York County, South Carolina

Date of amendment request: July 9, 1986, as supplemented September 12, 1986.

Description of amendment request: The amendments would permit an exception to the experience requirements for two additional candidates for senior reactor operator (SRO) licenses. The exception is from the requirements stated in Section A.1.a of Enclosure 1 to the Denton letter, dated March 28, 1980, referenced in Technical Specification Section 6.0, "Administrative Controls." The Commission has previously approved a similar exception for six candidates (51 FR 5282). The licensee's letter of September 12, 1986, provided additional information in response to NRC staff letter issued about August 25, 1986, undated.

Basis for proposed no significant hazards consideration determination: The Technical Specifications Section 6.3 "Unit Staff Qualifications" and Section 6.4 "Training" require, among other things, that the licensee's unit operating staff meet or exceed the requirements in Sections A and C of Enclosure 1 to the Denton letter dated March 28, 1980. Section A of Enclosure 1 requires that an applicant for SRO license shall have a minimum of 4 years of experience as a control room operator (fossil or nuclear). This experience requirement is a prerequisite for taking the SRO examination. However, the principal requirement is that the SRO candidates pass the NRC license examination.

Section A of the Denton letter allows exceptions to the experience requirements for SRO applicants for plants that are not yet licensed because there is no opportunity to obtain such experience on their plants. The proposed change to Technical Specification 6.3.1 is requested for a similar reason in that Catawba Unit 1, which received a fuel loading and precriticality testing license in July 1984, a low power license in December 1984, and a full power license in January 1985, has not been in operation long enough to provide an opportunity for reactor

operators to have 4 years of control room operating experience. Likewise, Catawba Unit 2 received a low power license in February 1986 and a full power license in May 1986.

The Commission has made a proposed determination that the amendment request involves no significant hazards consideration by application of the standards in 10 CFR 50.92. The Commission's staff has determined that should this request be implemented, it would not: (1) involve a significant increase in the probability or consequences of an accident previously evaluated because the two SRO candidates are highly trained at Catawba, each has held a reactor operator license for approximately 2 years and each would be required to pass the SRO license examination; or (2) create the possibility of a new or different kind of accident from any accident previously evaluated because the SRO candidates are experienced, licensed operators and the amendment does not change the manner in which the plant is to be operated; or (3) involve a significant reduction in a margin of safety because, in addition to the requirement that each candidate pass the NRC examination for an SRO license, each has greater than 8 years of experience on-site at Catawba, during which each has been actively involved in preoperational testing and checkout, startup testing, and operator training. Accordingly, the Commission proposes to determine that this change does not involve a significant hazards consideration.

Local Public Document Room location: York County Library, 138 East Black Street, Rock Hill, South Carolina 29730.

Attorney for licensee: Mr. Albert Carr, Esq., Duke Power Company, P.O. Box 33189, Charlotte, North Carolina 28242.

NRC Project Director: B.J. Youngblood.

Duke Power Company, Docket Nos. 50-369 and 50-370, McGuire Nuclear Station, Units 1 and 2, Mecklenburg County, North Carolina

Date of amendment request: March 19, 1986.

Description of amendment request: Existing Technical Specification (TS) 3.11.1.1 and its referenced Figure 5.1-4, "Site Boundary for Liquid Effluents" define the authorized discharge point for radioactive material released in liquid effluents to unrestricted areas as being only to Lake Norman, an upstream impoundment of the Catawba River. The proposed amendments would modify Figure 5.1-4 to add an additional discharge point from the Conventional

Wastewater Basin (CWWB) into the Catawba River. The change would affect only the discharge location, and would not increase existing TS requirements regarding: (1) The quantity of radioactive material which may be contained in or released from the pond, (2) allowable doses to the public from releases to unrestricted areas, and would not decrease existing TS requirements regarding liquid discharge monitoring.

The change would be accomplished by deleting from TS Figure 5.1-4 an existing, obsolete footnote which authorized a one-time discharge from the CWWB to the Catawba River on June 20, 1986, but retaining the existing arrow at the river and its label, "Liquid Waste Discharge Point." (The existing arrow, label, and footnote were added in response to a separate application by the licensee submitted subsequent to the March 19, 1986 request.)

Basis for proposed no significant hazards consideration determination: Chemical wastes from the McGuire Station (e.g., turbine building drains, water treatment system filter backwashes, demineralizer regeneration wastes), which are normally non-radioactive, are routed through the Conventional Waste Water Treatment System (CWWTS) and subjected to physicochemical treatment. The CWWTS includes a Basin of two parallel stream settling ponds with a capacity of about 2 million gallons each. Upon completion of treatment, the discharges from this system are released to the Catawba River downstream of Cowans Ford Dam. Waste containing radioactive material is normally routed to separate Liquid Radwaste Systems (see FSAR Section 11.2) for recycling, processing and discharge to Lake Norman. During operation with primary-to-secondary leakage in steam generators, the waste in the turbine building sumps will become contaminated; long-term operation with such leakage can create large volumes of liquid waste in the turbine building sumps in excess of the processing capacity of the Liquid Radwaste System. If the level of contamination is within limits, the sump contents are routed to the CWWTS.

The quantity of radioactive material contained in each chemical treatment pond, and in each batch of slurry (used power resins) to be transferred to the chemical treatment ponds, is limited consistent with 10 CFR Part 20, Appendix B, Table II by existing TS 3/4.11.1.5. The concentration of radioactive material released in liquid effluents to unrestricted areas is limited

consistent with 10 CFR Part 20, Appendix B, Table II by existing TS 3/4.11.1.1. The dose or dose commitment to a member of the public from radioactive materials in liquid effluents released from each McGuire unit is limited consistent with 10 CFR 50, Appendix I by existing TS 3/4.11.1.2. These TSs (3/4.11.1.1, 3/4.11.1.2, and 3/4.11.1.5) would apply to both the CWWTS and the Lake Norman discharge points. The change would also not decrease the existing monitoring requirements (TS 3.3.3.8 and referenced TS Table 3.3-12) which assure that instantaneous radioactive release rates remain within 10 CFR Part 20, Appendix B limits, and that radioactive liquid effluent monitoring instrumentation remains operable or appropriate compensatory action taken. Rather, the change provides for consistency of TS Figure 5.1-4 with these other existing TSs which assure that such discharges, concentrations and doses are consistent with the Commission's regulations. Therefore, as noted in the licensee's submittal, the change more accurately reflects station design and practice when operating with a primary-to-secondary leak in steam generators.

The Commission has provided certain examples (51 FR 7744) of actions likely to involve no significant hazards considerations. One of the examples (i) relates to amendments to a purely administrative change to Technical Specifications. Removal of the obsolete footnote has no safety implication and matches this example. The remainder of the change, which designates the river as a liquid waste discharge point, does not match any of those examples. However, the staff has reviewed the licensee's request for the above amendments and has determined that should this portion of the change be implemented, it would not involve: (1) A significant increase in the consequences of an accident previously evaluated or (2) a significant reduction in a margin of safety. The change does not increase the radioactive waste produced by or released from the station. The concentrations of radioactivity in the CWWB are maintained low in accordance with existing TS requirements and the potential accidental radioactive releases from the CWWB are bounded by the releases from the postulated design-basis liquid tank failures evaluated by the Commission in the McGuire Safety Evaluation Report, Section 15.3.10, and found to result in acceptable radionuclide concentrations in the Catawba River. This part of the change also would not (3) increase the

probability of an accident previously evaluated or create the possibility of a new or different kind of accident from any accident previously evaluated. Because the change does not involve any new or novel changes in equipment, design, operating procedures and limits, setpoints, or limiting conditions for operation, it has no effect on accident causal mechanisms.

On the above bases, the Commission proposes to determine that these proposed amendments do not involve a significant hazards consideration.

Local Public Document Room location: Atkins Library, University of North Carolina, Charlotte (UNCC Station), North Carolina 28223.

Attorney for licensee: Mr. Albert Carr, Duke Power Company, P.O. Box 33189, 422 South Church Street, Charlotte, North Carolina 28242.

NRC Project Director: B.J. Youngblood.

Duke Power Company, Dockets Nos. 50-269, 50-270 and 50-287, Oconee Nuclear Station, Units Nos. 1, 2 and 3, Oconee County, South Carolina

Date of amendment request: August 27, 1986, as supplemented with additional information on September 29, 1986.

Description of amendment request: The proposed amendments would revise the Station's common Technical Specifications (TSs) to add operability requirements of monitors and surveillance items required by the addition of the radwaste facility at the Oconee Nuclear Stations (ONS). The proposed amendments would also delete certain outdated footnotes with the gaseous process and effluent monitoring instrumentation.

In a letter dated June 10, 1985, and supplements, the licensee requested approval under 10 CFR Part 20, § 20.305, to treat or dispose of licensed material by incineration. The incinerator is one major integral component of the new volume reduction radwaste facility.

The licensee will monitor the process exhaust from the volume reduction system as it is mixed with normal facility heat, ventilation and air condition (HVAC) exhaust before release. An isokinetic sampling system is provided to obtain representative exhaust duct air samples for radiological monitoring and analyses. A continuous noble gas activity monitor and sample cartridge for continuous collection of iodine and particulate samples are provided.

Basis for proposed no significant hazards consideration determination: The Commission has provided guidance concerning the application of the

standards in 10 CFR 50.92 by providing certain examples (51 FR 7750). Example (i) of the types of amendments not likely to involve significant hazards considerations is an amendment considered to be a purely administrative change to the TSs; for example, a change to achieve consistency throughout the TSs, correction of an error, or a change in nomenclature.

One of the proposed changes to the TSs has been determined to contain only administrative changes. The requested changes are required so that the TSs are updated and no longer note obsolete footnotes. Also, some typing format changes have been proposed.

For the other proposed revision to the TSs, i.e., to add operability requirements of monitors and surveillance items required by the addition of the radwaste facility, the Commission has provided guidance concerning the determination of significant hazards considerations by providing certain standards (10 CFR 50.92(c)). A proposed amendment to an operating license for a facility involves no significant hazards considerations if operation of the facility in accordance with the proposed amendment would not:

- (1) Involve a significant increase in the probability or consequences of an accident previously evaluated; or
- (2) Create the possibility of a new or different kind of accident from any accident previously evaluated; or
- (3) Involve a significant reduction in a margin of safety.

These requested amendments will not involve a significant increase in the probability or consequences of an accident previously evaluated. The licensee states that the amendments constitute operability requirements of monitors and surveillance requirements for the incinerator. Appropriate accident analyses for the incinerator were provided in the June 10, 1985 submittal. The activity release by nuclide and the dose estimated for each of the accident cases analyzed are provided in the June 10, 1985 submittal. The doses calculated were derived with conservative assumptions and were found to be below 10 CFR Part 20 annual dose limits. Therefore, the consequences of these accidents analyzed will not be significantly increased. The proposed changes include additional operability requirements of monitors and surveillance requirements associated with the incinerator. As such, this change is not considered to be an initiator of the accidents analyzed. We agree with the licensee's analysis.

The proposed amendments do not create the possibility of a new or

different kind of accident from any accident previously evaluated because the proposed changes do not involve any physical changes to the plant. These amendments result from the addition of the radwaste facility at ONS. No new or different kind of accident can be created since these amendments only add additional sampling points for surveillance and define the operability requirements for the radwaste facility monitors.

The proposed amendments do not involve a significant reduction in a margin of safety. Operation of the radwaste facility including the incinerator will still be within Appendix I to 10 CFR Part 50 numerical guides for the three unit site, and accordingly the margin of safety is unchanged.

Based on the above, the Commission's staff proposes to determine that these proposed amendments do not involve a significant hazards consideration.

Local Public Document Room location: Oconee County Library, 501 West Southbroad Street, Walhalla, South Carolina 29691.

Attorney for licensee: J. Michael McGarry, III, Bishop, Lieberman, Cook, Purcell and Reynolds, 1200 17th Street, NW., Washington, DC 20036.

NRC Project Director: John F. Stolz.

Florida Power Corporation, et al., Docket No. 50-302, Crystal River Unit No. 3 Nuclear Generating Plant, Citrus County, Florida

Date of amendment request: February 17, 1986.

Description of amendment request: The proposed amendment would change the expiration date for Facility Operating License No. DPR-72 from September 25, 2008, to December 3, 2016, 40 years from the issuance of the operating license.

Basis for proposed no significant hazards consideration determination: The currently licensed term for the Crystal River Unit No. 3 Nuclear Generating Plant is 40 years commencing with issuance of the Construction Permit (September 25, 1968). Accounting for the time required for plant construction, this represents an effective operating license term of 31 years and 10 months. The licensee's application requests a 40-year operating license term.

The licensee's request for extension of the operating license is in accordance with 10 CFR 50.51 and is based on the fact that a 40-year service life was considered during the design and construction of the plant. Although this does not mean that some components will not wear out during the plant lifetime, design features were

incorporated to maximize the inspectability of structures, systems, and equipment. Surveillance and maintenance practices which have been implemented in accordance with the ASME code and the facility Technical Specifications provide assurance that any unexpected degradation in plant equipment will be identified and corrected.

The design of the reactor vessel and its internals considered the effects of a 40-year design life (32 Effective Full Power Years), and a comprehensive vessel material surveillance program is maintained in accordance with 10 CFR Part 50, Appendix H. Analyses showing compliance with the NRC pressurized thermal shock screening criteria have demonstrated that the expected neutron fluence will not be a limiting consideration. In addition to these calculations, surveillance capsules placed inside the reactor vessel provide a means of monitoring the cumulative effects of power operation.

Aging analyses have been performed for all safety-related electrical equipment in accordance with 10 CFR 50.49, "Environmental qualification of electrical equipment important to safety for nuclear power plants," identifying qualified lifetimes for this equipment. These lifetimes will be incorporated into plant equipment maintenance and replacement practices to ensure that all safety-related electrical equipment remain qualified and available to perform all safety functions regardless of the overall age of the plant.

The licensee has reviewed the Final Environmental Statement (FES) to determine if its calculations will be materially affected by the proposed extension and has determined that there will be no significant increase in annual risk to the public and that assurances to protect the environment will continue throughout the proposed plant operating life. The ALARA program is expected to offset any tendency for increased occupational exposure due to plant age. In addition, considerable financial benefits to the local population and to the utility's customers would continue to accrue from continued operation of the facility.

The licensee has concluded, and we agree, that the proposed extension will not modify any operating parameters and restrictions except to allow continued operation for a longer period of time. This is consistent with current regulatory practice under the requirements of 10 CFR 50.51. Based on the above, this amendment will not:

(1) Involve a significant increase in the probability or consequence of an accident previously evaluated. No

operational restrictions are modified by changing the duration of the license.

(2) Create the possibility of a new or different kind of accident from any accident previously evaluated. The proposed change introduces no new mode of plant operation nor does it require physical modification to the plant.

(3) Involve a significant reduction in the margin of safety. Any reduction in the margin of safety will be maintained within acceptable bounds by continued implementation of the referenced ongoing programs (Qualification Maintenance Program, Reactor Vessel Materials Surveillance Program, environmental monitoring, etc.). These programs are designed to assure there would be no significant reduction in the associated margin(s) of safety.

Based upon the above, the Commission proposes to determine that the proposed amendment, which provides for a 40-year operating life for the Crystal River Unit No. 3 Nuclear Generating Plant, involves no significant hazards considerations.

Local Public Document Room location: Crystal River Public Library, 668 N.W. First Avenue, Crystal River, Florida 32829.

Attorney for licensee: R.W. Neiser, Senior Vice President and General Counsel, Florida Power Corporation, P.O. Box 14042, St. Petersburg, Florida - 33733.

NRC Project Director: John F. Stolz.

General Public Utilities Nuclear Corporation Docket No. 50-320, Three Mile Nuclear Station Unit No. 2, Londonderry Township Dauphin County, Pennsylvania

Date of amendments request: August 15, 1986.

Description of amendments request: The proposed change would revise Section 6.3.2 of the Appendix A Technical Specifications by changing the title of the Radiological Controls Director at Three Mile Island Nuclear Generating Station, Unit 2. Section 6.3.2 specifies the qualifications for radiological controls personnel. The change is a change in title only, and there is no change in the required qualifications of the individual filling the position. The change is requested by the licensee to achieve consistency with the corporate organizational structure.

Basis for proposed no significant hazards consideration determination: The Commission has provided guidance concerning the application of standards for determining whether a significant hazards consideration exists by providing certain examples (51 FR 7751)

of amendments that are considered not likely to involve significant hazards considerations. Example (i) relates to purely administrative changes to the technical specifications and specifically identifies changes in nomenclature. Since the change requested by the licensee's August 15, 1986 submittal fits the example provided and satisfies the criteria of 50.92, it is concluded that: (1) The proposed changes do not constitute a significant hazards consideration as defined by 10 CFR 50.92; (2) there is a reasonable assurance that the health and safety of the public will not be endangered by the proposed changes; and (3) this action will not result in a condition which significantly alters the impact of the station on the environment as described in the March 1981 Final Programmatic Environmental Impact Statement.

Local Public Document Room

Location: State Library, Commonwealth and Walnut Streets, Harrisburg, Pennsylvania 17105.

Attorney for licensee: George F. Trowbridge, Esq., Shaw, Pittman, Potts and Trowbridge, 2300 N Street, NW., Washington, DC 20037.

NRC Project Director: William D. Travers.

Georgia Power Company, Oglethorpe Power Corporation, Municipal Electric Authority of Georgia, City of Dalton, Georgia, Docket No. 50-368, Edwin I. Hatch Nuclear Plant, Unit No. 2, Appling County Georgia

Date of amendment request: July 18, 1986.

Description of amendment request: The amendment would modify the Technical Specifications (TS) to: (1) Delete four primary containment isolation valves (PCIVs) from and add 15 PCIVs to Table 3.6.3-1; (2) add or correct part numbers (valve identification numbers) for 16 valves listed in Table 3.6.3-1; (3) move eight valves from Section B (Manual Isolation Valves) of Table 3.6.3-1 to Section A (Automatic Isolation Valves); (4) move the RPV head spray valve from Section A (Automatic Isolation Valves) of Table 3.6.3-1 to Section C (Other Isolation Valves), and (5) change the valves listed in Table 3.6.3-1 as the inboard and outboard isolation barriers for the fission product monitoring system sample line.

The changes are proposed to: (1) Reflect past design changes in the system and design changes that are proposed to be made during the refueling outage scheduled to begin in September 1986; (2) correct the valve listing to include all PCIVs and to correct previous errors in identification

number; and (3) correctly identify the valves as "automatic", "manual" and "other".

Basis for proposed no significant hazards consideration determination: The Commission has provided standards for determining whether a significant hazards consideration exists (10 CFR 50.92(c)). A proposed amendment to an operating license for a facility involves no significant hazards consideration if operation of the facility in accordance with the proposed amendment would not: (1) involve a significant increase in the probability or consequences of an accident previously evaluated; or (2) create the possibility of a new or different kind of accident from any accident previously evaluated; or (3) involve a significant reduction in a margin of safety. The changes expand and correct the listing of valves in Table 3.6.3-1. They will better assure that all the valves that are required to be tested for operability and leak tightness are identified and tested. This should insure the margin of safety provided by the isolation system. These changes are not expected to: (1) increase the probability or consequences of an accident previously evaluated; or (2) create the possibility of a new or different kind of accident previously evaluated; or (3) involve a significant reduction in a margin of safety.

On the basis of the above, the Commission has determined that the requested amendment meets the three criteria and therefore has made a proposed determination that the amendment application does not involve a significant hazards consideration.

Local Public Document Room

location: Appling County Public Library, 301 City Hall Drive, Baxley, Georgia.

Attorney for licensee: Bruce W. Churchill, Esquire, Shaw, Pittman, Potts and Trowbridge, 1800 M Street, NW., Washington, DC 20036.

NRC Project Director: Daniel R. Muller.

GPU Nuclear Corporation, et al., Docket No. 50-288, Three Mile Island Nuclear Station, Unit No. 1, Dauphin County, Pennsylvania

Date of amendment request: July 16, 1986.

Description of amendment request: The proposed amendment changes the order of preference of instrumentation used to monitor reactor power quadrant tilt. The current Technical Specifications require measuring quadrant tilt using the full incore detector system (FIT). If FIT is not available, then the minimum incore detector system (MIT) is used. If neither FIT or MIT is available, then the out of core detector system (OCT) is to

be used. Since the OCT is more accurate than the MIT, the proposed amendment reverses the order of preference of the MIT and the OCT. Thus, under the proposed amendment, if the FIT is not available, the OCT would be used next; and if FIT and OCT were both not available, then MIT would be used.

The proposed amendment also includes changes to allow the withdrawal of axial power shaping rods under end of cycle core conditions. These changes were noticed separately on July 30, 1986 (51 FR 27284) and were approved by Amendment No. 120 issued September 2, 1986.

Basis for proposed no significant hazards consideration determination: The Commission has provided standards for determining whether a significant hazards consideration exists as stated in 10 CFR 50.92. A proposed amendment to an operating license for a facility involves no significant hazards considerations if it meets three standards as described in 10 CFR 50.92. Each standard is discussed in turn.

Standard 1—The proposed amendment should not involve a significant increase in the probability or consequences of an accident previously evaluated. The proposed amendment simply revises the order of preference for selecting the system that shall be used to determine quadrant tilt. It does not change any set point or required system accuracy or surveillance interval. Thus, it does not increase the probability or consequences of any accident previously evaluated.

Standard 2—The proposed amendment should not create the possibility of a new or different kind of accident from any accident previously evaluated. As discussed in Standard 1, the proposed amendment only revises the order of preference for selecting the equipment used to measure quadrant tilt. It changes no limits. Thus, it does not create the possibility of a new or different kind of accident.

Standard 3—The proposed amendment should not involve a significant reduction in a margin of safety. The proposed amendment changes no limits and thus has no effect on existing margins of safety.

Accordingly, based on the above discussions, the Commission proposes to determine that the proposed amendment does not involve significant hazards considerations.

Local Public Document Room location: Government Publications Section, State Library of Pennsylvania, Education Building, Commonwealth and Walnut Streets, Harrisburg, Pennsylvania 17126.

Attorney for licensee: G.F. Trowbridge, Shaw, Pittman, Potts and Trowbridge, 2300 N Street, NW., Washington, DC 20037.

NRC Project Director: John F. Stolz.

Kansas Gas and Electric Company, Kansas City Power and Light Company, Kansas Electric Power Cooperative, Inc., Docket No. 50-482, Wolf Creek Generating Station, Coffey County, Kansas

Date of Amendment request: September 10, 1986.

Description of amendment request: The proposed amendment proposes to change the title "Manager Nuclear Safety" to "Manager Analyses Service" and changes the reporting responsibility of the Independent Safety Engineering Group from the Manager Nuclear Safety to the Chairman of the Nuclear Safety Review Committee.

Basis for proposed no significant hazards consideration determination: The proposed revisions do not involve a significant increase in the probability or consequences of an accident previously evaluated. These changes involve organizational modifications and enhancements and as such, have no effect on plant equipment or the technical qualifications of plant personnel.

The proposed revisions do not create the possibility of a new or different kind of accident from any accident previously evaluated. These changes do not affect the overall number or qualifications of personnel who operate Wolf Creek Generating Station, nor do they involve any change to installed plant systems or the overall operating philosophy of Wolf Creek Generating Station.

The proposed revisions do not involve a significant reduction in a margin of safety. These changes do not involve any changes in overall organizational commitments or individual job responsibilities. Organizational modifications alone do not reduce any margin of safety.

Based on the above analysis the licensee has concluded that the proposed revisions to the Wolf Creek Generating Station Technical Specifications involve no significant hazards considerations. The NRC staff has reviewed the licensee's significant hazards consideration determination and agrees with the licensee's analysis. The staff has, therefore, made a proposed determination that the licensee's request does not involve a significant hazards consideration.

Local Public Document Room location: Emporia State University, William Allen White Library, 1200 Commercial Street, Emporia, Kansas,

66801 and Washburn University School of Law Library, Topeka, Kansas.

Attorney for licensee: Jay Silberg, Esq., Shaw, Pittman, Potts and Trowbridge, 1800 M Street, NW., Washington, DC 20036.

NRC Project Director: B.J. Youngblood.

Louisiana Power and Light Company, Docket No. 50-382, Waterford Steam Electric Station, Unit 3, St. Charles Parish, Louisiana.

Date of amendment request: June 24, 1986.

Description of amendment request: The licensee plans to implement a station modification at the Waterford 3 Steam Electric Station during the first refueling outage to provide the plant operators with the capability of bypassing the high steam generator level reactor trip. The proposed change to Technical Specification 3.3.1 will allow the operations staff to bypass the trip while in Modes 1 and 2. As currently noted in Section 2.2.1 of the Technical Specification Bases, the Steam Generator Level—High trip is provided to protect the turbine from excessive moisture carry-over. Because the turbine is automatically tripped when the reactor is tripped, the Steam Generator Level-High trip provides a reliable means for providing protection to the turbine from excessive moisture carry-over. The trip's set point does not correspond to a Technical Specification Safety Limit and no credit is taken in the safety analyses for operation of this trip. Its functional capability at the specified trip setting enhances the overall reliability of the Reactor Protection System.

Additionally, the high steam generator level trip is described in Section 7.2.1.1.10 of the Waterford 3 FSAR. It is an equipment protective trip only and, therefore, does not fall within the scope of IEEE 279-1971, "Criteria for Protection Systems for Nuclear Generating Stations". However, in order to enhance the overall reliability of the Reactor Protection System (RPS) and, as stated in the FSAR, "to preserve uniformity of function and design, the high steam generator level trip function meets the design bases" for other RPS components, including IEEE 279-71.

The proposed change will not affect the design or testing of the non-safety related high steam generator level trip function but will only provide the option to bypass the function in Modes 1 and 2.

Basis for Proposed No Significant Hazards Considerations Determination: The NRC staff proposes that this specific change does not involve a significant hazards consideration

because, as required by the criteria of 10 CFR 50.92(c), operation of the facility in accordance with the proposed amendment would not: (1) involve a significant increase in the probability or consequences of an accident previously evaluated; or (2) create the possibility of a new or different kind of accident from any accident previously evaluated; or (3) involve a significant reduction in the margin of safety. The basis for this proposed finding is given below.

(a) The proposed change allows bypassing the non-safety related steam generator high level trip. This trip is not credited in the Waterford 3 safety analyses nor does the trip setpoint correspond to a Technical Specification Safety Limit. The design, testing and reliability of the RPS is unaffected by the proposed change. Therefore the proposed change will not involve a significant increase in the probability or consequences of any accident previously evaluated.

(b) The most adverse consequence of bypassing the high steam generator level trip is the potential for moisture carry over to the turbine and subsequent damage. This, however, is not a safety concern. The main steam line piping to the main steam isolation valves is designed to carry a water loading. Even should the main steam line piping be postulated to rupture due to the water loading, the resulting event is bounded by the main steam line break event analyzed in the FSAR. No new systems, modes of operation, failure modes or other plant perturbations are introduced; therefore, the proposed change will not create the possibility of a new or different kind of accident from any previously evaluated.

(c) As previously stated, the high steam generator level trip is not credited in any safety evaluation. By definition, bypassing the trip cannot provide any reduction in the margin of safety that presently exists in the accident analysis and in the plant design.

As the change requested by the licensee's June 24, 1986 submittal satisfies the criteria of 50.92, it is concluded that: (1) The proposed change does not constitute a significant hazards consideration as defined by 10 CFR 50.92; (2) there is a reasonable assurance that the health and safety of the public will not be endangered by the proposed change; and (3) this action will not result in a condition which significantly alters the impact of the station on the environment as described in the NRC Final Environmental Statement.

Local Public Document Room location: University of New Orleans

Library, Louisiana Collection, Lakefront, New Orleans, Louisiana 70122.

Attorney for licensee: Mr. Bruce W. Churchill, Esq., Shaw, Pittman, Potts and Trowbridge, 1800 M St., NW., Washington, DC 20036.

NRC Project Director: George W. Knighton.

Louisiana Power and Light Company, Docket No. 50-382, Waterford Steam Electric Station, Unit 3, St. Charles Parish, Louisiana

Date of amendment request: July 15, 1986.

Description of amendment request: The proposed change would revise Technical Specification 3.10.2, "Moderator Temperature Coefficient, Group Height, Insertion, and Power Distribution Limits", along with the associated surveillance requirements in 4.10.2. The proposed change will allow suspension of certain limits specified in the specification to accommodate physics tests following startup after refueling. The associated Bases is also revised to reflect technical terminology utilized at Waterford 3.

In order to perform certain startup tests for Cycle 2 such as the verification of radial peaking factors at high power levels, it is necessary to insert the Part Length Control Element Assemblies (PLCEAs) and CEAs beyond the limits specified in Technical Specifications 3.1.3.6 and 3.1.3.7. Technical Specification 3.10.2 currently allows suspension of the insertion limits for full length CZAs specified in Technical Specification 3.1.3.6. Technical Specification 3.1.3.7 imposes similar limits on the insertion of PLCEAs; it is, therefore, necessary to also suspend these limits to perform physics tests.

Basis for Proposed No Significant Hazards Consideration Determination: The NRC staff proposes that the proposed changes do not involve a significant hazards consideration because, as required by the criteria of 10 CFR 50.92(c), operation of the facility in accordance with the proposed amendment would not: (1) involve a significant increase in the probability or consequences of an accident previously evaluated; or (2) create the possibility of a new or different kind of accident from any accident previously evaluated; or (3) involve a significant reduction in the margin of safety. The basis for this finding is given below.

(a) Suspending the limits on PLCEA insertion allows for measurement of data necessary to verify proper operation of the Core Protection Calculators (CPCs) following a refueling of the reactor core. Because the tests which rely on the SPECIAL TEST

EXCEPTIONS in Specification 3.10.2 are relatively short in duration, core parameters related to the safety analyses are not adversely affected. Therefore, this change does not significantly increase the probability or consequences of any accident previously evaluated.

(b) Insertion of the PLCEAs beyond the limits specified in the proposed change to Technical Specification 3.1.3.7 is required to verify certain assumptions necessary to complete the Cycle 2 safety analyses. These tests are required to verify the safety analyses assumptions and are relatively short in duration. Core parameters related to the safety analysis are not adversely affected. No new systems, failure modes or plant perturbations from any previously analyzed are introduced. Therefore, this change does not create the possibility of a new or different kind of accident from any accident previously evaluated.

(c) The limits imposed on PLCEA insertion, which are more restrictive than those currently allowed, are used as inputs to the Cycle 2 safety analyses. All safety analyses assumptions are still valid when this special test exception is invoked because the surveillance requirements associated with this specification confirm that the core parameters related to safety are not adversely affected. Therefore, this change does not involve a significant reduction in the margin of safety.

As the change requested by the licensee's June 24, 1986 submittal satisfies the criteria of § 50.92, it is concluded that: (1) The proposed changes do not constitute a significant hazards consideration as defined by 10 CFR 50.92; (2) there is reasonable assurance that the health and safety of the public will not be endangered by the proposed change; and (3) this action will not result in a condition which significantly alters the impact of the station on the environment as described in the NRC Final Environmental Statement.

Local Public Document Room location: University of New Orleans Library, Louisiana Collection, Lakefront, New Orleans, Louisiana 70122.

Attorney for licensee: Mr. Bruce W. Churchill, Esq., Shaw, Pittman, Potts and Trowbridge, 1800 M St., NW., Washington, DC 20036.

NRC Project Director: George W. Knighton.

Louisiana Power and Light Company, Docket No. 50-382, Waterford Steam Electric Station, Unit 3, St. Charles Parish, Louisiana

Date of amendment request: July 15, 1986.

Description of amendment request: Technical Specification 3.1.3.7 imposes limits on the allowable position of the Part Length Control Element Assembly (PLCEA) groups and on the allowable burnup span during which the PLCEA may remain within a given position range during Modes 1 and 2. Technical Specification 3.1.3.7 currently states that the PLCEA groups shall be restricted in position between 0°—17° withdrawn (i.e. between fully inserted and 11% withdrawn) for a maximum period of seven Effective Full Power Days (EFPD) out of any 30 EFPD period. The proposed change would replace the entire current technical specification and would add a Figure 3.1-3 which: (1) allows a maximum PLCEA insertion to 75% withdrawal (112.5 inches) during long term steady state operation above 20% thermal power, (2) allows any PLCEA insertion below 20% thermal power (i.e. PLCEA insertion below 20% power has negligible effect on unexpected reactivity additions, axial flux perturbations, and axial peaking), and (3) allows a maximum transient PLCEA insertion to 15% withdrawal (22.5 inches) between 50% and 20% thermal power for a specified limited burnup duration. The more restrictive PLCEA insertion limits provided by the proposed changes to the Technical Specification, including Figure 3.1-3, will be used in the Cycle 2 Safety Analysis.

Surveillance Requirement 4.1.3.7 currently requires determination of the PLCEA group positions at least once per 12 hours. The proposed change would replace the entire current surveillance requirement with an equivalent requirement to determine that the PLCEA groups are within the transient insertion range once each 12 hours.

Basis for proposed no significant hazards consideration determination: The NRC staff proposes that this change does not involve a significant hazards consideration because, as required by the criteria of 10 CFR 50.92(c), operation of the facility in accordance with the proposed amendment would not: (1) involve a significant increase in the probability or consequences of an accident previously evaluated; or (2) create the possibility of a new or different kind of accident from any accident previously evaluated; or (3) involve a significant reduction in a margin of safety. The basis for this proposed finding is given below.

(a) There are two reasons for changing the technical specification. First, the proposed change, by imposing more restrictive insertion limits, will provide an improvement in the potential consequences of a PLCEA drop or slip

which initiates from an allowable inserted position. Second, the proposed change adds a more explicit Limiting Condition for Operation to clarify the allowable duration for the PLCEA to remain within defined ranges of axial position. Therefore, this proposed change will provide additional assurance that adverse axial shapes and rapid local power changes which affect radial power peaking factors and DNB considerations do not occur as a result of the part length CEA group being positioned in the same axial segment of fuel assemblies for an extended period of time during operation. Because the proposed change will impose more restrictive limits along with surveillance requirements to ensure adherence with the insertion limits, this proposed change does not involve a significant increase in the probability or consequences of any accident previously evaluated.

(b) For the same reasons given in (a), this proposed change does not create the possibility of a new or different kind of accident from any accident previously analyzed.

(c) For the same reasons given in (a), the proposed change does not involve a significant reduction in the margin of safety.

Moreover, adherence to this proposed technical specification will: (1) Eliminate the potential for unexpected reactivity addition which otherwise might occur should a PLCEA drop or move from a less to a more reactive axial position, (2) prevent undesirable perturbations on the axial distribution of core burnup due to PLCEA insertion, and (3) prevent unacceptably high axial peaking upon subsequent movement of the PLCEA groups.

As the change requested by the licensee's July 15, 1986 submittal satisfies the criteria of 50.92, it is concluded that: (1) The proposed changes do not constitute a significant hazards consideration as defined by 10 CFR 50.92; (2) there is a reasonable assurance that the health and safety of the public will not be endangered by the proposed change; and (3) this action will not result in a condition which significantly alters the impact of the station on the environment as described in the NRC Final Environmental Statement.

Local Public Document Room location: University of New Orleans Library, Louisiana Collection, Lakefront, New Orleans, Louisiana 70122.

Attorney for Licensee: Mr. Bruce W. Churchill, Esq., Shaw, Pittman, Potts and Trowbridge, 1800 M St., NW., Washington, DC 20036.

NRC Project Director: George W. Knighton.

Louisiana Power and Light Company, Docket No. 50-382, Waterford Steam Electric Station, Unit 3, St. Charles Parish, Louisiana

Date of amendment request: August 29, 1986.

Description of amendment request: The proposed change would revise ACTION statements "c" and "d" to Technical Specification 3.1.3.1, "Movable Control Assemblies, CEA Position". The reason for this change is to impose new requirements on power reduction during the period from 15 minutes to one hour following a full or part length CEA misalignment. This change would reduce the inward CEA deviation penalty factors currently provided by the CEA Calculators (CEACs) to the CPCs to a value of 1.0. The reduction of these penalty factors will reduce the sensitivity of the CPCs to CEA drops and to electronic noise which can be interpreted in the logic as a major CEA deviation and will therefore eliminate some unnecessary reactor trips.

The margins on DNBR and Linear Heat Rate (LHR) which now exist will be maintained after the reduction in the penalty factors. Currently, if an inward CEA deviation event occurs, the CPC algorithm applies two penalty factors to the DNBR and LHR calculations. The first, a static penalty factor is applied upon detection of the CEA deviation event. The second, a xenon redistribution penalty, is applied linearly as a function of time over a one-hour period following the detection of the deviation.

In the proposed change, the margin reserved by the DNBR Limiting Condition for Operation (LCO) is based on the maximum inward CEA deviation (i.e., the CEA Drop) and therefore accommodates changes in the static power distribution. This margin also accommodates the first 15 minutes of xenon redistribution effects for the limiting CEA drop. Thereafter, for up to one hour after the deviation event, the proposed change to this specification imposes a core power reduction to accommodate xenon redistribution effects occurring beyond the first 15 minutes. Therefore, the combination of the margin reserved by the DNBR LCO and the required core power reduction starting 15 minutes after the deviation is sufficient to maintain the required margins to DNB and LHR for the first hour after detection of the event.

Thereafter, the current action statements in the Technical Specification apply.

Basis for proposed no significant hazards considerations determination:

The NRC staff proposes that the proposed change does not involve a significant hazards consideration because, as required by the criteria of 10 CFR 50.92(c), operation of the facility in accordance with the proposed amendment would not: (1) Involve a significant increase in the probability or consequences of any accident previously evaluated; or (2) create the possibility of a new or different kind of accident from any accident previously evaluated; or (3) involve a significant reduction in the margin of safety. The basis for this proposed finding is given below.

(1) Reducing the static penalty factor generated by the CEACs to a value of 1.0 is accounted for by setting aside the margin in the DNBR LCO. This ensures that the Specified Acceptable Fuel Design Limits (SAFDLs) on both DNBR and LHR can be maintained for up to 15 minutes following the limiting CEA drop event without any reduction in core power. Similarly, reducing the xenon redistribution penalty factor to a value of 1.0 is accounted for by imposing new requirements for core power reduction starting 15 minutes after the postulated CEA drop and continuing for an additional 45 minutes. Thereafter, all other ACTION statements in the Technical Specifications are applicable. Adhering to the proposed power reduction requirements ensures that the power peaking resulting from xenon redistribution will not result in a violation of the SAFDLs. Therefore, since the consequences of the limiting CEA drop event are still acceptable, the proposed change will not significantly increase the probability or consequences of any accident previously evaluated.

(2) The proposed change does not affect the logic used by the CPCs to mitigate the consequences of any Anticipated Operational Occurrence (AOO). Since the proposed change will not affect the ability of the CPCs to perform their design function of protecting the core against a violation of the SAFDLs (during an AOO), it will not create the possibility of a new or different kind of accident from any accident previously evaluated.

(3) In the proposed change, credit is taken for available margin in the DNBR LCO. By staying within this LCO, there is margin to accommodate the first 15 minutes of the most limiting CEA drop. Thereafter, the proposed change requires a core power reduction to accommodate the increased power peaking associated with xenon

redistribution in the core. Therefore, the combination of additional margin reserved in the DNBR LCO and the required power reduction ensures that the proposed change will not involve a significant reduction in the margin of safety.

As the change requested by the licensee's August 29, 1986 submittal satisfies the criteria of 10 CFR 50.92(c), it is concluded that: (1) The proposed change does not constitute a significant hazards consideration as defined by 10 CFR 50.92; (2) there is a reasonable assurance that the health and safety of the public will not be endangered by the proposed change; and (3) this action will not result in a condition which significantly alters the impact of the station on the environment as described in the NRC Final Environmental Statement.

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Attorney for licensee: Mr. Bruce W. Churchill, Esq., Shaw, Pittman, Potts and Trowbridge, 1800 M St., NW, Washington, DC 20036.

NRC Project Director: George W. Knighton.

Louisiana Power and Light Company, Docket No. 50-382, Waterford Steam Electric Station, Unit 3, St. Charles Parish, Louisiana

Date of amendment request: September 10, 1986.

Description of amendment request: Item 6.2.2.d of the Administrative Controls section of the Waterford 3 Technical Specifications defines responsibilities to be observed during any core alterations. The intent of this specification is to require one licensed Senior Reactor Operator (SRO) to supervise/observe the core alterations. When core alterations are being performed by a licensed operator, the present Technical Specification allows the supervising SRO to be remote from the core alteration activities (with the understanding that direct communications are maintained). When non-licensed personnel are performing the core alterations, the intent of the present Technical Specification is to require the supervising SRO to be present during the alterations to also perform a direct observation function. However, the wording in the present Technical Specification could be misconstrued to require two SRO's (one to observe and one to supervise) for the case of non-licensed personnel performing the core alterations. The proposed change will resolve any

ambiguity in SRO requirements for core alterations.

Basis for proposed no significant hazards consideration determination: The NRC staff proposes to determine that the proposed changes do not involve a significant hazards consideration. As required by the criteria of 10 CFR 50.92(c), operation of the facility in accordance with the proposed amendment would not: (1) Involve a significant increase in the probability or consequences of an accident previously evaluated; or (2) create the possibility of a new or different kind of accident from any accident previously evaluated; or (3) involve a significant reduction in the margin of safety. The basis for this proposed finding is as follows.

(1) The existing Technical Specification is ambiguous. This change is intended solely for clarification and as such, makes no changes in the operation of the facility. Therefore, this proposed change will not involve a significant increase in the probability or consequences of any accident previously evaluated.

(2) As stated above, no change in operation will result from this change. In addition, no new systems, modes of operation, failure modes or plant perturbations are created with this change. Therefore, the proposed change will not create the possibility of a new or different kind of accident from any accident previously evaluated.

(3) In providing clarification of responsibilities during core alterations, the actual activity of performing the alterations remains unchanged. Therefore, the proposed change will not involve a reduction in a margin of safety.

The Commission has provided guidance concerning the application of standards for determining whether a significant hazards consideration exists by providing certain examples (51 FR 7751) of amendments that are considered not likely to involve significant hazards considerations. Example (1) relates to a purely administrative change to the Technical Specification: for example, a change to achieve consistency throughout that Technical Specifications, correction of an error, or a change in nomenclature.

In this case, the proposed change is similar to Example (1) in that the change is intended only for purposes of clarifying potentially ambiguous wording and will impose no change on current or future operations of the Waterford facility.

As the change requested by the licensee's June 24, 1986 submittal satisfies the criteria of 50.92, it is

concluded that: (1) the proposed changes does not constitute a significant hazards consideration as defined by 10 CFR 50.92; (2) there is reasonable assurance that the health and safety of the public will not be endangered by the proposed change; and (3) this action will not result in a condition which significantly alters the impact of the station on the environment as described in the NRC Final Environmental Statement.

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Attorney for licensee: Mr. Bruce W. Churchill, Esq., Shaw, Pittman, Potts and Trowbridge, 1800 M St., NW., Washington, DC 20036.

NRC Project Director: George W. Knighton.

Louisiana Power and Light Company, Docket No. 50-382, Waterford Steam Electric Station, Unit 3, St. Charles Parish, Louisiana

Date of amendment request: September 12, 1986 and September 29, 1986.

Description of amendment request: The proposed change would revise ACTION Statement "c" of Technical Specification 3.3.3.8, "Fire Detection Instrumentation" and the associated Table 3.3-11. The proposed change to Table 3.3-11 will divide the present annulus fire zone RCB 1-1 into two distinct fire zones RCB 1-1 and RCB 1-2, with each zone containing approximately half of the smoke detectors presently assigned to RCB 1-1. This change is reflected in a new ACTION STATEMENT "c" in Technical Specification 3.3.3.8 which will require that with less than one annulus zone of smoke detectors operable, one zone must be restored to operable status within one hour or an eight-hour fire watch must be implemented except during the period when the shield building ventilation system surveillance testing is in progress.

Basis for proposed no significant hazards considerations determination: The annulus is an open area, approximately four feet wide, located between the primary containment steel wall and the secondary containment concrete wall of the Reactor Containment Building (RCB). Its function is to prevent the escape of contaminants by providing a space which can be maintained at a negative pressure around the primary reactor area.

Equipment within the annulus consists of the smoke detection system, communication and lighting fixtures, and the piping and ventilation ducts of

the annulus negative pressure system. All of the electrical cables for these systems are routed in conduit. Other components within the annulus are those which pass through from an adjoining building into the primary containment area. Such components include the fuel transfer canal from the Fuel Handling Building, personnel accessways from the Auxiliary Building and piping from the main steam, feedwater and purge systems. Electrical components, including power, instrumentation and control cables, are enclosed within metal sleeves.

The only insitu combustibles within the annulus are the smoke detectors themselves which represent a negligible combustible loading. All cable in the annulus is routed in conduit or metal sleeves. All cabling meets noncombustible test requirements of IEEE Standard 383, "IEEE Standard of Type Test of Class 1E Electrical Cables, Field Splices and Connections for Nuclear Power Generating Stations." All insulation and jacketing material for piping penetration assemblies passing through the annulus is non-combustible. Transient combustibles during repair or maintenance operations are strictly controlled by administrative procedure.

The annulus is void of heat and/or electrical spark generating equipment, i.e., potential ignition sources are absent. Maintenance and/or repairs which involve hot work are strictly controlled in accordance with administrative procedures.

An ionization smoke detection system is provided within the annulus. The system consists of 23 detectors encircling the -4' elevation, 23 detectors encircling the +21' elevation and 23 detectors encircling the +46' elevation. Detection alarm and trouble annunciation are provided in the Control Room on a local panel and the Master Remote Control Panel. The proposed change will divide the detection zone listed in Table 3.3-11 into two fire zones. One zone will consist of the upper (+46') string of detectors and half of the middle (+21') string. The second zone will comprise the lower (-4') string of detectors and half of the middle (+21') string.

The annulus smoke detectors, particularly the upper string, have had a tendency to spuriously alarm. This problem may be caused by dust accumulating on the detectors. Due to the high radiation environment at the upper string during power operation, the actual cause and resolution of the spurious alarms cannot be determined except during an outage.

The proposed change is requested for two reasons. First, during plant

operation the environment within the annulus is not recommended for personnel entry. Temperatures range from 100 °F to 120 °F, radiation dose levels can be in excess of 100 mrem/hr., airborne contamination exists and oxygen levels are reduced to a level requiring SCBAs to be worn. ALARA (As Low As Reasonably Achievable) and other personnel safety concerns outweigh the need for an eight-hour fire watch in an area with no combustible loading or ignition sources when half the smoke detectors are operable.

Second, the requirement for an eight-hour watch conflicts with other Technical Specification surveillance requirements that could ultimately force the plant to shut down due to a spurious fire alarm. Specifically, Surveillance Requirement 4.6.6.1 requires that the shield building ventilation system be demonstrated operable. This surveillance, to be performed once every 31 days, requires the containment building to remain closed for ten continuous hours. Should the annulus fire detectors spuriously alarm near the end of such a 31-day period (and cannot be repaired due to the high radiation environment), the eight-hour fire watch will not allow a continuous 10-hour period for the shield building ventilation system test, thus mandating a plant shutdown, as required by Technical Specification 3.6.6.1.

The NRC staff has determined that the proposed change does not involve a significant hazards consideration because, as required by the criteria of 10 CFR 50.92(c), operation of the facility in accordance with the proposed amendment would not: (1) Involve a significant increase in the probability or consequences of any accident previously evaluated; or (2) create the possibility of a new or different kind of accident from any accident previously evaluated; or (3) involve a significant reduction in the margin of safety. The basis for this proposed finding is given below.

(1) The proposed change will still require that approximately half of the annulus smoke detectors remain operable in the absence of an eight-hour fire watch. The annulus region contains no combustibles (with the exception of the smoke detectors, themselves), nor does it contain potential ignition sources. The proposed change will not significantly increase the probability or consequences of an annulus fire. Therefore, the proposed change will not significantly increase the probability or consequences of any accident previously evaluated.

(2) The proposed change maintains fire detection capability within the

annulus when a fire watch is not required. No combustible or ignition sources are introduced by the change, nor are new components or modes of operation introduced. Therefore, the proposed change will not create the possibility of a new or different kind of accident from any accident previously evaluated.

(3) The proposed change preserves the capability to detect annulus fires. It does not change the total number of smoke detectors; it divides the fire zone into two fire zones. Therefore, coupled with the absence of combustible and ignition sources, the proposed change will not involve a significant reduction in the margin of safety.

As the change requested by the licensee's submittals dated September 12 and September 29, 1986 satisfy the criteria of 10 CFR 50.92(c), it is concluded that: (1) The proposed change does not constitute a significant hazards consideration as defined by 10 CFR 50.92(c); (2) there is a reasonable assurance that the health and safety of the public will not be endangered by the proposed change; and (3) this action will not result in a condition which significantly alters the impact of the station on the environment as described in the NRC Final Environmental Statement.

Local Public Document Room location: University of New Orleans Library, Louisiana Collection, Lakefront, New Orleans, Louisiana 70122.

Attorney for licensee: Mr. Bruce W. Churchill, Esq., Shaw, Pittman, Potts and Trowbridge, 1800 M St., NW., Washington, DC 20036.

NRC Project Director: George W. Knighton.

Mississippi Power & Light Company, Middle South Energy, Inc., South Mississippi Electric Power Association, Docket No. 50-416, Grand Gulf Nuclear Station, Unit 1, Claiborne County, Mississippi

Date of amendment request: June 26, 1986.

Description of amendment request: The amendment would delete Technical Specification 3/4.3.3.7.8 "Chlorine Detection System" and associated Bases.

Basis for proposed no significant hazards consideration determination: The Commission has provided standards for determining whether a significant hazards consideration exists as stated in 10 CFR 50.92. A proposed Amendment to an operating license for a facility involves no significant hazards considerations if operation of the facility in accordance with a proposed

amendment would not: (1) Involve a significant increase in the probability or consequences of an accident previously evaluated; or (2) create the possibility of a new or different kind of accident from any accident previously evaluated; or (3) involve a significant reduction in a margin of safety.

The licensee has provided an analysis of significant hazards considerations in its June 26, 1986 request for a license amendment. The licensee has concluded, with appropriate bases, that the proposed amendment meets the three standards in 10 CFR 50.92 and, therefore, involves no significant hazards considerations.

The Commission has also provided guidance concerning the application of these standards by providing examples of amendments considered likely, and not likely, to involve a significant hazards consideration. These were published in the Federal Register on March 6, 1986 (51 FR 7744). The NRC staff has made a preliminary review of the licensee's submittal. A discussion of these examples as they relate to the proposed amendment follows.

One of the examples of actions involving no significant hazards consideration (vi) is a change which may result in some increase to the probability or consequences of a previously analyzed accident or may reduce in some way a safety margin, but where the results of the change are clearly within all acceptable criteria with respect to the component system specified in the Standard Review plan (NUREG-0800). The proposed change would delete the requirements for chlorine detectors in the outside air intake duct of the control room heating, ventilating and air conditioning system. These chlorine detectors automatically close a damper in the air intake duct if chlorine concentration exceeds the trip setpoint of the detectors. The licensee has estimated the probability of occurrence of an offsite chlorine accident from barge traffic on the Mississippi River to be approximately 10^{-7} per year, which meets the acceptance criterion given in the Standard Review Plan (SRP) Section 2.2.3 "Evaluation of Potential Accidents". The SRP states that such offsite hazards do not need to be considered as design basis events if their expected rate of occurrence is less than 10^{-6} per year. Onsite liquid chlorine is stored in 150-pound cylinders at four different locations. The location closest to the reactor control building is approximately 225 meters away from the building. This complies with Regulatory Guide 1.95, Position 1

(referenced in Standard Review Plan Section 6.4, "Control Room Habitability System") which recommends that liquid chlorine in quantities greater than 20 pounds be stored at least 100 meters away from the reactor control building. The control room is provided with the capability for manual isolation thus complying with the guidance in Regulatory Guide 1.95, Position 2, which recommends such capability for 150-pound cylinders stored on site. Using methodology in NUREG-0570, "Toxic Vapor Concentrations in the Control Room Following a Postulated Accidental Release", and diffusion calculations from Regulatory Guide 1.78 (Referenced in SRP 6.4), the licensee calculated the consequences of a postulated failure of a chlorine cylinder, and found that the chlorine concentration inside the control room would be well below the toxicity guidelines of Regulatory Guide 1.78. Because the results of the deletion of chlorine detectors from the design are clearly within all the applicable acceptance criteria in the Standard Review Plan, the proposed change is found to be similar to example (vi) in the Commission guidance (51 FR 7744).

Accordingly, the Commission proposes to determine that this change does not involve significant hazards considerations.

Local Public Document Room location: Hinds Junior College, McLendon Library, Raymond, Mississippi 39154.

Attorney for licensee: Nicholas S. Reynolds, Esquire, Bishop, Liberman, Cook, Purcell and Reynolds, 1200 17th Street, NW., Washington, DC 20036.

NRC Project Director: Walter R. Butler.

Mississippi Power & Light Company, Middle South Energy, Inc., South Mississippi Electric Power Association, Docket No. 50-418, Grand Gulf Nuclear Station, Unit 1, Claiborne County, Mississippi

Date of amendment request: September 12, 1986.

Description of amendment request:

This amendment would add maintenance and surveillance requirements for the Transamerica Delaval, Inc. (TDI) emergency diesel generators as an attachment to the Operating License. In addition, License Condition 2.C.(25)(b) would be changed to reference the new requirements. License Condition 2.C.(25)(b) now requires that recommendations from the TDI Owners Group Program applicable to GGNS Unit 1 and MP&L's actions in response to this program be submitted for review and approval prior to startup following the first refueling outage.

Basis for proposed no significant hazards consideration determination: The Commission has provided standards for determining whether a significant hazards consideration exists as stated in 10 CFR 50.92. A proposed amendment to an operating license for a facility involves no significant hazards considerations if operation of the facility in accordance with a proposed amendment would not: (1) Involve a significant increase in the probability or consequences of an accident previously evaluated; or (2) create the possibility of a new or different kind of accident from any accident previously evaluated; or (3) involve a significant reduction in a margin of safety.

The licensee has provided an analysis of significant hazards considerations in its request for a license amendment. The licensee has concluded, with appropriate bases, that the proposed amendment meets the three standards in 10 CFR 50.92 and, therefore, involves no significant hazards considerations.

The Commission has also provided guidance concerning the application of these standards by providing examples of amendments considered likely, and not likely, to involve a significant hazards consideration. These were published in the Federal Register on March 6, 1986 (51 FR 7744). The NRC staff has made a preliminary review of the licensee's submittal. A discussion of these examples as they relate to the proposed amendment follows.

One of the examples of actions involving no significant hazards consideration, (ii), is a change that constitutes an additional limitation, restriction or control not presently included in the Technical Specifications. The proposed amendment is similar to this example. The revised License Condition 2.C.(25)(b) and the referenced attachment would constitute controls on maintenance and surveillance requirements for the TDI emergency diesel generators in addition to those presently included in the Technical Specifications. The present License Condition 2.C.(25)(b) has been fulfilled by the licensee's June 7 and August 13, 1986 submittals regarding Design Review/Quality Revalidation (DR/QR) inspections recommended by the TDI Owners Group and by the July 18 and September 12, 1986 submittals regarding maintenance and surveillance requirements recommended by the TDI Owners Group. Because the present License Condition 2.C.(25)(b) has been fulfilled and the proposed amendment would add new controls or surveillance and maintenance for the TDI emergency diesel generators, the changes in this

proposed amendment are similar to example ii of the Commission's examples (51 FR 7744).

Accordingly, the Commission proposes to determine that these changes do not involve significant hazards considerations.

Local Public Document Room location: Hinds Junior College, McLendon Library, Raymond, Mississippi 39154.

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NRC Project Director: Walter R. Butler.

Mississippi Power & Light Company, Middle South Energy, Inc., South Mississippi Electric Power Association, Docket No. 50-416, Grand Gulf Nuclear Station, Unit 1, Claiborne County, Mississippi

Date of amendment request: September 15, 1986.

Description of amendment request: This amendment would change the Technical Specifications by adding three containment isolation valves to Table 3.6.4-1; one inboard valve in the post accident sampling system connection to Penetration No. 71B, and one inboard and one outboard valve in the test line to Penetration No. 71B.

Basis for proposed no significant hazards consideration determination: The Commission has provided standards for determining whether a significant hazards consideration exists as stated in 10 CFR 50.92. A proposed amendment to an operating license for a facility involves no significant hazards considerations if operation of the facility in accordance with a proposed amendment would not: (1) involve a significant increase in the probability or consequences of an accident previously evaluated; or (2) create the possibility of a new or different kind of accident from any accident previously evaluated; or (3) involve a significant reduction in a margin of safety.

The licensee has provided an analysis of significant hazards considerations in its request for a license amendment. The licensee has concluded, with appropriate bases, that the proposed amendment meets the three standards in 10 CFR 50.92 and, therefore, involves no significant hazards considerations.

The Commission has also provided guidance concerning the application of these standards by providing examples of amendments considered likely, and not likely, to involve a significant hazards consideration. These were published in the Federal Register on March 6, 1986 (51 FR 7744). The NRC

staff has made a preliminary review of the licensee's submittal. A discussion of these examples as they relate to the proposed amendment follows.

One of the examples of actions involving no significant hazards consideration, (ii), is a change that constitutes an additional limitation, restriction, or control not presently included in the Technical Specifications. The proposed change is similar to this example because the addition of the three isolation valves in Table 3.6.4-1 results in additional limiting conditions for operation of the plant. Technical Specification 3.6.4 requires that valves listed in Table 3.6.4-1 must be operable when the plant is in hot shutdown, startup or power operational conditions.

Accordingly, the Commission proposes to determine that this change does not involve significant hazards considerations.

Local Public Document Room location: Hinds Junior College, McLendon Library, Raymond, Mississippi 39154.

Attorney for licensee: Nicholas S. Reynolds, Esquire, Bishop, Lieberman, Cook, Purcell and Reynolds, 1200 17th Street NW., Washington, DC. 20036.

NRC Project Director: Walter R. Butler.

Nebraska Public Power District, Docket No. 50-296, Cooper Nuclear Station, Nemaha County, Nebraska

Date of amendment request: September 5, 1986.

Description of amendment request: The amendment would modify the Technical Specification (TS) to reflect the removal of the reactor vessel head spray system. The spray nozzle within the reactor vessel, the head spray piping between the reactor vessel and the refueling bulkhead, and the containment isolation valves (RHR-MOV-32 and RHR-MOV-33) will be removed. Blind flanges will be installed on the vessel head and bulkhead penetration flanges. The outboard isolation valve (RHR-MOV-33) will be replaced by a welded cap. The proposed changes to the Technical Specifications would (1) delete the operability and surveillance requirements for containment isolation valves RHR-MOV-32 and RHR-MOV-33, (2) delete the aforementioned valves from Table 3.7.1 which identifies the primary containment isolation valves, (3) delete the aforementioned valves from Table 3.7.4 which identifies the testable containment penetrations associated with the isolation valves, and (4) delete the head spray function from the listing of containment isolation groups in Table 3.2.A.

Basis for proposed no significant hazards consideration determination: The reactor vessel head spray system is part of the Residual Heat Removal (RHR) System. It was provided for the purpose of facilitating plant shutdown by aiding reactor vessel head cooldown. At Cooper Nuclear Station and other similar facilities, head spray has proven unnecessary. Removal of the head spray system will decrease maintenance requirements and reduce personnel radiation exposure.

The Commission has provided standards for determining whether a significant hazards determination exists as stated in 10 CFR 50.92(c). A proposed amendment to an operating license involves no significant hazards considerations if operation of the facility in accordance with the proposed amendment would not: (1) involve a significant increase in the probability or consequences of an accident previously evaluated, or (2) create the possibility of a new or different kind of accident from any accident previously evaluated, or (3) involve a significant reduction in a margin of safety.

The licensee has provided the following analysis of the proposed amendment with respect to the Commission standards:

(1) Does the proposed change involve a significant increase in the probability or consequences of an accident previously evaluated?

No. The proposed change will decrease the probability of a leak from the reactor coolant system and reduce the potential for a small break loss of coolant accident. Also, since blind flanges and welded-in pipe caps are less subject to leakage and not subject to failure-to-close, as are motor-operated valves, primary containment integrity is enhanced. Since no credit is given to reactor head spray in the facility safety design basis for accident mitigation, the modification does not impact the consequences of a previously evaluated accident.

(2) Does the proposed change create the possibility of a new or different kind of accident from any accident previously evaluated?

No. The capped-off piping outside the drywell will not be physically or functionally connected to any system, component, or equipment in a manner which could create a new or different kind of accident. Due to the welded-in cap, the remaining piping will be dead-ended to flow. Primary containment integrity will be insured by performance of 10 CFR 50 Appendix J leakage tests.

(3) Does the proposed amendment involve a significant reduction in a margin of safety?

No. The head spray piping penetration will continue to be testable as a spare penetration and subject to local leak rate test requirements. The head spray nozzle and piping have no safety function and their removal will not affect the capability of the remainder of the RHR system to perform its safety function. The proposed modification will, in fact, improve safety by slightly reducing LOCA potential and improving containment integrity. Therefore, the proposed amendment involves no significant reduction in a margin of safety.

Since the application for amendment involves proposed changes that are encompassed by the criteria for which no significant hazards consideration exists, the staff has made a proposed determination that the application involves no significant hazards consideration.

Local Public Document Room location: Auburn Public Library, 118 15th Street, Auburn, Nebraska 68305.

Attorney for licensee: Mr. G.D. Watson, Nebraska Public Power District, Post Office Box 499, Columbus, Nebraska 68601.

NRC Project Director: Daniel R. Muller.

Nebraska Public Power District, Docket No. 50-298, Cooper Nuclear Station, Nemaha County, Nebraska

Date of amendment request: September 17, 1986.

Description of amendment request: The amendment would modify the Technical Specifications to (1) indicate that barrier fuel is now included in the reactor design, (2) revise Minimum Critical Power Ratio (MCPR) limits, and (3) specify a MAPHLGR reduction factor of 0.77 for single-loop operation with barrier fuel. The affected sections of the Technical Specifications are (1) Section 5.2.A "Major Design Features-Reactor", (2) Section 3.11.C "Minimum Critical Power Ratio" and (3) Section 3.11.A "Average Planar Linear Heat Generation Rate".

Basis for proposed no significant hazards consideration determination: The Commission has provided guidance for the application of criteria for no significant hazards consideration determination by providing examples of amendments that are considered not likely to involve significant hazards considerations (51 FR 7751). These examples include:

(iii) For a nuclear power reactor, a change resulting from a nuclear reactor core

reloading, if no fuel assemblies significantly different from those found previously acceptable to NRC for a previous core at the facility in question are involved. This assumes that no significant changes are made to the acceptance criteria for the technical specifications, that the analytical methods used to demonstrate conformance with the technical specifications and regulations are not significantly changed, and that NRC has previously found such methods acceptable.

In the staff Safety Evaluation for Amendment 93, supporting the Cycle 10 reload, the use of barrier fuel was approved. The staff Safety Evaluation stated that use of barrier fuel has been previously approved and no further review of the fuel design is required. Although Amendment 93 approved the use of barrier fuel for future cycles, it did not actually amend the Cooper Technical Specifications to indicate actual installation of barrier fuel, as barrier fuel was not included in the Cycle 10 core design. The proposed amendment would specify the actual installation of barrier fuel in the upcoming Cycle 11 and future core designs.

In Amendment 94, single-loop operation was approved for the current and future cycles with the condition that the MAPHLGR be reduced by various factors depending on the types of fuel installed. A reduction factor for barrier fuel was not included in Amendment 94 since barrier fuel was not installed at the time. The proposed amendment would add a reduction factor of 0.77 for single-loop operation with barrier fuel. The barrier fuel MAPHLGR reduction factor is the same as for similar non-barrier fuel.

The revised MCPR figure would reflect the Cycle 11 reload transient analysis.

These changes are within the scope of criterion (iii). Since the application for amendment involves proposed changes that are encompassed by the criteria for which no significant hazards consideration exists, the staff has made a proposed determination that the application involves no significant hazards consideration.

Local Public Document Room location: Auburn Public Library, 118 15th Street, Auburn, Nebraska 68305.

Attorney for licensee: Mr. G.D. Watson, Nebraska Public Power District, Post Office Box 499, Columbus, Nebraska 68601.

NRC Project Director: Daniel R. Muller.

Niagara Mohawk Power Corporation, Docket No. 50-220, Nine Mile Point Nuclear Station, Unit No. 1, Oswego County, New York

Date of amendment request: August 22, 1986.

Description of amendment request: The proposed amendment would add surveillance requirements to Technical Specification (TS) Section 4.4.5 as requested by the NRC staff in the Safety Evaluation transmitted with TS Amendment 73. Currently, the TS do not require a test to verify that the control room air treatment system can provide a positive pressure in the control room. The control room air treatment system is designed to provide a positive pressure in the control room during accident conditions. By maintaining the control room pressure positive compared to adjacent areas in order to assure that all leakage is out-leakage, control room habitability during accident conditions is assured.

Basis for proposed no significant hazards consideration determination: The Commission has provided standards for determining whether a significant hazards consideration exists as stated in 10 CFR 50.92(c).

The licensee has presented its determination of no significant hazards consideration as follows:

10 CFR 50.91 requires that at the time a licensee requests an amendment, it must provide to the Commission its analysis, using the standards in Section 50.92, about the issue of no significant hazards consideration. Therefore, in accordance with 10 CFR 50.91 and 10 CFR 50.92, the following analysis has been performed:

The operation of Nine Mile Point Unit 1, in accordance with the proposed amendment will not involve a significant increase in the probability or consequences of an accident previously evaluated.

The addition of this surveillance test to the technical specifications will verify the capability of the control room air treatment system to meet its intended design of providing a positive pressure in the control room under accident conditions. Therefore, adding this test to the technical specifications will not involve a significant increase in the probability or consequences of an accident previously evaluated.

The operation of Nine Mile Point Unit 1, in accordance with the proposed amendment will not create the possibility of a new or different kind of accident from any accident previously evaluated.

The operation of Nine Mile Point Unit 1 in accordance with this proposed amendment will essentially remain the same. Additional testing of the control room air treatment system will not create the possibility of a new or different kind of accident from any accident previously evaluated. The test consists of simply reading pressure gauges at the control room boundary and will not

interfere with operations or the function of safety systems.

The operation of Nine Mile Point Unit 1, in accordance with the proposed amendment will not involve a significant reduction in a margin of safety.

This additional testing will not decrease the margin of safety at Nine Mile Point Unit 1. Since similar testing was not previously required by our Technical Specifications, this addition to the surveillance requirements of our Technical Specifications will increase our ability to assess the functional operation of our control room air treatment system. According to our current Technical Specification Bases, the Control Room Ventilation System can maintain a "positive pressure" in the Control Room. This proposal changes the bases to indicate that the Control Room Ventilation System can maintain "one-sixteenth of an inch positive pressure" within the control room. Therefore, the margin of safety will not decrease.

Therefore, based on the above considerations, it has been determined that the proposed amendment does not involve a significant hazards consideration.

The staff has reviewed the licensee's no significant hazards consideration determination and agrees with the licensee's analysis. Therefore, the staff proposes to determine that the application for amendment involves no significant hazards consideration.

Local Public Document Room location: State University of New York, Penfield Library, Reference and Documents Department, Oswego, New York 13126.

Attorney for licensee: Troy B. Conner, Jr., Esquire, Conner and Wetterhahn, Suite 1050, 1747 Pennsylvania Avenue, NW, Washington, DC 20006.

NRC Project Director: John A. Zwolinaki.

Northeast Nuclear Energy Company, Docket No. 50-423, Millstone Nuclear Power Station, Unit 3 New London County, Connecticut

Date of application for amendment: September 5, 1986.

Description of amendment request: The amendment would revise Technical Specifications Sections 4.6.6.1, 4.7.7, 4.7.9 and 4.9.12 by replacing the 31-day requirement to use certified fan curves to verify building filtration system flow rates with a direct flowrate measurement and, deleting the 18 month requirement to verify the fan curves based on observed flow rates and pressure drops.

Basis for proposed no significant hazards consideration determination: The staff has evaluated this proposed amendment and determined that it involves no significant hazards considerations. According to 10 CFR 50.92(c), a proposed amendment to an operating license involves no significant

hazards considerations if operation of the facility in accordance with the proposed amendment would not:

- (1) Involve a significant increase in the probability or consequences of an accident previously evaluated; or
- (2) Create the possibility of a new or different kind of accident from any accident previously evaluated; or
- (3) Involve a significant reduction in a margin of safety.

This proposed amendment requires the filtration system flowrates to be verified within a range of a specific value. Presently the technical specifications require use of fan curves to verify system flowrates against the system pressure drop with no specific value given. Therefore, the proposed surveillance requirements are more stringent than those currently in the technical specifications.

Since the proposed amendment does not modify the surveillance frequency, the changes will not affect system reliability. Direct flow measurements will increase the accuracy of the flow verification. This will improve the surveillance test verification that system flow rates are within unit design parameters.

Based on this information, the frequency of occurrence or the consequences of an accident or malfunction of equipment important to safety previously evaluated in the safety analysis report is not increased.

Based on the preceding assessment, the staff believes this proposed amendment involves no significant hazards considerations.

Local Public Document Room location: Waterford Public Library, 49 Rope Ferry Road, Waterford, Connecticut 06385.

Attorney for licensee: Gerald Garfield, Esq., Day, Berry and Howard, City Place, Hartford, Connecticut 06103-3499.

NRC Project Director: Vincent S. Noonan.

Northern States Power Company, Docket No. 50-263, Monticello Nuclear Generating Plant, Wright County, Minnesota

Date of amendment request: May 5, 1986.

Description of amendment request: The proposed amendment would revise the Technical Specifications (TS) to include the changes as a result of a detailed review of the TS that occurred following the 1985 refueling and recirculation piping replacement outage. Several of these changes are administrative in nature or are to clarify the interpretation of existing TS. Specifically, the changes are as follows:

(1) In Section 1.0.A, clarify the definition of "Alteration of the Reactor Core" by adding words, "with the vessel head removed and fuel in the vessel," to the end of the first sentence.

(2) In Section 2.3, page 17, delete the partial sentence in the first line of the first paragraph. These words should have been deleted with a previous license amendment request but were left through an oversight.

(3) In Table 3.1.1, "Reactor Protection System Instrument Requirements," move the reference to Note 4 from "Refuel" column to the "Trip Function" column and add the following note to the table on page 30: 9. High reactor pressure and main steam line high radiation are not required to be operable when the reactor vessel head is unbolted. Add a reference to Note 9 to the table entries for high reactor pressure and main steam line high radiation on pages 28 and 29.

(4) Delete Note 1 of Table 4.1.1 on page 33 and Note 1 of Table 4.2.1 on page 63a. Delete Figure 4.1.1 and correct the List of Figures to reflect deletion of this Figure. Delete all references to Note 1 on both tables and replace with a requirement for monthly surveillance. Delete those portions of the 4.1 and 4.2 Bases on pages 41-45 and 72-76 which refer to variable surveillance frequencies. These changes eliminate the options of extending the surveillance intervals to a maximum of 3 months by application of Figure 4.1.1.

(5) Add a Note 9 to Table 4.2.1 on page 63a to state "Testing of SRM Not-Full-In rod block is not required if the SRM detectors are secured in the full-in position." Also add a reference to Note 9 on page 61 under item 8 of Rod Blocks. Change the item to read, "SRM Detector Not-Full-In Position instead of, ". . . Note in Start-Up Position." Change the sensor check requirement from "Note 2" to "None."

(6) In Table 4.2.1, expand the headings for main steam, HPCL, and RCIC isolation by adding a reference to the containment isolation group and add a new category for Group 2 and Group 3 containment isolation. Delete Note 7 and all reference to Note 7 in the Table. Add a new Note 10 to state, "Uses contacts from scram system. Tested and calibrated in accordance with Table 4.1.1 and 4.1.2." Add a reference to Note 10 for containment isolation Group 2 reactor low water level and drywell high pressure surveillance.

(7) Revise the Bases section to explain the surveillance testing requirements in Section 4.0 of the TS and add information to assist in understanding and interpreting this section.

(8) In Table 3.2.5, "ATWS Instrumentation Requirements" revise Note 1 to read: "When one of the two trip systems is made or found to be inoperable, restore the inoperable trip system to operable status within 14 days or place the plant in the specified required condition within the next eight hours. When both trip systems are inoperable, place the plant in the specified required condition within eight hours unless at least one trip system is sooner made operable."

(9) Revise Section 3.3.D, "Control Rod Accumulators," to clarify the operability requirements for control rod accumulators. Delete the last paragraph of Specification 3.3.D and redesignate items 1 and 2 under 3.3.D as items 3.3.D.1 (a) and (b). Reword the opening paragraph to state, "Control rod accumulators shall be operable in the Startup, Run, or Refuel modes except as provided below." Add Specification 3.3.D.2 as follows:

In the Refuel Mode, a rod accumulator may be inoperable provided:

- (a) All fuel is removed from the cell containing the associated control rod, or
- (b) The one-rod-out refuel interlock for the associated rod drive is operable.

(10) Revise Sections 3.5.D.1, "High Pressure Coolant Injection (HPCI) System," Section 3.5.E.1, "Automatic Pressure Relief System (APRS)," and Section 3.5.F.1, "Reactor Core Isolation (RCIC) System," so that the operability of these systems is not required above 150 psig during reactor coolant system leakage and hydrostatic tests.

(11) In Section 3.7.A.1, "Primary Containment," reword the first paragraph to allow draining of the suppression chamber when irradiated fuel is not in the reactor vessel as follows, "When irradiated fuel is in the reactor vessel and either the reactor coolant temperature is greater than 212° F or work is being done which has the potential to drain the vessel, the following requirements shall be met except as permitted by Specifications 3.5.G.4. . . ."

(12) In Section 3.7.C.2.b, delete the phrase ". . . and the reactor coolant system is vented," since the requirements for the reactor to be vented as a condition for not requiring secondary containment integrity conflicts with normal and reasonable activities during outages.

(13) In Section 3.10.E, "Extended Core and CRD Maintenance", delete Specification 3.10.E.2 and redesignate Specification 3.10.E.1 and 3.10.E.3. Reword the first portion of the Specification to read, "More than one control rod may be withdrawn from the reactor core during outages provided that, except for

momentary switching to the startup mode for interlock testing, the reactor mode switch is locked in the refuel position. The refueling interlock. . . ." Change "withdrawn control rod" to "control rod" in two locations. Existing TS 3.10.E.2 is totally redundant to TS 3.10.B and therefore unnecessary and this change also clears the conflict between existing TS 3.10.E.1 which requires the mode switch to be locked "Refuel," and TS 4.10.A, which requires weekly check of the refueling interlocks requiring switching momentarily to the "startup" mode.

(14) In Section 3.14, "Accident Monitoring Instrumentation," clarify the operability requirements by revising the words "Whenever the reactor is in the startup or run mode," to "Whenever irradiated fuel is in the reactor vessel and reactor coolant water temperature is greater than 212° F." The revised wording allows testing and other normal operations during outages and is consistent with other accident mitigation system operability requirements (i.e., above 212° F) and NRC Standard TS. In addition, revise the notes to Table 3.14.1 to require placing the plant in the cold shutdown condition within 24 hours when required conditions of instrument operability are not satisfied.

(15) In Tables 3.14.1 and 4.14.1, add the operability and surveillance requirements for the suppression pool temperature monitoring instrumentation as required by the Mark I Containment Long-Term Improvement Program to accurately monitor suppression pool average temperature.

(16) In Table 4.14.1, "Minimum Test and Calibration Frequency for Accident Monitoring Instrumentation," provide additional notes to clarify sensor check requirements for reactor water level, SRV valve position pressure switches, and SRV valve position thermocouples as follows: (2) Once/month sensor check will consist of verifying that the pressure switches are not tripped. (3) Once/month sensor check will consist of verifying fuel zone level indicates off scale high. (4) Following every Safety Relief Valve actuation it will be verified that recorder traces or computer logs indicate sensor responses. Add a reference to Note 2 for SRV position pressure switches. Add a reference to Note 3 for reactor vessel fuel zone water level, and add a reference to Note 4 for SRV position pressure switches and thermocouples.

Basis for proposed no significant hazards consideration determination: The Commission has provided standards for determining whether a significant hazard determination exists

as stated in 10 CFR 50.92(c). 10 CFR 50.91 requires that at the time a licensee requests an amendment, it must provide to the Commission its analysis using the standards in 10 CFR 50.92, about the issue of no significant hazards considerations. Therefore, in accordance with 10 CFR 50.92, the following analysis has been performed by the licensee.

(1) The proposed amendment will not involve a significant increase in the probability or consequences of an accident previously evaluated.

The proposed amendment would (1) clarify the definition of Core Alteration, (2) correct a typographical error in the Section 2.3 Bases, (3) correct and clarify the Startup Mode operability requirements for high drywell pressure, high reactor pressure, and main steam line high radiation, (4) delete the obsolete provisions of the Technical Specifications which allow surveillance intervals to be extended, (5) correct conflicts with the SRM-Not-Full-In rod block interlock and CRD maintenance, (6) correct and clarify the surveillance requirements for containment isolation instrumentation, (7) provide an additional section to the Bases related to general surveillance requirements, (8) correct the action statements for ATWS instrumentation to correspond with the as-installed logic, (9) clarify CRD accumulator operability requirements, (10) correct the HPCI, RCIC, and APRS operability requirements to permit reactor coolant system leakage and hydrostatic testing, (11) clarify the requirements for containment integrity when no fuel is in the reactor, (12) correct and clarify the relationship between secondary containment requirements and reactor venting, (13) clarify the requirements for extended CRD maintenance, (14) correct and clarify the operability conditions for accident monitoring instrumentation, (15) add Technical Specifications limiting conditions for operation and surveillance requirements for suppression pool temperature monitoring instrumentation, and (16) clarify the meaning of sensor checks for safety/relief valve position pressure switches and reactor fuel zone water level instrumentation.

With the exception of item 2, which corrects a typographical error, and item 15, which adds Technical Specification requirements for a new instrumentation system, all of these changes have the intent of eliminating conflicts and interpretation problems in the Technical Specifications.

These items were identified during a detailed review of the Technical

Specifications by senior SRO licensed members of the Monticello technical staff. This review was made to fulfill a commitment made to NRC Region III and NRC NRR management following the discovery during the last refueling and maintenance outage of a number of conflicts in the Technical Specifications.

With the exception of items 2 and 15, these changes improve the clarity and logic of the wording in the Technical Specifications. While some relief from impossible or unreasonable restrictions is granted in several instances (e.g. HPCI will no longer be required operable during hydrostatic tests—but because the vessel is filled solid with subcooled water during these tests it is an impossible condition to impose), the requested changes will not, in any significant way, change the way the plant is operated or maintained.

Item 2 is a purely administrative change. Item 15 adds new requirements for an instrumentation system installed to meet the requirements of the NRC approved Mark I Containment Long Term Program and NRC Regulatory Guide 1.97, Revision 2. It is a new instrumentation system which will enhance the information available to plant operators during normal and postulated accident conditions.

Since the requested changes will not, in any significant way, affect any aspect of plant operation or maintenance or relax, in any significant way, valid limitations placed on systems and equipment, they will not increase the probability of (or) consequences of any previously evaluated accident.

(2) The proposed amendment will not create the possibility of a new or different kind of accident from any accident previously evaluated.

As discussed above, item 2 is an administrative change which corrects a typographical error. Item 15 adds additional requirements to the Technical Specifications for a new instrumentation system. The remainder of the requested changes make desirable clarifications and remove conflicts from the Technical Specifications. Since the requested changes will not, in any significant way, affect any aspect of plant operation or maintenance or relax, in any significant way, valid limitations placed on systems and equipment, they will not create the possibility of a new or different kind of accident from any accident previously analyzed.

(3) The proposed amendment will not involve a significant reduction in the margin of safety.

As discussed above, the proposed changes involve the corrections of a typographical error, adding limiting conditions for operation and

surveillance requirements for a new monitoring instrument, clarifications, changes which remove conflicts between various sections of the Technical Specifications, and a number of changes which eliminate impossible or unreasonable limitations on plant systems and components. This latter group of changes may be considered relief from restrictions imposed by the Technical Specifications, but in every case the proposed change will not in any significant way, change any aspect of plant operation and maintenance or relax, in any significant way, valid limitations placed on systems and equipment. Therefore no proposed change significantly reduces any margin of safety as described in the Technical Specifications or Updated Safety Analysis Report.

The Commission has provided guidance concerning the application of the Standards for determining whether a significant hazards consideration exists by providing certain examples of amendments that are considered not likely to involve significant hazards considerations. These examples were published in the Federal Register on March 8, 1986.

Item 2 of this application is representative of a purely administrative change presented as NRC example (i). Items 4 and 15 of this application are similar to NRC example (ii) since they consist of additional limitations, restrictions, or controls not presently in the Technical Specifications. The remaining items are similar to NRC example (i) since they can be described as corrections of errors, correction of nomenclature, and changes necessary to achieve consistency.

The staff has reviewed the licensee's no significant hazards consideration determination and agrees with the licensee's analysis. Therefore, based on this review, the staff has made a proposed determination that the application for amendment involves no significant hazards consideration.

Local Public Document Room location: Minneapolis Public Library, Technology and Science Department, 300 Nicollet Mall, Minneapolis, Minnesota 55401.

Attorney for licensee: Gerald Charnoff, Esq., Shaw, Pittman, Potts and Trowbridge, 1800 M Street NW., Washington, DC 20036.

NRC Project Director: John A. Zwolinski.

Power Authority of the State of New York, Docket No. 50-333, James A. FitzPatrick Nuclear Power Plant, Oswego County, New York

Date of amendment request: July 11, 1986.

Description of amendment request: The proposed amendment to the Technical Specifications (TS) revises Section 6.3 and Figure 6.2-1 to note the use of dual-role Senior Reactor Operator/Shift Technical Advisors (SRO/STA) in the operating shift organization. Provisions are maintained for optional use of a separate STA position and are also maintained for STA qualification of thirteen SRO's who have already completed the FitzPatrick Advanced Technical Training Program.

The proposed changes reflect the guidance contained in Generic Letter 86-04, "Policy Statement on Engineering Expertise on Shift," which specifies the qualifications of personnel eligible to fulfill the duty of STA and encourages licensees to utilize the dual-role position. In addition, several editorial changes have been proposed to reflect the above revisions.

Basis for proposed no significant hazards consideration determination. In accordance with the Commission's Regulations in 10 CFR 50.92, the Commission has made a determination that the proposed amendment involves no significant hazards considerations. To make this determination, the staff must establish that operation of the facility in accordance with the proposed amendment would not: (1) Involve a significant increase in the probability or consequences of an accident previously evaluated, or (2) create the possibility of a new or different kind of accident from any accident previously evaluated, or (3) involve a significant reduction in a margin of safety.

The proposed TS revisions do not involve a physical modification to the plant, a change in operating procedures, or a change in limiting conditions of operation. Additionally, the proposed revisions will not result in a decrease in expertise on shift or a change in the minimum shift complement, and are consistent with the guidance provided in Generic Letter 86-04. On these bases, plant operation in accordance with the proposed amendment would satisfy the three criteria stated above.

Based on the foregoing, the Commission proposes to determine that the proposed amendment does not involve a significant hazards consideration.

Local Public Document Room location: Penfield Library, State

University College of Oswego, Oswego, New York.

Attorney for licensee: Mr. Charles M. Pratt, Assistant General Counsel, Power Authority of the State of New York, 10 Columbus Circle, New York, New York 10019.

NRC Project Director: Daniel R. Muller.

Power Authority of The State of New York, Docket No. 50-286, Indian Point Unit No. 3, Westchester County, New York

Date of amendment request: July 22, 1986.

Description of amendment request: The licensee provided the following description:

A. Proposed Changes to Figure 6.2-1.

The proposed changes to figure 6.2-1 of the Indian Point 3 Technical Specifications illustrate the following changes in responsibility and management reorganization:

1. The Department of Quality Assurance and Reliability has been changed to the Department of Appraisal & Compliance Services. In addition, the title of the Vice President of Quality Assurance and Reliability has been changed to Senior Vice President—Appraisal and Compliance Services.
2. The Director of Safety and Fire Protection who previously reported to the Vice President—Quality Assurance and Reliability, now reports to the new position of Director of Security, Safety, and Fire Protection.
3. The Executive Vice President & Chief Engineer—Engineering and Design (formerly Executive Vice President—Chief Engineer) reports to the First Executive Vice President—Operations (formerly the First Executive Vice President and Chief Operations Officer). The position of First Executive Vice President and Chief Development Officer has been eliminated.

B. Proposed Changes to Figure 6.2-2.

The proposed changes to figure 6.2-2 reflect the change in title of the Senior Vice President—Quality Assurance & Reliability. In addition, a new position has been added. The Director—QA will report to the Senior Vice President—Appraisal and Compliance Services. Consequently, the QA Superintendent & Staff will now report to the Director—QA.

C. Proposed Changes to Subsection 6.5.2.2.

The proposed changes to subsection 6.5.2.2 of the Technical Specifications consists of the following changes:

1. The title of Vice President Nuclear Support—BWR has been changed to Vice President—Nuclear Operations;
2. The title of Vice President Nuclear Support—PWR has been changed to Vice President—Nuclear Engineering;
3. The title of Vice President—Generic Nuclear Support has been changed to Vice President—Nuclear Support.

Basis for proposed no significant hazards consideration determination:

The Commission has provided standards for determining whether a significant hazards consideration exists as stated in 10 CFR 50.92. A proposed amendment to an operating license for a facility involves no significant hazards considerations if operation of the facility in accordance with a proposed amendment would not: (1) involve a significant increase in the probability or consequences of an accident previously evaluated, or (2) create the possibility of a new or different kind of accident from any accident previously evaluated; or (3) involve a significant reduction in a margin of safety.

The licensee has provided the following analysis of these changes:

(1) Does the proposed license amendment involve a significant increase in the probability or consequences of an accident previously evaluated?

Response

The proposed changes described and evaluated above do not involve a significant increase in the probability or the consequences of an accident previously evaluated since the reorganization of the Authority is purely an administrative change and does not involve a hardware or procedural change to the facility. The chain of command from the President and Chief Operating Officer to the facility Resident Manager does not change in length or in personnel or SRC function. All personnel affected by the reorganization continue to meet the educational and experience levels described in the FSAR for positions previously having these responsibilities. This change will not adversely impact previously evaluated accidents.

(2) Does the proposed license amendment create the possibility of a new or different kind of accident from any accident previously evaluated?

Response

These changes do not create the possibility of a new or different kind of accident previously evaluated since the reorganization is designed to enhance the management and efficiency of the Authority. This cannot create the possibility of a new or different kind of accident.

(3) Does the proposed amendment involve a significant reduction in a margin of safety?

Response

The proposed changes do not involve a reduction in a margin of safety since all individuals affected by the reorganization described in this application continue to meet the educational and experience levels described in the FSAR for positions previously having these responsibilities.

Based on the above, the staff proposes to determine that the proposed changes do not involve a significant hazards consideration.

Local Public Document Room location: White Plains Public Library,

100 Martine Avenue, White Plains, New York 10601.

Attorney for licensee: Mr. Charles M. Pratt, 10 Columbus Circle, New York, New York 10019.

NRC Project Director: Steven A. Varga.

Public Service Company of Colorado, Docket No. 50-267, Fort St. Vrain Nuclear Generating Station, Platteville, Colorado

Date of amendment request: July 22, 1986.

Description of amendment request: The proposed amendment updates the Technical Specifications description of the Nuclear Operations organization for Public Service Company of Colorado. These changes do not directly affect plant operation.

Basis for proposed no significant hazards consideration determination: Since the proposed changes to Section 7.1 of the Fort St. Vrain Technical Specifications are administrative in nature, no significant safety hazards considerations are involved. Operation of Fort St. Vrain in accordance with the proposed changes will not: (1) involve a significant increase in the probability or consequences of an accident previously evaluated, (2) create the possibility of a new or different kind of accident from any accident previously evaluated, or (3) involve a significant reduction in a margin of safety. This change can be considered to come under example (i) of the examples provided by the Commission (51 FR 7751) of amendments that are considered not likely to involve significant hazards considerations.

Local Public Document Room location: Greeley Public Library, City Complex Building, Greeley, Colorado.

Attorney for licensee: Bryant O'Donnell, Public Service Company of Colorado, P.O. Box 840, Denver, Colorado 80201-0840.

NRC Project Director: Herbert N. Berkow.

Sacramento Municipal Utility District, Docket No. 50-312, Rancho Seco Nuclear Generating Station, Sacramento County, California

Date of amendment request: October 14, 1985, as revised February 13, 1986. (This request completely supersedes an application dated June 27, 1984, as amended on December 24, 1984 which was noticed on February 27, 1985 [50 FR 8005].)

Description of amendment request: The proposed amendment would delete from the Rancho Seco Technical Specifications (TSs) all references to

reactor vessel material surveillance. More specifically, TS Section 4.2.1 and Table 4.2-1 related to reactor vessel material surveillance and reporting requirements would be deleted, including the associated paragraph on supporting bases.

In addition, the proposed amendment requests withdrawal of the exemption from 10 CFR Part 50, Appendix H, contingent upon Commission approval of the Integrated Reactor Vessel Material Surveillance Program (IRVSP) documented in BAW-1543A, Revision 2, for Rancho Seco.

Basis for proposed no significant hazards consideration determination: In letters dated March 13 and May 3, 1985, the Commission concluded that the Babcock & Wilcox Owners Group (B&WOG) Materials Committee Report, BAW-1543, Revision 2, for an IRVSP was acceptable for reference in licensing applications.

Furthermore, in another letter to B&WOG dated May 8, 1985, the Commission stated any applicable licensee may formally request specific approval of the IRVSP in accordance with Section II.C of Appendix H, 10 CFR Part 50; and with such a request, each licensee may also submit a license amendment to remove the current reactor vessel material surveillance requirements from their TSs.

Following the guidance established by the aforementioned letters, the licensee has requested approval of the B&WOG IRVSP for Rancho Seco and has submitted a license amendment to delete the associated TS requirements. Additionally, the licensee has requested withdrawal of the exemption from 10 CFR Part 50, Appendix H, upon Commission approval of IRVSP for Rancho Seco.

In the proposed license amendment, the licensee maintains that operation of Rancho Seco in accordance with the proposed TS changes, to delete all references to the current reactor vessel material surveillance requirements, does not involve significant hazards considerations. This conclusion was based upon a licensee evaluation of the criteria for no significant hazards considerations prescribed by 10 CFR 50.92(c), which requires that a proposed amendment:

- (1) Would not involve a significant increase in the probability or consequences of an accident previously evaluated, or
- (2) Would not create the possibility of a new or different kind of accident from any accident previously evaluated, or
- (3) Would not involve a significant reduction in a margin of safety.

The Commission's staff has reviewed this proposed amendment and concurs with the licensee's conclusion of no significant hazards considerations. Removing reactor vessel material surveillance requirements from the TSs does not involve a change in system(s) configuration or operation of Rancho Seco. As such, it does not increase the probability or consequences of an accident, nor does it introduce the possibility of a new or different kind of accident. The Rancho Seco reactor vessel material surveillance program will be conducted in compliance with 10 CFR Part 50, Appendix H. Development of TS pressurization, heatup and cooldown limitations are based upon reactor vessel surveillance capsule analysis. There will be no significant reduction in the margin of safety because capsule analysis and the methodology for developing TS limitations are not appreciably changed by this amendment. Therefore, the Commission's staff proposes to determine that this application for amendment does not involve significant hazards considerations.

Local Public Document Room location: Sacramento City-County Library, 828 I Street, Sacramento, California 95814.

Attorney for licensee: David S. Kaplan, Sacramento Municipal Utility District, 6201 S Street, P.O. Box 15830, Sacramento, California 95813.

NRC Project Director: John F. Stolz.

Southern California Edison Company, et al, Docket Nos. 50-361 and 50-362, San Onofre Nuclear Generating Station, Units 2 and 3, San Diego County, California

Date of amendment request: June 13, 1986 (PCN-217).

Description of amendment request: The fuel handling building isolation system (FHIS) is designed to prevent the release of radioactivity from the fuel handling building (FHB) in the event of a fuel handling accident. The FHIS is actuated by high radiation level in the FHB, as detected by an airborne radiation monitor that measures noble gas activity. The current setpoint (130 cpm above normal background) has on several occasions been exceeded due to normal fuel handling activities in the FHB.

The proposed change would modify Table 3.3-4 of Technical Specification 3/4.3.2, "Engineered Safety Features Actuation System (ESFAS) Instrumentation", which provides a listing of trip values for various ESFAS instrumentation. Specifically, the change would revise the allowable noble gas alarm setpoint in Table 3.3-4 for the San

Onofre Nuclear Generating Station (SONGS) Units 2 and 3 fuel handling buildings. The proposed change would require that the trip setpoint be set "sufficiently high to prevent spurious alarms/trips, yet sufficiently low to assure an alarm/trip if a fuel handling accident should occur." A specific value for the setpoint was not proposed, because the background radiation level in the FHB will change with time as fuel is moved into and out of the FHB. The proposed wording will allow the licensee to select the appropriate setpoint for a given background level.

Basis for proposed no significant hazards consideration determination: The NRC staff proposes to determine that the proposed change does not involve a significant hazards consideration because, as required by the criteria of 10 CFR 50.92(c), operation of the facility in accordance with the proposed amendment would not: (1) Involve a significant increase in the probability or consequences of an accident previously evaluated; or (2) create the possibility of a new or different kind of accident from any accident previously evaluated; or (3) involve a significant reduction in the margin of safety. The basis for this proposed finding is given below.

(1) The proposed setpoint change would not involve a significant increase in the probability or consequences of the fuel handling accident previously evaluated. This proposed change will impose a new administratively controlled alarm setpoint high enough to prevent any spurious alarms resulting from normal fuel handling activities and yet sufficiently low to assure that the fuel handling isolation system (FHIS) will properly actuate in the event of a fuel handling accident. This requirement is similar to that used for the containment purge isolation system.

A study has been performed by the licensee to justify the proposed setpoint change. The study shows that the monitor response resulting from a design basis fuel handling accident of sixty (60) broken fuel rods is of the order of 497,000,000 cpm. A less severe accident involving only sixteen (16) failed fuel rods will give rise to 128,000,000 cpm. Thus, a conservative value for the setpoint can be determined which is greater than the highest ambient background level but well below the calculated monitor response to a fuel handling accident. This value would ensure early activation of the FHIS in the event of a fuel handling accident and would also eliminate nuisance alarms from either noise spikes or fuel handling operations. Thus, the revised setpoint

will not result in a reduction in the monitoring and isolation capability of the FHIS.

(2) No change to operating procedures for SONGS 2 and 3 is involved. Operations pertinent to fuel movement and reconstitution activities still fall within the scope of the existing fuel handling accident analysis. Therefore, the proposed change would not create the possibility of a new or different kind of accident from any accident previously evaluated.

(3) The proposed setpoint change would not involve a significant reduction in margin of safety even though it would increase the allowable technical specification alarm setpoint. The licensee's analysis of the monitor response to a fuel handling accident shows that the noble gas contamination levels under various accident circumstances far exceed ambient background levels at SONGS 2 and 3. A fuel handling accident will be detected by the FHB gaseous monitor with essentially the same level of confidence under the proposed change, because the revised setpoint will be maintained well below the radiation level that would result from a fuel handling accident.

As the change requested by the licensee's June 13, 1986 submittal satisfies the criteria of 10 CFR 50.92(c), it is concluded that: (1) The proposed change does not constitute a significant hazards consideration as defined by 10 CFR 50.92; (2) there is a reasonable assure that the health and safety of the public will not be endangered by the proposed change; and (3) this action will not result in a condition which significantly alters the impacts of the station on the environment as described in the NRC Final Environmental Statement.

Local Public Document Room
location: General Library, University of California at Irvine, Irvine, California 92713.

Attorney for licensees: Charles R. Kocher, Esq., Southern California Edison Company, 2244 Walnut Grove Avenue, P.O. Box 800, Rosemead, California 91770 and Orrick, Herrington & Sutcliffe, Attn.: David R. Pigott, Esq., 600 Montgomery Street, San Francisco, California 94111.

NRC Project Director: George W. Knighton.

Southern California Edison Company, et al., Docket Nos. 50-361 and 50-362z San Onofre Nuclear Generating Station, Units 2 and 3, San Diego County, California

Date of amendment request: August 22, 1986.

Description of amendment request: The proposed change would modify Technical Specification 3/4.9.8, "Refueling Machine." Specifically, the proposed change would revise the existing Limiting Conditions for Operation (LCOs) to reflect an increase of 200 pounds to the load limit for the refueling machine to accommodate the installation of a removable TV camera unit rather than a fixed TV camera on the refueling machine hoist box. The change would redefine the minimum capacity of the refueling machine from 3000 pounds to 3200 pounds. The overload cut off limit would also be changed to 3550 pounds instead of 3350 pounds.

Basis for proposed no significant hazards determination: The NRC staff proposes that the proposed change does not involve a significant hazards consideration because, as required by the criteria of 10 CFR 50.92(c), operation of the facility in accordance with the proposed amendment would not: (1) involve a significant increase in the probability or consequences of an accident previously evaluated; or (2) create the possibility of a new or different kind of accident from any accident previously evaluated; or (3) involve a significant reduction in the margin of safety. The basis for this proposed finding is given below.

(1) The probability or consequences of an accident are not increased by the proposed change since the removable TV camera unit meets the design criteria for Control Element Assemblies (CEA) and fuel assembly handling equipment specified in the Final Safety Analysis Report (FSAR) for SONGS Units 2 and 3.

(2) The increase in load limits will accommodate the installation of the removable TV camera unit. Since the overload limit is active only when the fuel assembly is enclosed in the hoist box, no fuel damage is credible with respect to the proposed setpoint change. Also, the added weight of the removable TV camera does not exceed the capacity of the refueling machine hoist mechanism. Thus, the operation of the facility in accordance with the proposed amendments will not create the possibility of a new or different kind of accident from any accident previously evaluated.

(3) The results of the change are clearly within all acceptance criteria with respect to the system or component specified in the Standard Review Plan; therefore, there is no reduction in the margin of safety previously established, since the operation of the refueling machine under the proposed LCOs will not present any increased potential for damage to CEAs or fuel assemblies, nor

will it affect the existing safety analyses and design criteria.

As the change requested by the licensee's August 22, 1986 submittal satisfies the criteria of 10 CFR 50.92(c), it is concluded that: (1) The proposed change does not constitute a significant hazards consideration as defined by 10 CFR 50.92; (2) there is a reasonable assurance that the health and safety of the public will not be endangered by the proposed change; and (3) this action will not result in a condition which significantly alters the impact of the station on the environment as described in the NRC Final Environmental Statement.

Local Public Document Room
location: General Library, University of California at Irvine, Irvine, California 92713.

Attorney for licensees: Charles R. Kocher, Esq., Southern California Edison Company, 2244 Walnut Grove Avenue, P.O. Box 800, Rosemead, California 91770 and Orrick, Herrington & Sutcliffe, Attn.: David R. Pigott, Esq., 600 Montgomery Street, San Francisco, California 94111.

NRC Project Director: George W. Knighton.

Toledo Edison Company and The Cleveland Electric Illuminating Company, Docket No. 50-348, Davis-Besse Nuclear Power Station, Unit No. 1, Ottawa County, Ohio

Date of amendment request: August 5, 1986.

Description of amendment request: The amendment would correct a clerical error, which has been incorporated into Amendment No. 93 to the Davis-Besse Technical Specifications (TSs), by inserting the word "or" after the words "within 7 days," in action statement "a" of Section 3.7.9.1.

Basis for proposed no significant hazards consideration determination: The Commission has provided guidance concerning the application of the standards for determining whether a significant hazards consideration exists by providing certain examples (51 FR 7750). One of the examples (f) of actions involving no significant hazards considerations relates to amendments of a purely administrative change to TSs; for example, a change to achieve consistency throughout the TSs, correction of an error, or a change in nomenclature. The proposed revision would make the reporting requirements of Section 3.7.9.1 consistent with the reporting requirements of fire protection TS Sections 3.3.3.8.b, 3.7.9.2.a, and 3.7.10.a, and correct an error, which would match this example of an

administrative change to TSs. The change would also make the reporting requirements consistent with requirements of the B&W Standard TSs.

On this basis, the Commission proposes to determine that the proposed amendment does not involve a significant hazards consideration.

Local Public Document Room

Location: University of Toledo Library, Documents Department, 2801 Bancroft Avenue, Toledo, Ohio 43606.

Attorney for licensee: Gerald Charnoff, Esq., Shaw, Pittman, Potts and Trowbridge, 1800 M Street NW., Washington, D.C. 20036.

NRC Project Director: John F. Stolz.

Virginia Electric and Power Company, Docket Nos. 50-338 and 50-339, North Anna Power Station, Units No. 1 and No. 2, Louisa County, Virginia

Date of amendments request: August 29, 1986.

Description of amendments request:

The proposed change would revise the NA-1&2 Technical Specifications (TS), Section 6 (Administrative Controls). Specifically, the proposed change would modify Section 6 as follows: (1) reflect a recent company reorganization in which the Quality Assurance (QA) organization will now report to the Senior Vice President—Engineering and Construction, instead of the Senior Vice President—Power Operations. (2) change the title of the QC Supervisors reporting to the Manager, QA from "Supervisors—Quality Control—Q.A. Activities" to "Supervisors Quality." (3) change the title of "Supervisor Health Physics" to "Superintendent—Health Physics." (4) change the title of "Director—Emergency Planning" to "Supervisor—Corporate Emergency Planning." and (5) change the facility organization chart to reflect the recent administrative title changes. Since the major emphasis of the company's nuclear program is on operations rather than construction, it is appropriate that the QA organization be realigned with construction to enhance the independence of the QA Organization. The remaining changes are purely administrative in nature involving changes in nomenclature as well as a change to achieve consistency with the NRC approved VEPCO QA Topical Report.

Basis for proposed no significant hazards consideration determination: The Commission has provided guidance concerning the application of the standards by providing certain examples which were published in the Federal Register on March 8, 1986 (51 FR 7751). Example (i) states: "A purely administrative change to technical

specifications: for example, a change to achieve consistency throughout the technical specifications, corrections of an error, or a change in nomenclature." The proposed change is enveloped by example (i) above, since the proposed change reflects the current reorganization of the Quality Assurance Organization to provide more emphasis on construction rather than operations and, also, to achieve consistency with the NRC approved VEPCO Quality Assurance Topical Report. It is noted that this change will enhance the independence of the licensee's Quality Assurance Organization. The other changes will provide consistency with the licensee's NRC approved Quality Assurance Topical Report and make administrative title changes to corporate and station organizations.

Accordingly, the Commission proposes to determine this change involves no significant hazards consideration.

Local Public Document Room

locations: Board of Supervisors Office, Louisa County Courthouse, Louisa, Virginia 23093 and the Alderman Library, Manuscripts Department, University of Virginia, Charlottesville, Virginia 22901.

Attorney for licensee: Michael W. Maupin, Esq., Hunton, Williams, Gay and Gibson, P.O. Box 1535, Richmond, Virginia 23212.

NRC Project Director: Lester S. Rubenstein.

Virginia Electric and Power Company, Docket Nos. 50-338 and 50-339, North Anna Power Station, Units No. 1 and No. 2, Louisa County, Virginia

Date of amendment request: September 12, 1986.

Description of amendment request:

The proposed changes would revise the NA-1&2 Technical Specifications (TS) in order to allow the tie-in, startup, and operation of a replacement spray system for the existing Service Water Spray System (SWSS) and its related components. The replacement spray system now being installed has been designed to the original code requirements of the NA-1&2 SWS. Per the repair and replacement rules of ASME Boiler and Pressure Vessel Code, Section XI, the system will meet the requirements of the Nuclear Power Piping Code, ANSI (formerly USAS) B 31.7—1969 Edition with addenda through 1970.

The operating and design bases for the replacement system are consistent with the original spray system. Increased operating flexibility has been provided by utilizing eight individual arrays (as opposed to the four existing

which cover a larger area of the reservoir. Operating experience with the existing system has led to the incorporation of a winter bypass feature to improve operability during extreme winter weather. The replacement spray system design provides additional margin between design heat rejection capability and required heat rejection capability through improvements in piping layout. The materials of construction and arrangement of the piping and support system minimize and facilitate routine maintenance. Easy access to the piping and associated supports is provided to facilitate periodic inspection and surveillance activities.

The proposed changes to the NA-1&2 would modify several component and structure tabulations to allow operation and surveillance of a replacement SW spray array system. The need to revise the TS arises primarily from the addition of equipment (i.e. replacement of motor-operated valves, piping, instrumentation) and the SW valve house and tie-in vault.

TS 3/4.3.3 addresses various types of monitoring instrumentation. Section 3/4.3.3.7 discusses Fire Detection Instrumentation. Table 3.3-11 requires revision to include fire detection instrumentation in the new SW valve house. This instrumentation consists of temperature detectors of the rate-compensated, electric-contact type similar to those used throughout the plant in areas such as the normal switchgear room, cable-tray-spreading room, primary cable vault and tunnels, etc. The minimum number of operable heat detectors required in the valve house is four: two in the west room and one in each of the east rooms. A total of seven heat detectors will be installed which includes three in the west room and two in each of the east rooms. The heat detection system for the SW valve house has been designed in accordance with applicable NFPA Standards and is consistent with the requirements outlined in the NA-1&2 UFSAR Section 9.5.1 in terms of spacing and location.

TS 3/4.7.12 addresses settlement of Class 1 structures. The new SW valve house and tie-in vault need to be included in the Settlement Monitoring program. Table 3.7-5 requires revision to include these two structures. In addition to the Limiting Condition for Operation (LCO) listed in Table 3.7-5, the Bases section of 3/4.7.12 also requires revision to include the valve house and tie-in vault monitoring points and their associated limiting items.

Four settlement markers will be added to the valve house and tie-in vault.

Baseline elevations will be established for the points prior to final system tie-in. At intervals defined in the TS, the elevations of these points will be measured by accurate survey. The baseline elevations will be periodically compared to current values; if the change exceeds prescribed limits, appropriate action is taken. The settlement monitoring points, limiting values and monitoring frequencies for the SW valve house and tie-in vault are consistent with the requirements for other Class I structures and satisfy the requirements outlined in the bases of TS and the NA-1&2 UFSAR Section 3.8.4.

TS 3/4.7.13 addresses the groundwater level in the SW reservoir. Table 3.7-6 requires revision to show the location of Piezometer No. 18 at the new SW valve house. This location is currently monitored as part of 3/4.7.13. The change proposed here is to change the designation of this point from "SWPH. (Units 3 and 4)" to "SWVH. (Units 1 and 2)."

TS 3/4.3.1 "A.C. Sources" addresses the A.C. electrical power sources. Table 4.8-1 provides a listing of load sequencing timers and this table requires revision to include new timers for the SW reservoir discharge Motor Operated Valves (MOV's). This change incorporates a 15 second time delay between the occurrence of a Safety Injection (SI) signal and the actuation of the replacement spray array MOVs. All bypass MOVs will receive SI signals to "close" and all spray array MOVs will receive SI signals to "open." However, in order to reduce the negative starting effects these MOVs would have on the emergency electrical distribution system, a time delay to start has been incorporated into the design. Delaying the operation of the spray array and bypass MOV's 15 seconds would not detrimentally affect the SW system for the following reasons:

(1) The additional heat dissipation requirement (above normal heat load) on the SW system during the first 15 seconds following a SI signal is negligible with the delayed MOV starting.

(2) The most significant heat load generated from the accident unit and removed by SW originates from the Recirculation Spray (RS) coolers. The RS system does not function until $t = 195$ seconds after the accident when the inside recirculation spray pumps start, provided a Containment Depressurization Signal (CDA) is present.

TS do not specify engineered safety feature response times for the SW system, therefore, delaying the operation of the SW spray array and bypass

MOV's for 15 seconds is justifiable in accordance with the above. Delaying the operation of these valves will have a positive effect on the station's electrical distribution system during accident scenarios (i.e., GCD-17 voltage profiles).

TS 3/4.8.2 "Onsite Power Distribution System" addresses the onsite power distribution system which must be operable. Table 3.8-2 identifies MOVs with thermal overload protectors and/or bypass devices. This table will be revised to reflect the addition of new MOVs. The table will also show that there are no bypass devices for these MOVs. In addition, since these valves are replacing the existing spray array motor-operated valves, the entries in the table for the existing MOV-SW200 A&B are being deleted.

The new valves meet or exceed the original design requirements of the existing valves and their design, including the motor thermal overload protection, and is consistent with the design basis of the SW system as outlined in the NA-1&2 UFSAR Section 9.2.1.

Basis for proposed no significant hazards consideration determination:

The Commission has provided standards for determining whether a significant hazards consideration exists as stated in 10 CFR Part 50.92(c). A proposed amendment to an operating license for a facility involves no significant hazards consideration if operation of the facility in accordance with the proposed amendment would not:

(1) involve a significant increase in the probability or consequences of an accident previously evaluated; or (2) create the possibility of a new or different kind of accident from any accident previously evaluated; or (3) involve a significant reduction in a margin of safety.

The proposed TS changes do not involve a significant hazards consideration because operation of NA-1&2 in accordance with these changes would not:

(1) Involve a significant increase in the probability or consequences of an accident previously evaluated. The replacement design and equipment (specifically, the fire detection instrumentation—Section 3/4.3.3.7, settlement markers and limits—Section 3/4.7.12, load sequence timers—Section 3/4.3.1 and motor operated valves with thermal overload protectors—Section 3/4.8.2) meet or exceed the original safety-related requirements of the existing SWS as noted above. Also, the change to TS 3/4.7.13 only involves a change in nomenclature;

(2) create the possibility of a new or different kind of accident from any accident previously identified. It has been determined that a new or different kind of accident will not be possible due to these changes. The design and operating bases of the SW replacement spray system are consistent with and meet or exceed the requirements of the existing system. No new accidents are introduced by the new design; or

(3) involve a significant reduction in a margin of safety. The margin of safety is not reduced since the replacement system serves the same purpose as the existing spray array system and the replacement design and equipment meet or exceed the original safety-related requirements of the existing SW system. The Limiting Condition for Operation and Surveillance Requirements of the TS sections 3/4.7.4 Service Water System and 3/4.7.5 Ultimate Heat Sink remain unchanged by the proposed modifications.

Therefore, the proposed changes meet the criteria specified in 10 CFR 50.92(c) and, thus, the NRC staff proposes to determine that the proposed changes involve no significant hazards considerations, and that operation of the facility in accordance with the proposed changes would not involve a significant hazards consideration.

Local Public Document Room location: Board of Supervisors Office, Louisa County Courthouse, Louisa, Virginia 23093 and the Alderman Library, Manuscripts Department, University of Virginia, Charlottesville, Virginia 22901.

Attorney for licensee: Michael W. Maupin, Esq., Hunton, Williams, Gay and Gibson, P.O. Box 1535, Richmond, Virginia 23212.

NRC Project Director: Lester S. Rubenstein.

Virginia Electric and Power Company, Docket Nos. 50-280 and 50-281, Surry Power Station, Unit Nos. 1 and 2, Surry County, Virginia

Date of amendment requests: August 22, 1986.

Description of amendment requests: The proposed changes would extend the duration of the Operating Licenses (DPR-32 and DPR-37) to 40 years from the date of issuance of the Operating Licenses. This request would allow for 40 full years of operation by changing the license expiration dates to May 25, 2012, for Unit 1 (DPR-32) and to January 29, 2013, for Unit 2 (DPR-37).

Basis for proposed no significant hazards consideration determination: The Commission has provided standards in 10 CFR 50.92(c) for

determining whether a proposed license amendment involves significant hazards considerations. The licensee has reviewed its amendment request and determined that the proposed amendments would not:

1. . . . involve a significant increase in the probability or consequences of any accident previously evaluated since no changes are required to the design or operation of the station. This [sic] amendment[s] do not involve new or revised safety analyses, physical plant modifications, procedure changes, [Updated Final Safety Analysis Report] UFSAR Revisions or Technical Specification revisions. The proposed license extensions are within the original design considerations for the station[;] and the current surveillance, inspection, testing and maintenance practices provide assurance that degradation in plant equipment will be identified and corrected throughout the lifetime of the facility.

2. . . . create the possibility of a new or different kind of accident from any accident previously evaluated since no changes are required to the design or operation of the station.

3. . . . involve a significant reduction in a margin of safety since no changes are required to the design or operation of the station and since the amendment[s] do not involve new or revised safety analyses, procedure changes, UFSAR revisions or Technical Specification revisions. The current surveillance, inspection, testing and maintenance practices provide assurance that degradation in plant equipment will be identified and corrected throughout the lifetime of the facility.

Based on the above considerations, the licensee concluded that there is no significant hazards consideration associated with the proposed revision to Surry Operating Licenses. The staff has reviewed the licensee's no significant hazard determination and agrees with the licensee's conclusions. Therefore, the staff proposes to determine that the requested amendments involve no significant hazard considerations.

Local Public Document Room location: Swem Library, College of William and Mary, Williamsburg, Virginia 23185.

Attorney for licensee: Mr. Michael W. Maupin, Hunton and Williams, Post Office Box 1535, Richmond, Virginia 23213.

NRC Project Director: Lester S. Rubenstein.

Wisconsin Electric Power Company, Docket Nos. 50-266 and 50-301 Point Beach Nuclear Plants, Unit Nos. 1 and 2, Town of Two Creeks, Manitowoc County, Wisconsin

Date of amendments request: August 28, 1986.

Description of amendments request: The proposed changes to the Technical

Specifications would revise the surveillance requirements for main steam stop valves, main steam safety valves, and pressurizer safety valves. The periodicity for testing main steam safety valves and pressurizer safety valves would be changed from once each refueling to once every five years. The test conditions for main steam stop valves closure times would be changed from no-flow conditions to low-flow conditions based upon the minimal steam flow that may exist under the proposed hot initial test condition.

Basis for proposed no significant hazards consideration determination: 10 CFR 50.92 states that the Commission may make a determination that a proposed amendment involves no significant hazards consideration if operation of the facility in accordance with the proposed amendment would not (1) involve a significant increase in the probability or consequences of an accident previously evaluated, (2) create the possibility of a new or different kind of accident from any accident previously evaluated, or (3) involve a significant reduction in a margin of safety.

The licensee has stated in support of the requested amendments that changing the main steam and pressurizer safety valve testing periodicity does not significantly increase the probability or consequences of an accident previously evaluated in that the periodicity requested is in compliance with the guidelines of a nationally accepted standard. The licensee also stated that the changing of the test conditions for main steam stop valve surveillance does not alter the initial conditions or consequences of the analyzed main steam line rupture accident as contained in the FSAR.

Regarding the second criterion, the licensee has stated that the changes are revisions to surveillance requirements and conditions. Thus, no new or different accident can be created as no changes or modification to the physical plant have occurred.

Regarding the third criterion, the licensee has stated that the changes would not involve a significant reduction in a margin of safety because the changes relative to main steam and pressurizer safety valve testing are a request for adherence to the guidelines of the ASME Code Section XI for inservice testing of safety valves. The purpose of this section of the Code is to ensure a sufficient margin of safety exists relative to the proper functioning of these components, verifiable through a specified testing periodicity. Also, no reduction in the margin of safety will occur with the new test conditions for

main steam stop valve surveillance. Since the applicable accident analysis remains unchanged, the margin of safety remains unaffected.

On the basis of the above analysis, the licensee has determined that the proposed amendments would not involve a significant hazards consideration. The staff has reviewed the licensee's determination that the proposed amendments would not involve a significant hazards consideration. The staff feels that the licensee has correctly addressed the three criteria contained in 10 CFR 50.92 and, therefore, proposes to determine that the amendments would involve no significant hazards consideration.

Local Public Document Room location: Joseph P. Mann Library, 1516 Sixteenth Street, Two Rivers, Wisconsin.

Attorney for licensee: Gerald Charnoff, Esq., Shaw, Pittman, Potts and Trowbridge, 1800 M Street, NW., Washington, DC 20036.

NRC Project Director: George E. Lear.

Wisconsin Electric Power Company, Docket Nos. 50-266 and 50-301 Point Beach Nuclear Plants, Unit Nos. 1 and 2, Town of Two Creeks, Manitowoc County, Wisconsin

Date of amendments request: August 29, 1986.

Description of amendments request: The proposed changes to the Technical Specifications (TS) would modify the limiting conditions for operation (LCO) of the component cooling water (CCW) system to correspond to changes in the system configuration in which another shared heat exchanger was added to the system. Specifically, the current TS require that a unit not be made critical unless both CCW heat exchangers which can be aligned to a unit are operable. This proposed change would allow one of three heat exchangers which can be aligned to a unit to be inoperable prior to startup.

A second change involves the number of heat exchangers which may be inoperable during power operation of either one or two units. The current TS allows one CCW heat exchanger to be out of service for 48 hours during power operation. The proposed change would allow two of the three heat exchangers which may be aligned to a unit to be inoperable for up to 48 hours.

A third change involves removing the limiting condition for operation based on one passive component other than a heat exchanger being out of service.

Basis for proposed no significant hazards consideration determination: The number of operable CCW heat

exchangers per unit which would be required by the proposed TS is not different than would be required by the current TS if the system had not been modified to add an additional shared heat exchanger. Therefore, the proposed amendments would not involve an increase in the probability or consequences of an accident previously evaluated, or create the possibility of a new or different kind of accident than any accident previously evaluated, or involve a significant reduction in a margin of safety. On this basis, the staff proposes to determine that the proposed amendments would not involve a significant hazards consideration.

Local Public Document Room
location: Joseph P. Mann Library, 1516 Sixteenth Street, Two Rivers, Wisconsin.

Attorney for licensee: Gerald Charnoff, Esq., Shaw, Pittman, Potts and Trowbridge, 1800 M Street, NW., Washington, DC 20036.

NRC Project Director: George E. Lear.

PREVIOUSLY PUBLISHED NOTICES OF CONSIDERATION OF ISSUANCE OF AMENDMENTS TO OPERATING LICENSES AND PROPOSED NO SIGNIFICANT HAZARDS CONSIDERATION DETERMINATION AND OPPORTUNITY FOR HEARING

The following notices were previously published as separate individual notices. The notice content was the same as above. They were published as individual notices because time did not allow the Commission to wait for this bi-weekly notice. They are repeated here because the bi-weekly notice lists all amendments proposed to be issued involving no significant hazards consideration.

For details, see the individual notice in the Federal Register on the day and page cited. This notice does not extend the notice period of the original notice.

Duke Power Company, Dockets Nos. 50-269, 50-270 and 50-267, Oconee Nuclear Station, Units Nos. 1, 2 and 3, Oconee County, South Carolina

Date of amendment request: June 30, 1986, as superseded September 2, 1986.

Brief description of amendment: The proposed amendments would revise the Station's common Technical Specifications (TSs) to support the operation of Oconee Unit 2 at full rated power during the upcoming Cycle 9. The proposed amendment request changes the following areas:

1. Core Protection Safety Limits (TS 2.1);
2. Protective System Maximum Allowable Setpoints (TS 2.3);
3. Rod Position Limits (TS 3.5.2); and

4. Power Imbalance Limits (TS 3.5.2).

To support the license amendment request for operation of Oconee Unit 2, Cycle 9, the licensee submitted, as an attachment to the application, a Duke Power Company (DPC) Report, DPC-RD-2007, "Oconee Unit 2, Cycle 9 Reload Report," dated June 1986. A summary of the Cycle 9 operating parameters is included in the report, along with safety analyses.

During the refueling outage, 117 fuel assemblies will be reinserted similar to those previously used, and 60 fuel assemblies will be discharged and replaced with new, but substantially similar, assemblies of the Mark BZ type. As in the previous cycle, Cycle 9 will utilize gray (less-absorbing) axial power shaping rods (APSRs) instead of the previously used black (highly-absorbing) APSRs. The use of the Mark BZ fuel assemblies and the gray APSRs was approved by the Commission's staff for use at Oconee Unit 1 during Cycle 9, in amendments dated November 23, 1984.

Date of publication of individual notice in Federal Register: September 11, 1986 (51 FR 32383).

Expiration date of individual notice: October 14, 1986.

Local Public Document Room
location: Oconee County Library, 501 West Southbroad Street, Walhalla, South Carolina 29691.

Florida Power Corporation, et al., Docket No. 50-302, Crystal River Unit No. 3 Nuclear Generating Plant, Citrus County, Florida

Date of amendment request: August 14, 1986.

Brief description of amendment: The proposed amendment would extend the surveillance interval from once per 18 months to once per fuel cycle, permanently for reactor vessel internals vent valves (RVVVs) and for Cycle 6 only for high pressure injection (HPI) and low pressure injection (LPI) pumps and valves.

Date of publication of individual notice in Federal Register: September 19, 1986 (51 FR 33322).

Expiration date of individual notice: October 20, 1986.

Local Public Document Room
location: Crystal River Public Library, 668 NW. First Avenue, Crystal River, Florida 32829.

GPU Nuclear Corporation and Jersey Central Power and Light Company, Docket No. 50-219, Oyster Creek Nuclear Generating Station, Ocean County, New Jersey

Date of amendment requests: September 5 and 9, 1986 (TSCR 153 and 147).

Brief description of amendment: The first proposed amendment would revise the footnote marked with an asterisk **** to Table 3.1.1, Protective Instrumentation Requirements, of the Appendix A Technical Specifications (TS). When it is necessary to conduct tests and calibrations of the protective instrumentative channels in accordance with the TS, the licensee proposes that one channel may be made inoperable for up to 2 hours without tripping the channel's trip system. This is instead of the existing requirement which allows that channel to be inoperable without tripping the trip system for only up to 1 hour per month. This first amendment is in accordance with the licensee's application dated September 5, 1986, for Technical Specification Change Request (TSCR) No. 153.

The second proposed amendment would (1) increase the high drywell pressure trip setpoint from not greater than 2.4 psig to not greater than 3.5 psig and (2) add a bypass to the high flow trip of the "B" Isolation Condenser when initiating the alternate shutdown panel. The licensee is proposing to increase the value of the high drywell pressure trip setting in Table 3.1.1 of the TS. This applies to reactor scram, core spray initiation, containment spray initiation, containment isolation, automatic reactor vessel depressurization, Reactor Building isolation and the Bases in Section 3.1 of the TS for the table. For the bypass, the licensee is proposing to add a footnote "hh" stating that the trip function is bypassed upon initiation of the alternate shutdown panel to prevent a spurious trip of the "B" Isolation Condenser in the event of fire induced circuit damage. This second amendment is in accordance with the licensee's application for amendment dated September 9, 1986, for TSCR 147.

Date of publication of individual notice in Federal Register: September 17, 1986 (51 FR 32980).

Expiration date of individual notice: October 17, 1986.

Local Public Document Room
location: Ocean County Library, 101 Washington Street, Toms River, New Jersey 08753.

Louisiana Power and Light Company, Docket No. 50-362, Waterford Steam Electric Station, Unit 3, St. Charles Parish, Louisiana

Date of amendment request: June 24, 1986, and supplemental letters dated August 4, 1986 and September 2, 1986.

Brief description of amendment: Technical Specification change to authorize an increase in the fuel enrichment limit.

Date of publication of individual notice in Federal Register: September 11, 1986 (51 FR 32383).

Expiration Date of Individual Notice: October 14, 1986.

Local Public Document Room location: University of New Orleans Library, Louisiana Collection, Lakefront, New Orleans, Louisiana 70122.

NOTICE OF ISSUANCE OF AMENDMENT TO FACILITY OPERATING LICENSE

During the period since publication of the last bi-weekly notice, the Commission has issued the following amendments. The Commission has determined for each of these amendments that the application complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment.

Notice of Consideration of Issuance of Amendment to Facility Operating License and Proposed No Significant Hazards Consideration Determination and Opportunity for Hearing in connection with these actions was published in the Federal Register as indicated. No request for a hearing or petition for leave to intervene was filed following this notice.

Unless otherwise indicated, the Commission has determined that these amendments satisfy the criteria for categorical exclusion in accordance with 10 CFR 51.22. Therefore, pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared for these amendments. If the Commission has prepared an environmental assessment under the special circumstances provision in 10 CFR 51.12(b) and has made a determination based on that assessment, it is so indicated.

For further details with respect to the action see (1) the applications for amendments, (2) the amendments, and (3) the Commission's related letters, Safety Evaluations and/or Environmental Assessments as indicated. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street NW., Washington, DC, and at the local public document rooms for the particular facilities involved. A copy of items (2) and (3) may be obtained upon request addressed to the U.S. Nuclear Regulatory Commission, Washington, DC 20555, Attention: Director, Division of Licensing.

Alabama Power Company, Docket Nos. 50-348 and 50-364, Joseph M. Farley Nuclear Plant, Unit Nos. 1 and 2, Houston County, Alabama

Date of application for amendments: February 7, 1986.

Brief description of amendments: Technical Specification (TS) 3.4.1.2 is revised to require all three reactor coolant loops to be operating in Mode 3 (Hot Standby) or that the rod control system be disabled. The existing Technical Specifications require only one coolant loop to be operating in Mode 3.

Date of issuance: September 9, 1986.

Effective date: September 9, 1986.

Amendment Nos.: 65 and 58.

Facilities Operating License Nos. NPF-2 and NPF-8. Amendments revised the Technical Specifications.

Date of initial notice in Federal Register: April 9, 1986 (51 FR 12223).

The Commission's related evaluation of the amendment is contained in a Safety Evaluation dated September 9, 1986.

No significant hazards consideration comments received: No.

Local Public Document Room location: George S. Houston Memorial Library, 212 W. Burdeshaw Street, Dothan, Alabama 36303.

Arkansas Power and Light Company, Docket No. 50-313, Arkansas Nuclear One, Unit No. 1, Pope County, Arkansas

Date of application for amendment: January 24, 1986.

Brief description of amendment: The amendment revised the Technical Specifications to delete the tabular listing of shock suppressors (snubbers) in accordance with the Commission's guidance contained in Generic Letter 84-13.

Date of issuance: September 18, 1986.

Effective date: September 18, 1986.

Amendment No.: 109.

Facility Operating License No. DPR-51. Amendment revised the Technical Specifications.

Date of initial notice in Federal Register: March 28, 1986 (51 FR 10453).

The Commission's related evaluation of the amendment is contained in a Safety Evaluation dated September 19, 1986.

No significant hazards consideration comments received: No.

Local Public Document Room location: Tomlinson Library, Arkansas Tech University, Russellville, Arkansas 72801.

Arkansas Power and Light Company, Docket No. 50-313, Arkansas Nuclear One, Unit No. 1, Pope County, Arkansas

Date of application for amendment: May 21, 1986.

Brief description of amendment: The amendment revised the Technical Specification minimum level requirement for emergency feedwater (EFW) condensate storage tank T41B due to the substitution of the new seismically qualified, partially tornado protected EFW condensate storage tank T41B for the original non-seismic, non-tornado protected condensate storage tank as the primary EFW system water source.

Date of issuance: September 26, 1986.

Effective date: September 26, 1986.

Amendment No.: 101.

Facility Operating License No. DPR-51. Amendment revised the Technical Specifications.

Date of initial notice in Federal Register: July 30, 1986 (51 FR 27278).

The Commission's related evaluation of the amendment is contained in a Safety Evaluation dated September 26, 1986.

No significant hazards consideration comments received: No.

Local Public Document Room location: Tomlinson Library, Arkansas Tech University, Russellville, Arkansas 72801.

Carolina Power and Light Company, Docket No. 50-261, H.B. Robinson Steam Electric Plant, Unit No. 2, Darlington County, South Carolina

Date of application for amendment: November 6, 1985.

Brief description of amendment: The amendment revises the Technical Specifications by eliminating the requirement for shutting down the ventilation system in the fuel handling building on a high radiation signal, reduces the waste gas decay tank radioactivity limit, and corrects the bases for the control of explosive gas mixtures in the waste gas decay tanks. The amendment also involves changes of an editorial nature.

Date of issuance: September 18, 1986.

Effective date: September 18, 1986.

Amendment No.: 103.

Facility Operating License No. DPR-23. Amendment revised the Technical Specifications.

Date of initial notice in Federal Register: May 21, 1986 (51 FR 18680).

The Commission's related evaluation of the amendment is contained in a Safety Evaluation dated September 18, 1986.

No significant hazards consideration comments received: No

Local Public Document Room
location: Hartsville Memorial Library, Home and Fifth Avenues, Hartsville, South Carolina 29535.

Connecticut Yankee Atomic Power Company, Docket No. 50-213, Haddam Neck Plant, Middlesex County, Connecticut

Date of application for amendment: June 6, 1986.

Brief description of amendment: The license amendment formalizes a requirement to perform a quadrant power tilt surveillance at least once per seven days. This surveillance test has been performed by administrative procedure at the above frequency since 1983, and is being formalized to satisfy a staff request made during the review of the Cycle 14 reload application. The surveillance requirement provides further assurance that the input assumptions of the transient analyses are valid.

Date of issuance: September 18, 1986.

Effective date: September 18, 1986.

Amendment No. 84.

Facility Operating License No. DPR-61. Amendment revised the technical specifications.

Date of initial notice in Federal Register: July 2, 1986 (51 FR 24252).

The Commission's related evaluation of the amendment is contained in a Safety Evaluation dated September 18, 1986.

No significant hazards consideration comments received: No.

Local Public Document Room
location: Russell Library, 124 Broad Street, Middletown, Connecticut 06457.

Connecticut Yankee Atomic Power Company, Docket No. 50-213, Haddam Neck Plant, Middlesex County, Connecticut

Date of application for amendment: August 6, 1986.

Brief description of amendment: Technical Specification 6.3.1(d) required the licensee to forward Monthly Operating Reports to the Director, Office of Management Information and Program Control within the Nuclear Regulatory Commission (NRC). As a result of NRC reorganizations, the addressee presently identified in the technical specifications is no longer applicable. This license amendment revises the current addressee to be consistent with the guidance found in the Standard Technical Specifications for this area.

Date of issuance: September 29, 1986.

Effective date: September 29, 1986.

Amendment No. 85.

Facility Operating License No. DPR-61. Amendment revised the technical specifications.

Date of initial notice in Federal Register: August 27, 1986 (51 FR 30563).

The Commission's related evaluation of the amendment is contained in a Safety Evaluation dated September 29, 1986.

No significant hazards consideration comments received: No.

Local Public Document Room
location: Russell Library, 124 Broad Street, Middletown, Connecticut 06457.

Duke Power Company, et al., Docket Nos. 50-413 and 50-414, Catawba Nuclear Station, Units 1 and 2, York County, South Carolina

Date of application for amendments: March 24, 1986, as supplemented June 30 and July 28, 1986.

Brief description of amendments: The amendments modify testing requirements for the diesel generators and the diesel generators' fuel oil storage requirements.

Date of issuance: September 15, 1986.

Effective date: September 15, 1986.

Amendment Nos.: 10 and 3.

Facility Operating License Nos. NPF-35 and NPF-52. Amendments revised the Technical Specifications.

Date of initial notice in Federal Register: June 18, 1986 (51 FR 22233).

The Commission's related evaluation of the amendments is contained in a Safety Evaluation dated September 15, 1986.

No significant hazards consideration comments received: No.

Local Public Document Room
location: York County Library, 138 East Black Street, Rock Hill, South Carolina 29730.

Duke Power Company, et al., Docket Nos. 50-413 and 50-414, Catawba Nuclear Station, Units 1 and 2, York County, South Carolina

Date of application for amendments: June 6, 1986.

Brief description of amendments: The amendments modify Technical Specifications to reflect the upgrade of the Reactor Coolant System Power Operated Relief Valves to safety grade for Catawba Unit 1.

Date of issuance: September 16, 1986.

Effective date: September 16, 1986.

Amendment Nos.: 11 and 4.

Facility Operating License Nos. NPF-35 and NPF-52. Amendments revised the Technical Specifications.

Date of initial notice in Federal Register: August 13, 1986 (51 FR 28996).

The Commission's related evaluation of the amendments is contained in a

Safety Evaluation dated September 16, 1986.

No significant hazards consideration comments received: No.

Local Public Document Room
location: York County Library, 138 East Black Street, Rock Hill, South Carolina 29730.

Duke Power Company, et al., Docket No. 50-413, Catawba Nuclear Station, Unit 1, York County, South Carolina

Date of application for amendment: June 6, 1986.

Brief description of amendment: The amendment permits an extension of time for the submittal of the steam generator tube rupture analysis.

Date of issuance: September 18, 1986.

Effective date: September 18, 1986.

Amendment No.: 12.

Facility Operating License No. NPF-35. Amendment revised the Operating License.

Date of initial notice in Federal Register: August 13, 1986 (51 FR 28996).

The Commission's related evaluation of the amendment is contained in a Safety Evaluation dated September 18, 1986.

No significant hazards consideration comments received: No.

Local Public Document Room
location: York County Library, 138 East Black Street, Rock Hill, South Carolina 29730.

Duke Power Company, Docket Nos. 50-369 and 50-370, McGuire Nuclear Station, Units 1 and 2, Mecklenburg County, North Carolina

Dates of applications for amendment: August 30, 1985, as supplemented December 13, 1985; July 22, 1985, as supplemented June 12, 1986; and January 10, 1986, as supplemented May 12, 1986.

Brief description of amendments: The amendments change the Technical Specifications to authorize use of the "Turbine Overspeed Reliability Assurance Program" for demonstrating operability of the turbine overspeed protection system, to increase the time during which an inoperable turbine stop valve instrument channel may be maintained in an untripped condition, and to increase the number of reactor coolant loops required to periodically be verified in operation in the hot standby mode.

Date of issuance: September 17, 1986.

Effective date: September 17, 1986.

Amendment Nos.: 62 and 43.

Facility Operating License Nos. NPF-9 and NPF-17. Amendments revised the Technical Specifications.

Dates of initial notices in Federal Register: December 18, 1985 (50 FR

51622); July 30, 1986 (51 FR 27283); June 18, 1986 (51 FR 22234).

The Commission's related evaluation of the amendments is contained in a Safety Evaluation dated September 17, 1986.

No significant hazards consideration comments received: No.

Local Public Document Room
location: Atkins Library, University of North Carolina, Charlotte (UNCC Station), North Carolina 28223.

Florida Power Corporation, et al.,
Docket No. 50-302, Crystal River Unit No. 3 Nuclear Generating Plant, Citrus County, Florida

Date of application for amendment:
April 23, 1984.

Brief description of amendment: This amendment permits the operation of certain containment isolation valves when they would normally be required to be isolated, provided that a dedicated operator is posted to isolate the valve if necessary. A portion of the amendment request has been denied by the Commission. A Notice of Denial is being published separately in the Federal Register.

Date of issuance: September 16, 1986.

Effective date: September 16, 1986.

Amendment No.: 91.

Facility Operating License No. DPR-72. Amendment revised the Technical Specifications.

Date of initial notice in Federal Register: November 21, 1984 (49 FR 45948).

The Commission's related evaluation of the amendment is contained in a Safety Evaluation dated September 16, 1986.

No significant hazards consideration comments received: No.

Local Public Document Room
location: Crystal River Public Library, 668 NW. First Avenue, Crystal River, Florida 32829.

Georgia Power Company, Oglethorpe Power Corporation, Municipal Electric Authority of Georgia, City of Dalton, Georgia, Docket No. 50-321, Edwin L. Hatch Nuclear Plant, Unit No. 1, Appling County, Georgia

Date of application for amendments:
April 15, 1986.

Brief description of amendments: The amendment updates TS Tables 3.7-1 and 3.7-4 to reflect the current plant design with respect to primary containment isolation valves (PCIVs). It also revises Section 3.7.D.1 to require that all PCIVs be operable.

Date of issuance: September 25, 1986.

Effective date: September 25, 1986.

Amendment No.: 129.

Facility Operating License No. DPR-57. Amendment revised the Technical Specifications.

Date of initial notice in Federal Register: July 30, 1986 (51 FR 27283).

The Commission's related evaluation of the amendment is contained in a Safety Evaluation dated September 25, 1986.

No significant hazards consideration comments received: No.

Local Public Document Room
location: Appling County Public Library, 301 City Hall Drive, Baxley, Georgia.

Iowa Electric Light and Power Company,
Docket No. 50-331, Duane Arnold Energy Center, Linn County, Iowa

Date of application for amendment:
October 12, 1984.

Brief Description of amendment: The amendment revises the DAEC Technical Specifications to incorporate changes reflecting the elimination of the differential pressure system between the drywell and the wetwell of the DAEC Containment.

Date of issuance: September 19, 1986.

Effective date: September 19, 1986.

Amendment No.: 137.

Facility Operating License No. DPR-49. Amendment revised the Technical Specifications.

Date of initial notice in Federal Register: December 31, 1984 (49 FR 50806).

The Commission's related evaluation of the amendment is contained in a Safety Evaluation dated September 19, 1986.

No significant hazards consideration comments received: No.

Local Public Document Room
location: Cedar Rapids Public Library, 500 First Street, S.E., Cedar Rapids, Iowa 52401.

Mississippi Power & Light Company, Middle South Energy, Inc., South Mississippi Electric Power Association, Docket No. 50-416, Grand Gulf Nuclear Station, Unit 1, Claiborne County, Mississippi

Date of application for amendment:
April 14 and May 12, 1986.

Brief description of amendment: License amendment changes Technical Specifications to add transfer switch to remote shutdown system controls, identify the plant exclusion area and gaseous effluents release points for Unit 1, revise the setpoint and instrumentation actuation values for the reactor core isolation cooling steam line high flow trip based on plant specific parameters, and makes administrative changes to correct errors.

Date of issuance: September 23, 1986.

Effective date: Changes on Technical Specification Pages 3/4 3-18, 3/4 3-88, B 3/4 3-2, 5-2, and 5-6 are effective upon issuance of the amendment. Changes on Technical Specification Page 3/4 3-71 are effective when equipment necessitating the changes on that page is installed and operable.

Amendment No. 19.

Facility Operating License No. NPF-29. This amendment revised the Technical Specifications.

Date of initial notice in Federal Register: May 21, 1986 (51 FR 18685) and June 4, 1986 (51 FR 20371).

The Commission's related evaluation of the amendment is contained in a Safety Evaluation dated September 23, 1986.

No significant hazards consideration comments received: No.

Local Public Document Room
location: Hinds Junior College, McLendon Library, Raymond, Mississippi 39154.

Nebraska Public Power District, Docket No. 50-298, Cooper Nuclear Station, Nemaha County, Nebraska

Date of amendment request: June 24, 1986.

Brief description of amendment: The amendment changes the Administrative Controls section of the Technical Specifications to clarify requirements relating to procedures.

Date of issuance: September 9, 1986.

Effective date: September 9, 1986.

Amendment No.: 101.

Facility Operating License No. DPR-62. Amendment revised the Technical Specifications.

Date of initial notice in Federal Register: August 13, 1986 (51 FR 29004).

The Commission's related evaluation of the amendment is contained in a Safety Evaluation dated September 9, 1986.

No Significant hazards consideration comments received: No.

Local Public Document Room
location: Auburn Public Library, 118 15th Street, Auburn, Nebraska 68305.

Nebraska Public Power District, Docket No. 50-298, Cooper Nuclear Station, Nemaha County, Nebraska

Date of amendment request: April 28, 1985 as supplemented May 24, 1985, June 14, 1985, and July 3, 1986.

Brief description of amendment: The amendment changes the Technical Specifications in the following areas: (1) Standby Gas Treatment and Control Room Ventilation Systems, (2) Sample line isolation setpoint change (3)

Refueling interlocks (4) Typographical errors (5) Environmental Qualification deadline, and (6) Table of Contents correction.

Date of issuance: September 25, 1986.

Effective date: September 25, 1986.

Amendment No.: 102.

Facility Operating License No. DPR-62: Amendment revised the Technical Specifications.

Date of initial notice in Federal Register: July 31, 1985 (50 FR 31068).

The May 24, 1985 submittal was published as May 5, 1985. The July 3, 1986 submittal provided additional clarifying information and did not change the finding of the initial Federal Register notice.

The Commission's related evaluation of the amendment is contained in a Safety Evaluation dated September 25, 1986.

No Significant hazards consideration comments received: No.

Local Public Document Room location: Auburn Public Library, 118 15th Street, Auburn, Nebraska 68305.

Northern States Power Company, Docket Nos. 50-282 and 50-306, Prairie Island Nuclear Generating Plant, Unit Nos. 1 and 2, Goodhue County, Minnesota

Date of application for amendments: February 21, 1986.

Brief description of amendments: The proposed changes would extend the expiration date for the Unit 1 Facility Operating License, DPR-42, from June 25, 2008 to August 9, 2013, and change the expiration date for the Unit 2 Facility Operating License, DPR-60, from June 25, 2008, to October 29, 2014.

Date of issuance: September 23, 1986.

Effective date: September 23, 1986.

Amendment Nos.: 79 and 72.

Facility Operating License Nos. DPR-42 and DPR-60: Amendment revised the Technical Specifications.

Date of initial notice in Federal Register: May 21, 1986 (51 FR 18688).

The Commission's related evaluation of the amendment is contained in a Safety Evaluation dated September 23, 1986.

No significant hazards consideration comments received: No.

Local Public Document Room location: Environmental Conservation Library, Minneapolis Public Library, 300 Nicollet Mall, Minneapolis, Minnesota.

NRC Project Director: George E. Lear, Director.

Pennsylvania Power and Light Company, Docket Nos. 50-387 and 50-388, Susquehanna Steam Electric Station, Units 1 and 2, Luzerne County, Pennsylvania

Date of application for amendments: April 23, 1986, as revised on July 17, and August 29, 1986.

Brief description of amendments: These amendments revise the testing requirements for the required emergency diesel generators in accordance with Generic Letter 84-15. Additionally, plant specific Surveillance Requirements have been revised to more accurately consider unique plant systems.

Date of issuance: September 19, 1986.

Effective date: September 19, 1986.

Amendment Nos.: 60 and 30.

Facility Operating License Nos. NPF-14 and NPF-22: Amendment revised the Technical Specifications.

Date of initial notice in Federal Register: August 13, 1986 (51 FR 29008).

The Commission's related evaluation of the amendment is contained in a Safety Evaluation dated September 19, 1986.

No significant hazards consideration comments received: No.

Local Public Document Room location: Osterhout Free Library, Reference Department, 71 South Franklin Street, Wilkes-Barre, Pennsylvania 18701.

Philadelphia Electric Company, Public Service Electric and Gas Company, Delmarva Power and Light Company, and Atlantic City Electric Company, Dockets Nos. 50-277 and 50-278, Peach Bottom Atomic Power Station, Units Nos. 2 and 3, York County, Pennsylvania

Date of application for amendments: May 23, 1985, as supplemented January 31, 1986.

Brief description of amendments: The amendments change the Technical Specifications to increase the hydrogen concentration limit downstream of the off-gas recombiners to 4 percent (volume) and decreases the number of hydrogen analyzers required to be operational during power operation to one from the currently required two. In addition a revised definition for "Alteration of the Reactor Core" is approved.

Date of issuance: September 12, 1986.

Effective date: September 12, 1986.

Amendment Nos.: 121 and 125.

Facility Operating License Nos. DPR-44 and DPR-56: Amendments revised the Technical Specifications.

Date of initial notice in Federal Register: November 20, 1985 (50 FR 47868). The January 31, 1986 submittal provided additional clarifying

information. It did not change the initial determination published in the Federal Register.

The Commission's related evaluation of the amendments is contained in a Safety Evaluation dated September 12, 1986.

No significant hazards consideration comments received: No.

Local Public Document Room location: Government Publications Section, State Library of Pennsylvania, Education Building, Commonwealth and Walnut Streets, Harrisburg, Pennsylvania 17126.

Rochester Gas and Electric Corporation, Docket No. 50-244, R. E. Ginna Nuclear Power Plant, Wayne County, New York

Date of application for amendment: October 9, 1985.

Brief description of amendment: The amendment deletes the requirement from the Technical Specifications for operation of the Auxiliary Building ventilation and charcoal filter adsorber system when the fuel being moved or stored in the spent fuel storage pool had decayed at least 60 days since irradiation.

Date of issuance: September 18, 1986.

Effective date: September 18, 1986.

Amendment No.: 19.

Facility Operating License No. DPR-18: Amendment revised the Technical Specifications.

Date of initial notice in Federal Register: April 9, 1986 (51 FR 12238)

The Commission's related evaluation of the amendment is contained in a Safety Evaluation dated September 18, 1986.

No significant hazards consideration comments received: No.

Local Public Document Room location: Rochester Public Library, 115 South Avenue, Rochester, New York 14610.

NRC Project Director: George E. Lear, Director.

Tennessee Valley Authority, Docket Nos. 50-327 and 50-328, Sequoyah Nuclear Plant, Units 1 and 2, Hamilton County, Tennessee

Date of application for amendments: June 20, 1986.

Brief description of amendments: The amendments delete the maximum fuel rod weight limit of 1,766 grams of uranium from the Design Features Section of the Technical Specifications.

Date of issuance: September 15, 1986.

Effective date: September 15, 1986.

Amendment Nos.: 45 and 37.

Facility Operating License Nos. DPR-77 and DPR-79: Amendments revised the Technical Specifications.

Date of initial notice in Federal Register: August 13, 1986 (51 FR 29014).

The Commission's related evaluation of the amendments is contained in a Safety Evaluation dated September 15, 1986.

No significant hazards consideration comments received: No.

Local Public Document Room

location: Chattanooga-Hamilton County, Bicentennial Library, 1001 Broad Street, Chattanooga, Tennessee 37401.

Tennessee Valley Authority, Docket Nos. 50-327 and 50-328, Sequoyah Nuclear Plant, Units 1 and 2, Hamilton County, Tennessee

Date of application for amendments: January 25, 1984.

Brief description of amendments: The Technical Specifications (T.S.) were changed to include the reactor vessel level instrumentation system in the Accident Monitoring T.S.

Date of issuance: September 16, 1986.

Effective date: September 16, 1986.

Amendment Nos.: 46 and 38.

Facility Operating License Nos. DPR-77 and DPR-78: Amendments revised the Technical Specifications.

Date of initial notice in Federal Register: September 28, 1984 (49 FR 39410).

The Commission's related evaluation of the amendments is contained in a Safety Evaluation dated September 16, 1986.

No significant hazards consideration comments received: No.

Local Public Document Room

location: Chattanooga-Hamilton County Bicentennial Library, 1001 Broad Street, Chattanooga, Tennessee 37401.

Tennessee Valley Authority, Docket Nos. 50-327 and 50-328, Sequoyah Nuclear Plant, Units 1 and 2, Hamilton County, Tennessee

Date of application for amendments: May 23, 1986.

Brief description of amendments: The amendments extend surveillance frequencies and out of service times for the Reactor Trip System.

Date of issuance: September 17, 1986.

Effective date: September 17, 1986.

Amendment Nos.: 47 and 39.

Facility Operating License Nos. DPR-77 and DPR-78: Amendments revised the Technical Specifications.

Date of initial notice in Federal Register: July 2, 1986 (51 FR 24284).

The Commission's related evaluation of the amendments is contained in a Safety Evaluation dated September 17, 1986.

No significant hazards consideration comments received: No.

Local Public Document Room

location: Chattanooga-Hamilton County, Bicentennial Library, 1001 Broad Street, Chattanooga, Tennessee 37401.

Virginia Electric and Power Company, Docket Nos. 50-280 and 50-281, Surry Power Station, Unit Nos. 1 and 2, Surry County, Virginia

Date of application for amendments: February 7, 1986.

Brief description of amendments:

These amendments change Section 6.1.C.2 of Technical Specifications for Surry Unit Nos. 1 and 2 to specifically identify the Independent/Operational Event Review (IOER) Section of the Safety Evaluation and Control (SEC) group under the Vice President-Nuclear Operations as the organizational unit which would be responsible for providing the independent review of the activities designated. Prior to these amendments the Technical Specifications stated that the SEC group would have this responsibility.

Date of issuance: September 9, 1986.

Effective date: September 9, 1986.

Amendment Nos.: 109 and 109.

Facility Operating License Nos. DPR-32 and DPR-37: Amendments revised the Technical Specifications.

Date of initial notice in Federal Register: April 9, 1986 (51 FR 12241).

The Commission's related evaluation of the amendment is contained in a Safety Evaluation dated September 9, 1986.

No significant hazards consideration comments received: No.

Local Public Room location: Swem Library, College of William and Mary, Williamsburg, Virginia 23185.

Yankee Atomic Electric Company, Docket No. 50-029, Yankee Nuclear Power Station, Franklin County, Massachusetts

Date of application for amendment: September 30, 1985 as modified August 22, 1986, wherein part of the proposed change was deleted.

Brief description of amendment: The amendment modifies portions of the Radiological Effluent Technical Specifications to make them consistent with current NRC guidance.

Date of issuance: September 23, 1986.

Effective date: September 23, 1986.

Amendment No.: 99.

Facility Operating License No. DPR-3: Amendment revised the Technical Specifications.

Date of initial notice in Federal Register: May 7, 1986 (51 FR 16935).

The Commission's related evaluation of the amendment is contained in a Safety Evaluation dated September 23, 1986.

No significant hazards consideration comments received: No.

Local Public Document Room

location: Greenfield Community College, 1 College Drive, Greenfield, Massachusetts 01301.

NOTICE OF ISSUANCE OF AMENDMENT TO FACILITY OPERATING LICENSE AND FINAL DETERMINATION OF NO SIGNIFICANT HAZARDS CONSIDERATION AND OPPORTUNITY FOR HEARING (EXIGENT OR EMERGENCY CIRCUMSTANCES)

During the period since publication of the last bi-weekly notice, the Commission has issued the following amendments. The Commission has determined for each of these amendments that the application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment.

Because of exigent or emergency circumstances associated with the date the amendment was needed, there was not time for the Commission to publish, for public comment before issuance, its usual 30-day Notice of Consideration Determination and Opportunity for Hearing. For exigent circumstances, the Commission has either issued a Federal Register notice providing opportunity for public comment or has used local media to provide notice to the public in the area surrounding a licensee's facility of the licensee's application and of the Commission's proposed determination of no significant hazards consideration. The Commission has provided a reasonable opportunity for the public to comment, using its best efforts to make available to the public means of communication for the public to respond quickly, and in the case of telephone comments, the comments have been recorded or transcribed as appropriate and the licensee has been informed of the public comments.

In circumstances where failure to act in a timely way would have resulted, for example, in derating or shutdown of a nuclear power plant or in prevention of either resumption of operation or of increase in power output up to the plant's licensed power level, the Commission may not have had an opportunity to provide for public comment on its no significant hazards determination. In such case, the license

amendment has been issued without opportunity for comment. If there has been some time for public comment but less than 30 days, the Commission may provide an opportunity for public comment. If comments have been requested, it is so stated. In either event, the State has been consulted by telephone whenever possible.

Under its regulations, the Commission may issue and make an amendment immediately effective, notwithstanding the pendency before it of a request for a hearing from any person, in advance of the holding and completion of any required hearing, where it has determined that no significant hazards consideration is involved.

The Commission has applied the standards of 10 CFR 50.92 and has made a final determination that the amendment involves no significant hazards consideration. The basis for this determination is contained in the documents related to this action. Accordingly, the amendments have been issued and made effective as indicated.

Unless otherwise indicated, the Commission has determined that these amendments satisfy the criteria for categorical exclusion in accordance with 10 CFR 51.22. Therefore, pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared for these amendments. If the Commission has prepared an environmental assessment under the special circumstances provision in 10 CFR 51.12(b) and has made a determination based on that assessment, it is so indicated.

For further details with respect to the action see (1) the application for amendment, (2) the amendment to Facility Operating License, and (3) the Commission's related letter, Safety Evaluation and/or Environmental Assessment, as indicated. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, NW., Washington, DC, and at the local public document room for the particular facility involved.

A copy of items (2) and (3) may be obtained upon request addressed to the U.S. Nuclear Regulatory Commission, Washington, DC 20555, Attention: Director, Division of Licensing.

The Commission is also offering an opportunity for a hearing with respect to the issuance of the amendments. By November 7, 1986, the licensee may file a request for a hearing with respect to issuance of the amendment to the subject facility operating license and any person whose interest may be affected by this proceeding and who wishes to participate as a party in the

proceeding must file a written petition for leave to intervene. Requests for a hearing and petitions for leave to intervene shall be filed in accordance with the Commission's "Rules of Practice for Domestic Licensing Proceedings" in 10 CFR Part 2. If a request for a hearing or petition for leave to intervene is filed by the above date, the Commission or an Atomic Safety and Licensing Board, designated by the Commission or by the Chairman of the Atomic Safety and Licensing Board Panel, will rule on the request and/or petition and the Secretary or the designated Atomic Safety and Licensing Board will issue a notice of hearing or an appropriate order.

As required by 10 CFR § 2.714, a petition for leave to intervene shall set forth with particularity the interest of the petitioner in the proceeding and how that interest may be affected by the results of the proceeding. The petition should specifically explain the reasons why intervention should be permitted with particular reference to the following factors: (1) The nature of the petitioner's right under the Act to be made a party to the proceeding; (2) the nature and extent of the petitioner's property, financial, or other interest in the proceeding; and (3) the possible effect of any order which may be entered in the proceeding on the petitioner's interest. The petition should also identify the specific aspect(s) of the subject matter of the proceeding as to which petitioner wishes to intervene. Any person who has filed a petition for leave to intervene or who has been admitted as a party may amend the petition without requesting leave of the Board up to fifteen (15) days prior to the first prehearing conference scheduled in the proceeding, but such an amended petition must satisfy the specificity requirements described above.

Not later than fifteen (15) days prior to the first prehearing conference scheduled in the proceeding, a petitioner shall file a supplement to the petition to intervene which must include a list of the contentions which are sought to be litigated in the matter, and the bases for each contention set forth with reasonable specificity. Contentions shall be limited to matters within the scope of the amendment under consideration. A petitioner who fails to file such a supplement which satisfies these requirements with respect to at least one contention will not be permitted to participate as a party.

Those permitted to intervene become parties to the proceeding, subject to any limitations in the order granting leave to intervene, and have the opportunity to

participate fully in the conduct of the hearing, including the opportunity to present evidence and cross-examine witnesses.

Since the Commission has made a final determination that the amendment involves no significant hazards consideration, if a hearing is requested, it will not stay the effectiveness of the amendment. Any hearing held would take place while the amendment is in effect.

A request for a hearing or a petition for leave to intervene must be filed with the Secretary of the Commission, U.S. Nuclear Regulatory Commission, Washington, DC 20555, Attention: Docketing and Service Branch, or may be delivered to the Commission's Public Document Room, 1717 H Street, NW., Washington, DC, by the above date. Where petitions are filed during the last ten (10) days of the notice period, it is requested that the petitioner promptly so inform the Commission by a toll-free telephone call to Western Union at (800) 325-6000 (in Missouri (800) 342-6700). The Western Union operator should be given Datagram Identification Number 3737 and the following message addressed to (*Branch Chief*): petitioner's name and telephone number; date petition was mailed; plant name; and publication date and page number of this *Federal Register* notice. A copy of the petition should also be sent to the Executive Legal Director, U.S. Nuclear Regulatory Commission, Washington, DC 20555, and to the attorney for the licensee.

Nontimely filings of petitions for leave to intervene, amended petitions, supplemental petitions and/or requests for hearing will not be entertained absent a determination by the Commission, the presiding officer or the Atomic Safety and Licensing Board designated to rule on the petition and/or request, that the petitioner has made a substantial showing of good cause for the granting of a late petition and/or request. That determination will be based upon a balancing of the factors specified in 10 CFR 2.714(a)(1)(i)-(v) and 2.714(d).

Baltimore Gas & Electric Company,
Docket Nos. 80-317 and 80-318, Calvert
Cliffs Nuclear Power Plant, Unit Nos. 1
and 2, Calvert County, Maryland

Date of application for amendments:
September 19, 1986, supplemented
September 22, 1986.

Brief description of amendments: The amendments temporarily change Technical Specification (TS) 3/4.8.1, "A.C. Sources," to permit, for one time

only, continued at-power dual-unit operation of up to 240 hours with the swing diesel generator (No. 12) out of service. This extension of the allowed period of diesel generator inoperability has been made contingent in the Action Statements of T.S. 3/4.8.1 upon the continued operability of each unit's dedicated diesel generator, the 1000 kW portable diesel generator, and of all three offsite A.C. power supplies. The amendments shall be used only to determine and correct the cause of the carbon monoxide leakage into the No. 12 diesel generator jacket water coolant system. This extension shall expire upon completion of repairs, post-maintenance testing, and restoration to operability of the No. 12 diesel generator.

These amendments complete the Commission action initiated in our letter of September 19, 1986, "Waiver of Compliance with Technical Specification 3/4.8.1, 'A.C. Sources'" in response to the Baltimore Gas and Electric Company application of September 19, 1986.

Date of issuance: September 23, 1986.

Effective date: The license amendments are temporary and are to be used only once. These amendments became effective at 8:00 a.m. E.d.t on September 20, 1986. Upon completion of the repairs, post-maintenance testing and restoration to operability of the No. 12 diesel generator, these amendments are cancelled.

Amendment Nos.: 122 and 104.

Facility Operating License Nos. DPR-53 and DPR-69. Amendments revised the Technical Specifications.

Public comments requested as to proposed no significant hazards consideration: No.

The Commission's related evaluation is contained in a Safety Evaluation dated September 23, 1986.

No significant hazards consideration comments received: No.

Attorney for licensee: D.A. Brune, Esq., Shaw, Pittman, Potts and Trowbridge, 1800 M Street, NW., Washington, DC 20036.

Local Public Document Room location: Calvert County Library, Prince Frederick, Maryland.

Dated at Bethesda, Maryland this 2nd day of October 1986.

For the Nuclear Regulatory Commission.

R. Wayne Houston,

Acting Director, Division of BWR Licensing.
[FR Doc. 86-22705 Filed 10-7-86; 8:45 am]

BILLING CODE 7590-01-M

June 1, 1987

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DOCKET NO(S). 50-269, 50-270, and 50-287

U.S. Environmental Protection Agency
Region IV Office
AFIN: EIS Coordinator
345 Courtland Street
Atlanta, Georgia 30365

SUBJECT: Oconee Nuclear Station, Units 1, 2, and 3
(Duke Power Company)

The following documents concerning our review of the subject facility are transmitted for your information.

- Notice of Receipt of Application, dated _____.
- Draft/Final Environmental Statement, dated _____.
- Notice of Availability of Draft/Final Environmental Statement, dated _____.
- Safety Evaluation Report, or Supplement No. _____ dated _____.
- Environmental Assessment and Finding of No Significant Impact, dated _____.
- Notice of Consideration of Issuance of Facility Operating License or Amendment to Facility Operating License, dated _____.
- Bi-Weekly Notice; Applications and Amendments to Operating Licenses Involving No Significant Hazards Considerations, dated _____ [see page(s)] _____.
- Exemption, dated _____.
- Construction Permit No. CPPR-_____, Amendment No. _____ dated _____.
- Facility Operating License No. _____, Amendment No. _____ dated _____.
- Order Extending Construction Completion Date, dated _____.
- Monthly Operating Report for April 1987 transmitted by letter dated 5/15/87.
- Annual/Semi-Annual Report- _____
_____ transmitted by letter dated _____.

Office of Nuclear Reactor Regulation

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January 4, 1988

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DOCKET NO(S): 50-269/270/287
U. S. Environmental Protection Agency
Region IV Office
ATTN: EIS Coordinator
345 Courtland Street
Atlanta, Georgia 30365

SUBJECT: Oconee Nuclear Station, Units 1,,2, and 3 - Duke Power Company

The following documents concerning our review of the subject facility are transmitted for your information.

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- Bi-Weekly Notice; Applications and Amendments to Operating Licenses Involving No Significant Hazards Considerations, dated _____ [see page(s)] _____.
- Exemption, dated _____.
- Construction Permit No. CPPR- _____, Amendment No. _____ dated _____.
- Facility Operating License No. _____, Amendment No. _____ dated _____.
- Order Extending Construction Completion Date, dated _____.
- Monthly Operating Report for November 1987 transmitted by letter dated 12/15/87.
- Annual/Semi-Annual Report- _____
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Office of Nuclear Reactor Regulation

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February 2, 1988

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DOCKET NO(S). 50-269
50-270
50-287

SUBJECT: Oconee Nuclear Station, Units 1, 2, and 3 (Duke Power Company)

The following documents concerning our review of the subject facility are transmitted for your information.

- Notice of Receipt of Application, dated _____.
- Draft/Final Environmental Statement, dated _____.
- Notice of Availability of Draft/Final Environmental Statement, dated _____.
- Safety Evaluation Report, or Supplement No. _____ dated _____.
- Environmental Assessment and Finding of No Significant Impact, dated _____.
- Notice of Consideration of Issuance of Facility Operating License or Amendment to Facility Operating License, dated _____.
- Bi-Weekly Notice; Applications and Amendments to Operating Licenses Involving No Significant Hazards Considerations, dated _____ [see page(s)] _____.
- Exemption, dated _____.
- Construction Permit No. CPPR- _____, Amendment No. _____ dated _____.
- Facility Operating License No. _____, Amendment No. _____ dated _____.
- Order Extending Construction Completion Date, dated _____.
- Monthly Operating Report for December 1987 transmitted by letter dated 01/15/88.
- Annual/Semi-Annual Report- _____
_____ transmitted by letter dated _____.

Office of Nuclear Reactor Regulation

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March 28, 1988

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DOCKET NO(S). 50-269
50-270
50-287

SUBJECT: OCONEE NUCLEAR STATION, UNITS 1, 2, AND 3 (DUKE POWER COMPANY)

The following documents concerning our review of the subject facility are transmitted for your information.

- Notice of Receipt of Application, dated _____.
- Draft/Final Environmental Statement, dated _____.
- Notice of Availability of Draft/Final Environmental Statement, dated _____.
- Safety Evaluation Report, or Supplement No. _____ dated _____.
- Environmental Assessment and Finding of No Significant Impact, dated _____.
- Notice of Consideration of Issuance of Facility Operating License or Amendment to Facility Operating License, dated _____.
- Bi-Weekly Notice; Applications and Amendments to Operating Licenses Involving No Significant Hazards Considerations, dated _____ [see page(s)] _____.
- Exemption, dated _____.
- Construction Permit No. CPPR- _____, Amendment No. _____ dated _____.
- Facility Operating License No. _____, Amendment No. _____ dated _____.
- Order Extending Construction Completion Date, dated _____.
- Monthly Operating Report for _____ transmitted by letter dated _____.
- Annual/Semi-Annual Report- Effluent Release Report for July-December 1987
transmitted by letter dated 3/1/88.

Office of Nuclear Reactor Regulation

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OFFICE	PD II-3					
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DATE	03/28/88					

Chief
Division of Ecological Services
Bureau of Sport Fisheries & Wildlife
U.S. Department of the Interior
Washington, D.C. 20240

Chief (NOAA/BF/ECD/H6814)
Ecology and Conservation Division
National Oceanic & Atmospheric Administration
U.S. Department of Commerce, Rm. 6814
14th and Constitution Avenue, N.W.
Washington, D.C. 20230

Dr. William Cunningham
FDA Research Chemist
National Bureau of Standards
Reactor Building 235, Room B108
Gaithersburg, Maryland 20899

U.S. Environmental Protection Agency
Region IV Office
ATTN: EIS Coordinator
345 Courtland Street, N.E.
Atlanta, Georgia 30365

February 25, 1988

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DOCKET NO(S). 50-269
50-270
50-287

SUBJECT: Oconee Nuclear Station, Units 1, 2, and 3 (Duke Power Company)

The following documents concerning our review of the subject facility are transmitted for your information.

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- Bi-Weekly Notice; Applications and Amendments to Operating Licenses Involving No Significant Hazards Considerations, dated _____ [see page(s)] _____.
- Exemption, dated _____.
- Construction Permit No. CPPR- _____, Amendment No. _____ dated _____.
- Facility Operating License No. _____, Amendment No. _____ dated _____.
- Order Extending Construction Completion Date, dated _____.
- Monthly Operating Report for January 1988 transmitted by letter dated 02/15/88.
- Annual/Semi-Annual Report- _____
_____ transmitted by letter dated _____.

Office of Nuclear Reactor Regulation

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March 28, 1988

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DOCKET NO(S). 50-269
50-270
50-287

SUBJECT: OCONEE NUCLEAR STATION, UNITS 1, 2, AND 3 (DUKE POWER COMPANY)

The following documents concerning our review of the subject facility are transmitted for your information.

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- Draft/Final Environmental Statement, dated _____.
- Notice of Availability of Draft/Final Environmental Statement, dated _____.
- Safety Evaluation Report, or Supplement No. _____ dated _____.
- Environmental Assessment and Finding of No Significant Impact, dated _____.
- Notice of Consideration of Issuance of Facility Operating License or Amendment to Facility Operating License, dated _____.
- Bi-Weekly Notice; Applications and Amendments to Operating Licenses Involving No Significant Hazards Considerations, dated _____ [see page(s)] _____.
- Exemption, dated _____.
- Construction Permit No. CPPR- _____, Amendment No. _____ dated _____.
- Facility Operating License No. _____, Amendment No. _____ dated _____.
- Order Extending Construction Completion Date, dated _____.
- Monthly Operating Report for February 1988 transmitted by letter dated 3/15/88.
- Annual/Semi-Annual Report- _____
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Office of Nuclear Reactor Regulation

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Chief
Division of Ecological Services
Bureau of Sport Fisheries & Wildlife
U.S. Department of the Interior
Washington, D.C. 20240

Chief (NOAA/BF/ECD/H6814)
Ecology and Conservation Division
National Oceanic & Atmospheric Administration
U.S. Department of Commerce, Rm. 6814
14th and Constitution Avenue, N.W.
Washington, D.C. 20230

Dr. William Cunningham
FDA Research Chemist
National Bureau of Standards
Reactor Building 235, Room B108
Gaithersburg, Maryland 20899

U.S. Environmental Protection Agency
Region IV Office
ATTN: EIS Coordinator
345 Courtland Street, N.E.
Atlanta, Georgia 30365

April 20, 1988
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DOCKET NO(S). 50-269
 50-270
 50-287

Mr. H.B. Tucker, Vice President
 Nuclear Production Department
 Duke Power Company
 422 South Church Street
 Charlotte, North Carolina 28242

SUBJECT: OCONEE NUCLEAR STATION, UNITS 1, 2, AND 3

The following documents concerning our review of the subject facility are transmitted for your information.

- Notice of Receipt of Application, dated _____.
- Draft/Final Environmental Statement, dated _____.
- Notice of Availability of Draft/Final Environmental Statement, dated _____.
- Safety Evaluation Report, or Supplement No. _____ dated _____.
- Environmental Assessment and Finding of No Significant Impact, dated _____.
- Notice of Consideration of Issuance of Facility Operating License or Amendment to Facility Operating License, dated _____.
- Bi-Weekly Notice; Applications and Amendments to Operating Licenses Involving No Significant Hazards Considerations, dated 4/6/88 [see page(s)] 11369 and 11370.
- Exemption, dated _____.
- Construction Permit No. CPPR-_____, Amendment No. _____ dated _____.
- Facility Operating License No. _____, Amendment No. _____ dated _____.
- Order Extending Construction Completion Date, dated _____.
- Monthly Operating Report for _____ transmitted by letter dated _____.
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Office of Nuclear Reactor Regulation

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April 26, 1988.

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DOCKET NO(S). 50-269
50-270
50-287

SUBJECT: OCONEE NUCLEAR STATION, UNITS 1, 2, AND 3 (DUKE POWER COMPANY)

The following documents concerning our review of the subject facility are transmitted for your information.

- Notice of Receipt of Application, dated _____.
- Draft/Final Environmental Statement, dated _____.
- Notice of Availability of Draft/Final Environmental Statement, dated _____.
- Safety Evaluation Report, or Supplement No. _____ dated _____.
- Environmental Assessment and Finding of No Significant Impact, dated _____.
- Notice of Consideration of Issuance of Facility Operating License or Amendment to Facility Operating License, dated _____.
- Bi-Weekly Notice; Applications and Amendments to Operating Licenses Involving No Significant Hazards Considerations, dated _____ [see page(s)] _____.
- Exemption, dated _____.
- Construction Permit No. CPPR- _____, Amendment No. _____ dated _____.
- Facility Operating License No. _____, Amendment No. _____ dated _____.
- Order Extending Construction Completion Date, dated _____.
- Monthly Operating Report for March 1988 transmitted by letter dated 04/15/88.
- Annual/Semi-Annual Report- _____
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DATE	04/26/88					

Chief
Division of Ecological Services
Bureau of Sport Fisheries & Wildlife
U.S. Department of the Interior
Washington, D.C. 20240

Chief (NOAA/BF/ECD/H6814)
Ecology and Conservation Division
National Oceanic & Atmospheric Administration
U.S. Department of Commerce, Rm. 6814
14th and Constitution Avenue, N.W.
Washington, D.C. 20230

Dr. William Cunningham
FDA Research Chemist
National Bureau of Standards
Reactor Building 235, Room B108
Gaithersburg, Maryland 20899

U.S. Environmental Protection Agency
Region IV Office
ATTN: EIS Coordinator
345 Courtland Street, N.E.
Atlanta, Georgia 30365

May 16, 1988

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DOCKET NO(S). 50-269
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SUBJECT: OCONEE NUCLEAR STATION, UNITS 1, 2, and 3 (DUKE POWER COMPANY)

The following documents concerning our review of the subject facility are transmitted for your information.

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- Environmental Assessment and Finding of No Significant Impact, dated _____.
- Notice of Consideration of Issuance of Facility Operating License or Amendment to Facility Operating License, dated _____.
- Bi-Weekly Notice; Applications and Amendments to Operating Licenses Involving No Significant Hazards Considerations, dated _____ [see page(s)] _____.
- Exemption, dated _____.
- Construction Permit No. CPPR-_____, Amendment No. _____ dated _____.
- Facility Operating License No. _____, Amendment No. _____ dated _____.
- Order Extending Construction Completion Date, dated _____.
- Monthly Operating Report for _____ transmitted by letter dated _____.
- Annual/Semi-Annual Report- ~~Radiological Environmental Operating Report~~ for 1987 transmitted by letter dated 4/29/88.

Office of Nuclear Reactor Regulation

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OFFICE	PD II-3						
SURNAME	MRood						
DATE	05/16/88						

Chief
Division of Ecological Services
Bureau of Sport Fisheries & Wildlife
U.S. Department of the Interior
Washington, D.C. 20240

Chief (NOAA/BF/ECD/H6814)
Ecology and Conservation Division
National Oceanic & Atmospheric Administration
U.S. Department of Commerce, Rm. 6814
14th and Constitution Avenue, N.W.
Washington, D.C. 20230

Dr. William Cunningham
FDA Research Chemist
National Bureau of Standards
Reactor Building 235, Room B108
Gaithersburg, Maryland 20899

U.S. Environmental Protection Agency
Region IV Office
ATTN: EIS Coordinator
345 Courtland Street, N.E.
Atlanta, Georgia 30365

June 10, 1988

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DOCKET NO(S). 50-269
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50-287

SUBJECT: OCONEE NUCLEAR STATION, UNITS 1, 2, AND 3 (DUKE POWER COMPANY)

The following documents concerning our review of the subject facility are transmitted for your information.

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- Notice of Availability of Draft/Final Environmental Statement, dated _____.
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- Notice of Consideration of Issuance of Facility Operating License or Amendment to Facility Operating License, dated _____.
- Bi-Weekly Notice; Applications and Amendments to Operating Licenses Involving No Significant Hazards Considerations, dated _____ [see page(s)] _____.
- Exemption, dated _____.
- Construction Permit No. CPPR- _____, Amendment No. _____ dated _____.
- Facility Operating License No. _____, Amendment No. _____ dated _____.
- Order Extending Construction Completion Date, dated _____.
- Monthly Operating Report for April 1988 transmitted by letter dated 5/13/88.
- Annual/Semi-Annual Report- _____
_____ transmitted by letter dated _____.

Office of Nuclear Reactor Regulation

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OFFICE	PDII-3					
SURNAME	MRood					
DATE	06/10/88					

Chief
Division of Ecological Services
Bureau of Sport Fisheries & Wildlife
U.S. Department of the Interior
Washington, D.C. 20240

Chief (NOAA/BF/ECD/H6814)
Ecology and Conservation Division
National Oceanic & Atmospheric Administration
U.S. Department of Commerce, Rm. 6814
14th and Constitution Avenue, N.W.
Washington, D.C. 20230

Dr. William Cunningham
FDA Research Chemist
National Bureau of Standards
Reactor Building 235, Room B108
Gaithersburg, Maryland 20899

U.S. Environmental Protection Agency
Region IV Office
ATTN: EIS Coordinator
345 Courtland Street, N.E.
Atlanta, Georgia 30365

June 24, 1988

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DOCKET NO(S). 50-269
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SUBJECT: OCONEE NUCLEAR STATION, UNITS 1, 2, AND 3 (DUKE POWER COMPANY)

The following documents concerning our review of the subject facility are transmitted for your information.

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- Construction Permit No. CPPR- _____, Amendment No. _____ dated _____.
- Facility Operating License No. _____, Amendment No. _____ dated _____.
- Order Extending Construction Completion Date, dated _____.
- Monthly Operating Report for May 1988 transmitted by letter dated 6/5/88.
- Annual/Semi-Annual Report- _____
_____ transmitted by letter dated _____.

Office of Nuclear Reactor Regulation

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OFFICE	PDII-3					
SURNAME	MRood					
DATE	06/24/88					

Chief
Division of Ecological Services
Bureau of Sport Fisheries & Wildlife
U.S. Department of the Interior
Washington, D.C. 20240

Chief (NOAA/BF/ECD/H6814)
Ecology and Conservation Division
National Oceanic & Atmospheric Administration
U.S. Department of Commerce, Rm. 6814
14th and Constitution Avenue, N.W.
Washington, D.C. 20230

Dr. William Cunningham
FDA Research Chemist
National Bureau of Standards
Reactor Building 235, Room B108
Gaithersburg, Maryland 20899

U.S. Environmental Protection Agency
Region IV Office
ATTN: EIS Coordinator
345 Courtland Street, N.E.
Atlanta, Georgia 30365

August 2, 1988

DISTRIBUTION
Docket File
PDII-3 r/f
OCONEE PLANT FILE
MRood
HPastis

DOCKET NO(S). 50-269
50-270
50-287

SUBJECT: O OCONEE NUCLEAR STATION, UNITS 1, 2, AND 3 (DUKE POWER COMPANY)

The following documents concerning our review of the subject facility are transmitted for your information.

- Notice of Receipt of Application, dated _____.
- Draft/Final Environmental Statement, dated _____.
- Notice of Availability of Draft/Final Environmental Statement, dated _____.
- Safety Evaluation Report, or Supplement No. _____ dated _____.
- Environmental Assessment and Finding of No Significant Impact, dated _____.
- Notice of Consideration of Issuance of Facility Operating License or Amendment to Facility Operating License, dated _____.
- Bi-Weekly Notice; Applications and Amendments to Operating Licenses Involving No Significant Hazards Considerations, dated _____ [see page(s)] _____.
- Exemption, dated _____.
- Construction Permit No. CPPR- _____, Amendment No. _____ dated _____.
- Facility Operating License No. _____, Amendment No. _____ dated _____.
- Order Extending Construction Completion Date, dated _____.
- Monthly Operating Report for June 1988 transmitted by letter dated 7/15/88.
- Annual/Semi-Annual Report- _____
_____ transmitted by letter dated _____.

Office of Nuclear Reactor Regulation

Enclosures:
As stated

cc: See next page

OFFICE	PDII-3						
SURNAME	MRood						
DATE	08/2/88						

Chief
Division of Ecological Services
Bureau of Sport Fisheries & Wildlife
U.S. Department of the Interior
Washington, D.C. 20240

Chief (NOAA/BF/ECD/H6814)
Ecology and Conservation Division
National Oceanic & Atmospheric Administration
U.S. Department of Commerce, Rm. 6814
14th and Constitution Avenue, N.W.
Washington, D.C. 20230

Dr. William Cunningham
FDA Research Chemist
National Bureau of Standards
Reactor Building 235, Room B108
Gaithersburg, Maryland 20899

U.S. Environmental Protection Agency
Region IV Office
ATTN: EIS Coordinator
345 Courtland Street, N.E.
Atlanta, Georgia 30365

August 31, 1988

DISTRIBUTION
Docket File
PDII-3 Reading File
OCONEE FILE
MRood
HPastis

DOCKET NO(S). 50-269
50-270
50-287

SUBJECT: OCONEE NUCLEAR STATION, UNITS 1, 2, AND 3 (DUKE POWER COMPANY)

The following documents concerning our review of the subject facility are transmitted for your information.

- Notice of Receipt of Application, dated _____.
- Draft/Final Environmental Statement, dated _____.
- Notice of Availability of Draft/Final Environmental Statement, dated _____.
- Safety Evaluation Report, or Supplement No. _____ dated _____.
- Environmental Assessment and Finding of No Significant Impact, dated _____.
- Notice of Consideration of Issuance of Facility Operating License or Amendment to Facility Operating License, dated _____.
- Bi-Weekly Notice; Applications and Amendments to Operating Licenses Involving No Significant Hazards Considerations, dated _____ [see page(s)] _____.
- Exemption, dated _____.
- Construction Permit No. CPPR-_____, Amendment No. _____ dated _____.
- Facility Operating License No. _____, Amendment No. _____ dated _____.
- Order Extending Construction Completion Date, dated _____.
- Monthly Operating Report for July 1988 transmitted by letter dated 9/15/88.
- Annual/Semi-Annual Report- _____
_____ transmitted by letter dated _____.

Office of Nuclear Reactor Regulation

Enclosures:
As stated

cc: See next page

OFFICE ▶	PDII-3						
SURNAME ▶	MRood						
DATE ▶	08/31/88						

Chief
Division of Ecological Services
Bureau of Sport Fisheries & Wildlife
U.S. Department of the Interior
Washington, D.C. 20240

Chief (NOAA/BF/ECD/H6814)
Ecology and Conservation Division
National Oceanic & Atmospheric Administration
U.S. Department of Commerce, Rm. 6814
14th and Constitution Avenue, N.W.
Washington, D.C. 20230

Dr. William Cunningham
FDA Research Chemist
National Bureau of Standards
Reactor Building 235, Room B108
Gaithersburg, Maryland 20899

U.S. Environmental Protection Agency
Region IV Office
ATTN: EIS Coordinator
345 Courtland Street, N.E.
Atlanta, Georgia 30365

September 16, 1983

DISTRIBUTION
Docket File
PDII-3 Reading File
OCONEE PLANT FILE
MRood
HPastis

DOCKET NO(S). 50-269
50-270
50-287

SUBJECT: OCONEE NUCLEAR STATION, UNITS 1, 2, AND 3 (DUKE POWER COMPANY)

The following documents concerning our review of the subject facility are transmitted for your information.

- Notice of Receipt of Application, dated _____.
- Draft/Final Environmental Statement, dated _____.
- Notice of Availability of Draft/Final Environmental Statement, dated _____.
- Safety Evaluation Report, or Supplement No. _____ dated _____.
- Environmental Assessment and Finding of No Significant Impact, dated _____.
- Notice of Consideration of Issuance of Facility Operating License or Amendment to Facility Operating License, dated _____.
- Bi-Weekly Notice; Applications and Amendments to Operating Licenses Involving No Significant Hazards Considerations, dated _____ [see page(s)] _____.
- Exemption, dated _____.
- Construction Permit No. CPPR-_____, Amendment No. _____ dated _____.
- Facility Operating License No. _____, Amendment No. _____ dated _____.
- Order Extending Construction Completion Date, dated _____.
- Monthly Operating Report for _____ transmitted by letter dated _____.
- Annual/Semi-Annual Report- Radioactive Effluent Release Report for January-June
1988 transmitted by letter dated 8/29/88.

Office of Nuclear Reactor Regulation

Enclosures:
As stated

cc: See next page

OFFICE	PDII-3					
SURNAME	MRood					
DATE	09/16/88					

Chief
Division of Ecological Services
Bureau of Sport Fisheries & Wildlife
U.S. Department of the Interior
Washington, D.C. 20240

Chief (NOAA/BF/ECD/H6814)
Ecology and Conservation Division
National Oceanic & Atmospheric Administration
U.S. Department of Commerce, Rm. 6814
14th and Constitution Avenue, N.W.
Washington, D.C. 20230

Dr. William Cunningham
FDA Research Chemist
National Bureau of Standards
Reactor Building 235, Room B108
Gaithersburg, Maryland 20899

U.S. Environmental Protection Agency
Region IV Office
ATTN: EIS Coordinator
345 Courtland Street, N.E.
Atlanta, Georgia 30365

DISTRIBUTION
Docket File
K. Jabbour
M. Rood
Catawba

September 29, 1988

DOCKET NO(S). 50-269
50-270
50-287

SUBJECT: Oconee Nuclear Station, Units 1,2, and 3 (Duke Power Company)

The following documents concerning our review of the subject facility are transmitted for your information.

- Notice of Receipt of Application, dated _____.
- Draft/Final Environmental Statement, dated _____.
- Notice of Availability of Draft/Final Environmental Statement, dated _____.
- Safety Evaluation Report, or Supplement No. _____ dated _____.
- Environmental Assessment and Finding of No Significant Impact, dated _____.
- Notice of Consideration of Issuance of Facility Operating License or Amendment to Facility Operating License, dated _____.
- Bi-Weekly Notice; Applications and Amendments to Operating Licenses Involving No Significant Hazards Considerations, dated _____ [see page(s)] _____.
- Exemption, dated _____.
- Construction Permit No. CPPR-_____, Amendment No. _____ dated _____.
- Facility Operating License No. _____, Amendment No. _____ dated _____.
- Order Extending Construction Completion Date, dated _____.
- Monthly Operating Report for August 1988 transmitted by letter dated 09/15/88.
- Annual/Semi-Annual Report- _____
_____ transmitted by letter dated _____.

Office of Nuclear Reactor Regulation

Enclosures:
As stated

cc: See next page

OFFICE	PDII-3						
SURNAME	<i>MJ</i>						
DATE	9/29/88						

DISTRIBUTION
 Docket File
 PDII-3 Reading
 OCONEE PLANT FILE
 MRood
 HPastis

October 24, 1988

DOCKET NO(S). 50-269
 50-270
 50-287

SUBJECT: OCONEE NUCLEAR STATION, UNITS 1, 2, AND 3 (DUKE POWER COMPANY)

The following documents concerning our review of the subject facility are transmitted for your information.

- Notice of Receipt of Application, dated _____.
- Draft/Final Environmental Statement, dated _____.
- Notice of Availability of Draft/Final Environmental Statement, dated _____.
- Safety Evaluation Report, or Supplement No. _____ dated _____.
- Environmental Assessment and Finding of No Significant Impact, dated _____.
- Notice of Consideration of Issuance of Facility Operating License or Amendment to Facility Operating License, dated _____.
- Bi-Weekly Notice; Applications and Amendments to Operating Licenses Involving No Significant Hazards Considerations, dated _____ [see page(s)] _____.
- Exemption, dated _____.
- Construction Permit No. CPPR-_____, Amendment No. _____ dated _____.
- Facility Operating License No. _____, Amendment No. _____ dated _____.
- Order Extending Construction Completion Date, dated _____.
- Monthly Operating Report for September 1988 transmitted by letter dated 10/14/88.
- Annual/Semi-Annual Report- _____
 _____ transmitted by letter dated _____.

Office of Nuclear Reactor Regulation

Enclosures:
 As stated

cc: See next page

M. H.

OFFICE	PDII-3						
SURNAME	MRood						
DATE	10/24/88						

November 28, 1988

DISTRIBUTION
Docket File
PDII-3 Reading File
OCONEE PLANT FILE
MRood
HPastis

DOCKET NO(S). 50-269
50-270
50-287

SUBJECT: OCONEE NUCLEAR STATION, UNITS 1, 2, AND 3
(DUKE POWER COMPANY)

The following documents concerning our review of the subject facility are transmitted for your information.

- Notice of Receipt of Application, dated _____.
- Draft/Final Environmental Statement, dated _____.
- Notice of Availability of Draft/Final Environmental Statement, dated _____.
- Safety Evaluation Report, or Supplement No. _____ dated _____.
- Environmental Assessment and Finding of No Significant Impact, dated _____.
- Notice of Consideration of Issuance of Facility Operating License or Amendment to Facility Operating License, dated _____.
- Bi-Weekly Notice; Applications and Amendments to Operating Licenses Involving No Significant Hazards Considerations, dated _____ [see page(s)] _____.
- Exemption, dated _____.
- Construction Permit No. CPPR- _____, Amendment No. _____ dated _____.
- Facility Operating License No. _____, Amendment No. _____ dated _____.
- Order Extending Construction Completion Date, dated _____.
- Monthly Operating Report for October transmitted by letter dated 11/15/88.
- Annual/Semi-Annual Report- _____
_____ transmitted by letter dated _____.

Office of Nuclear Reactor Regulation

Enclosures:
As stated

cc: See next page

ACRS-1a

OFFICE	PDII-3						
SURNAME	MR. ROOD						
DATE	11/16/88						

Chief
Division of Ecological Services
Bureau of Sport Fisheries & Wildlife
U.S. Department of the Interior
Washington, D.C. 20240

Chief (NOAA/BF/ECD/H6814)
Ecology and Conservation Division
National Oceanic & Atmospheric Administration
U.S. Department of Commerce, Rm. 6814
14th and Constitution Avenue, N.W.
Washington, D.C. 20230

Dr. William Cunningham
FDA Research Chemist
National Bureau of Standards
Reactor Building 235, Room B108
Gaithersburg, Maryland 20899

U.S. Environmental Protection Agency
Region IV Office
ATTN: EIS Coordinator
345 Courtland Street, N.E.
Atlanta, Georgia 30365

DISTRIBUTION
Docket File
PDII-3 r/f
OCONEE FILE
MRood
HPastis

DOCKET NO(S). 50-269
50-270
50-287

SUBJECT: OCONEE NUCLEAR STATION, UNITS 1, 2, AND 3
(DUKE POWER COMPANY)

The following documents concerning our review of the subject facility are transmitted for your information.

- Notice of Receipt of Application, dated _____.
- Draft/Final Environmental Statement, dated _____.
- Notice of Availability of Draft/Final Environmental Statement, dated _____.
- Safety Evaluation Report, or Supplement No. _____ dated _____.
- Environmental Assessment and Finding of No Significant Impact, dated _____.
- Notice of Consideration of Issuance of Facility Operating License or Amendment to Facility Operating License, dated _____.
- Bi-Weekly Notice; Applications and Amendments to Operating Licenses Involving No Significant Hazards Considerations, dated _____ [see page(s)] _____.
- Exemption, dated _____.
- Construction Permit No. CPPR- _____, Amendment No. _____ dated _____.
- Facility Operating License No. _____, Amendment No. _____ dated _____.
- Order Extending Construction Completion Date, dated _____.
- Monthly Operating Report for November 1988 transmitted by letter dated 12/18/88.
- Annual/Semi-Annual Report- _____
_____ transmitted by letter dated _____.

Office of Nuclear Reactor Regulation

Enclosures:
As stated

cc:See next page

*ACRS-1
cc*

OFFICE	PDII-3						
SURNAME	MRood						
DATE	12/23/88						

Chief
Division of Ecological Services
Bureau of Sport Fisheries & Wildlife
U.S. Department of the Interior
Washington, D.C. 20240

Chief (NOAA/BF/ECD/H6814)
Ecology and Conservation Division
National Oceanic & Atmospheric Administration
U.S. Department of Commerce, Rm. 6814
14th and Constitution Avenue, N.W.
Washington, D.C. 20230

Dr. William Cunningham
FDA Research Chemist
National Bureau of Standards
Reactor Building 235, Room B108
Gaithersburg, Maryland 20899

U.S. Environmental Protection Agency
Region IV Office
ATTN: EIS Coordinator
345 Courtland Street, N.E.
Atlanta, Georgia 30365

January 24, 1989

DISTRIBUTION:
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MRood
HPastis
Oconee Plant File

DOCKET NO(S). 50-269
50-270
50-287

SUBJECT: OCONEE NUCLEAR STATION, UNITS 1, 2, AND 3 (DUKE POWER COMPANY)

The following documents concerning our review of the subject facility are transmitted for your information.

- Notice of Receipt of Application, dated _____.
- Draft/Final Environmental Statement, dated _____.
- Notice of Availability of Draft/Final Environmental Statement, dated _____.
- Safety Evaluation Report, or Supplement No. _____ dated _____.
- Environmental Assessment and Finding of No Significant Impact, dated _____.
- Notice of Consideration of Issuance of Facility Operating License or Amendment to Facility Operating License, dated _____.
- Bi-Weekly Notice; Applications and Amendments to Operating Licenses Involving No Significant Hazards Considerations, dated _____ [see page(s)] _____.
- Exemption, dated _____.
- Construction Permit No. CPPR- _____, Amendment No. _____ dated _____.
- Facility Operating License No. _____, Amendment No. _____ dated _____.
- Order Extending Construction Completion Date, dated _____.
- Monthly Operating Report for December 1988 transmitted by letter dated 01/13/89.
- Annual/Semi-Annual Report- _____
_____ transmitted by letter dated _____.

Office of Nuclear Reactor Regulation

Enclosures:
As stated

cc: See next page

ACRS-1

OFFICE ▶	PDII-3						
SURNAME ▶	MR MR000						
DATE ▶	01/24/89						

FORM 21

Chief
Division of Ecological Services
Bureau of Sport Fisheries & Wildlife
U.S. Department of the Interior
Washington, DC 20240

Chief (NOAA/BF/ECD/H6814)
Ecology & Conservation Div., NOAA
U.S. Dept. of Commerce, Rm. 6814
14th and Constitution Ave., NW
Washington, DC 20230

Dr. William Cunningham
FDA Research Chemist
National Bureau of Standards
Reactor Bldg. 235, Room B108
Gaithersburg, MD 20899

U. S. Environmental Protection Agency
Region IV Office
ATTN: EIS Coordinator
345 Courtland Street, NE
Atlanta, GA 30365

DISTRIBUTION:
 Docket File
 Oconee Plant File
 PDII-3 R/F
 MRood
 DHood
 DMatthews

February 28, 1989

DOCKET NO(S). 50-269, 50-270 and 50-287

SUBJECT: OCONEE NUCLEAR STATION, UNITS 1, 2, AND 3 (DUKE POWER COMPANY)

The following documents concerning our review of the subject facility are transmitted for your information.

- Notice of Receipt of Application, dated _____.
- Draft/Final Environmental Statement, dated _____.
- Notice of Availability of Draft/Final Environmental Statement, dated _____.
- Safety Evaluation Report, or Supplement No. _____ dated _____.
- Environmental Assessment and Finding of No Significant Impact, dated _____.
- Notice of Consideration of Issuance of Facility Operating License or Amendment to Facility Operating License, dated _____.
- Bi-Weekly Notice; Applications and Amendments to Operating Licenses Involving No Significant Hazards Considerations, dated _____ [see page(s)] _____.
- Exemption, dated _____.
- Construction Permit No. CPPR-_____, Amendment No. _____ dated _____.
- Facility Operating License No. _____, Amendment No. _____ dated _____.
- Order Extending Construction Completion Date, dated _____.
- Monthly Operating Report for January 1989 transmitted by letter dated 2/15/89.
- Annual/Semi-Annual Report- _____
 _____ transmitted by letter dated _____.

Office of Nuclear Reactor Regulation

Enclosures:
 As stated

cc: See next page

ACRS-1
cc

OFFICE ▶	PDII-3						
SURNAME ▶	MRood						
DATE ▶	2/28/89						

DISTRIBUTION:
 Docket File
 PDII-3 Reading
 MRood
 DHood
 Oconee Plant File

March 30, 1989

DOCKET NO(S). 50-269, 50-270, 50-287

SUBJECT: OCONEE NUCLEAR STATION, UNITS 1, 2, AND 3 (DUKE POWE COMPANY)

The following documents concerning our review of the subject facility are transmitted for your information.

- Notice of Receipt of Application, dated _____.
- Draft/Final Environmental Statement, dated _____.
- Notice of Availability of Draft/Final Environmental Statement, dated _____.
- Safety Evaluation Report, or Supplement No. _____ dated _____.
- Environmental Assessment and Finding of No Significant Impact, dated _____.
- Notice of Consideration of Issuance of Facility Operating License or Amendment to Facility Operating License, dated _____.
- Bi-Weekly Notice; Applications and Amendments to Operating Licenses Involving No Significant Hazards Considerations, dated _____ [see page(s)] _____.
- Exemption, dated _____.
- Construction Permit No. CPPR-_____, Amendment No. _____ dated _____.
- Facility Operating License No. _____, Amendment No. _____ dated _____.
- Order Extending Construction Completion Date, dated _____.
- Monthly Operating Report for February 1989 transmitted by letter dated 3/15/89.
- Annual/Semi-Annual Report- Radioactive Effluent Release Report for July-December 1988 transmitted by letter dated 3/01/89.

Office of Nuclear Reactor Regulation

Enclosures:
 As stated

cc: See next page

ACRS-1
cc

OFFICE ▶	PDII-3						
SURNAME ▶	<i>MRood</i>						
DATE ▶	03/30/89						

May 2, 1989

DOCKET NO(S). 50-269, 50-270, & 50-287

SUBJECT: OCONEE NUCLEAR STATION, UNITS 1, 2, AND 3 (DUKE POWER COMPANY)

The following documents concerning our review of the subject facility are transmitted for your information.

- Notice of Receipt of Application, dated _____.
- Draft/Final Environmental Statement, dated _____.
- Notice of Availability of Draft/Final Environmental Statement, dated _____.
- Safety Evaluation Report, or Supplement No. _____ dated _____.
- Environmental Assessment and Finding of No Significant Impact, dated _____.
- Notice of Consideration of Issuance of Facility Operating License or Amendment to Facility Operating License, dated _____.
- Bi-Weekly Notice; Applications and Amendments to Operating Licenses Involving No Significant Hazards Considerations, dated _____ [see page(s)] _____.
- Exemption, dated _____.
- Construction Permit No. CPPR-_____, Amendment No. _____ dated _____.
- Facility Operating License No. _____, Amendment No. _____ dated _____.
- Order Extending Construction Completion Date, dated _____.
- Monthly Operating Report for March 1989 transmitted by letter dated 4/14/89.
- Annual/Semi-Annual Report- _____
 _____ transmitted by letter dated _____.

Office of Nuclear Reactor Regulation

Enclosures:
 As stated

cc: See next page

ACRS-1
ca

OFFICE ▶	PDII-3						
SURNAME ▶	MRood						
DATE ▶	05/02/89						

DISTRIBUTION:
 Docket File
 PDII-3 Reading
 MRood
 DHood
 Oconee Plant File

May 10, 1989

DOCKET NO(S). 50-269, 50-270, and 50-287

SUBJECT: OCONEE NUCLEAR STATION, UNITS 1, 2, AND 3 (DUKE POWER COMPANY)

The following documents concerning our review of the subject facility are transmitted for your information.

- Notice of Receipt of Application, dated _____.
- Draft/Final Environmental Statement, dated _____.
- Notice of Availability of Draft/Final Environmental Statement, dated _____.
- Safety Evaluation Report, or Supplement No. _____ dated _____.
- Environmental Assessment and Finding of No Significant Impact, dated _____.
- Notice of Consideration of Issuance of Facility Operating License or Amendment to Facility Operating License, dated _____.
- Bi-Weekly Notice; Applications and Amendments to Operating Licenses Involving No Significant Hazards Considerations, dated _____ [see page(s)] _____.
- Exemption, dated _____.
- Construction Permit No. CPPR- _____, Amendment No. _____ dated _____.
- Facility Operating License No. _____, Amendment No. _____ dated _____.
- Order Extending Construction Completion Date, dated _____.
- Monthly Operating Report for _____ transmitted by letter dated _____.
- Annual/Semi-Annual Report- Environmental Radiological Monitoring Report for 1988 transmitted by letter dated 4/28/89.

Office of Nuclear Reactor Regulation

Enclosures:
 As stated

cc: See next page

ACRS-1
CC

OFFICE ▶	PDII-3						
SURNAME ▶	<i>MRood</i>						
DATE ▶	05/10/89						

Distribution:

Docket File
PD II-3 Reading
M. Rood
L. Wiens

May 31, 1989

DOCKET NO(S). 50-269
50-270
50-287

SEE ATTACHED LIST

SUBJECT: **OCONEE NUCLEAR STATION, UNITS 1, 2, and 3 (DUKE POWER COMPANY)**

The following documents concerning our review of the subject facility are transmitted for your information.

- Notice of Receipt of Application, dated _____.
- Draft/Final Environmental Statement, dated _____.
- Notice of Availability of Draft/Final Environmental Statement, dated _____.
- Safety Evaluation Report, or Supplement No. _____ dated _____.
- Environmental Assessment and Finding of No Significant Impact, dated _____.
- Notice of Consideration of Issuance of Facility Operating License or Amendment to Facility Operating License, dated _____.
- Bi-Weekly Notice; Applications and Amendments to Operating Licenses Involving No Significant Hazards Considerations, dated _____ [see page(s)] _____.
- Exemption, dated _____.
- Construction Permit No. CPPR-_____, Amendment No. _____ dated _____.
- Facility Operating License No. _____, Amendment No. _____ dated _____.
- Order Extending Construction Completion Date, dated _____.
- Monthly Operating Report for April 1989 transmitted by letter dated 5/15/89.
- Annual/Semi-Annual Report- Appendices C and D to Annual Radiological Environmental Report for 1988 transmitted by letter dated 5/15/89.

Office of Nuclear Reactor Regulation

Enclosures:
As stated

cc: See attached list

ACRS-1
cc

OFFICE	PD II-3						
SURNAME	M. Rood:sa						
DATE	6/2/89						

Distribution:

Docket File
PD II-3 Reading
M. Rood
L. Wiens

July 19, 1989

DOCKET NO(S). 50-269
50-270
50-287

SEE ATTACHED LIST

SUBJECT: OCONEE NUCLEAR STATION, UNITS 1, 2, and 3 (DUKE POWER COMPANY)

The following documents concerning our review of the subject facility are transmitted for your information.

- Notice of Receipt of Application, dated _____.
- Draft/Final Environmental Statement, dated _____.
- Notice of Availability of Draft/Final Environmental Statement, dated _____.
- Safety Evaluation Report, or Supplement No. _____ dated _____.
- Environmental Assessment and Finding of No Significant Impact, dated _____.
- Notice of Consideration of Issuance of Facility Operating License or Amendment to Facility Operating License, dated _____.
- Bi-Weekly Notice; Applications and Amendments to Operating Licenses Involving No Significant Hazards Considerations, dated _____ [see page(s)] _____.
- Exemption, dated _____.
- Construction Permit No. CPPR- _____, Amendment No. _____ dated _____.
- Facility Operating License No. _____, Amendment No. _____ dated _____.
- Order Extending Construction Completion Date, dated _____.
- Monthly Operating Report for May 1989 transmitted by letter dated 6/15/89.
- Annual/Semi-Annual Report- _____
_____ transmitted by letter dated _____.

Office of Nuclear Reactor Regulation

Enclosures:
As stated

CC: See attached list

ACRS-1
cc

OFFICE	PD II-3	PD II-3					
SURNAME	M. Rood:sa	L. Wiens					
DATE	7/11/89	/89					

Chief
Division of Ecological Services
Bureau of Sport Fisheries & Wildlife
U.S. Department of the Interior
Washington, DC 20240

Dr. William Cunningham
FDA Research Chemist
National Bureau of Standards
Reactor Bldg. 235, Room B108
Gaithersburg, MD 20899

U.S. Environmental Protection Agency
Region IV Office
ATTN: EIS Coordinator
345 Courtland Street, NE
Atlanta, GA 30365

Distribution:

August 8, 1989

Docket File
PD II-3 Reading File
M. Rood
L. Wiens

DOCKET NO(S). 50-269
50-270
50-287

SEE ATTACHED LIST

SUBJECT: OCONEE NUCLEAR STATION, UNITS 1, 2, AND 3 (DUKE POWER COMPANY)

The following documents concerning our review of the subject facility are transmitted for your information.

	DESCRIPTION OF DOCUMENT	DATED
	Notice of Receipt of Application	
	Draft/Final Environmental Statement	
	Notice of Availability of Draft/Final Environmental Statement	
	Safety Evaluation Report, or Supplement No. _____	
	Environmental Assessment and Finding of No Significant Impact	
	Notice of Issuance of Environmental Assessment	
	Notice of Consideration of Issuance of Facility Operating License or Amendment to Facility Operating License	
	Biweekly Notice; Applications and Amendments to Operating Licenses Involving No Significant Hazards Conditions See Page(s) _____	
	Exemption	
	Construction Permit No. CPPR- _____, Amendment No. _____	
	Facility Operating License No. _____, Amendment No. _____	
	Order	
X	Monthly Operating Report for <u>June 1989</u> transmitted by Letter	07/14/89
	Annual/Semi-Annual Report: _____ transmitted by Letter	
	Other _____	

Office of Nuclear Reactor Regulation

Enclosures:
As Stated

cc: See attached list

OFFICE ▶	PD II-3	PD II-3				
SURNAME ▶	MRood:sa	LWiens				
DATE ▶	8/8/89	1/ /89				

NRC FORM 318 (10/80) NRCM 0240

OFFICIAL RECORD COPY

ACRS-1

Distribution:
 Docket file
 PD II-3 Reading
 M. Rood
 L. Wiens

August 29, 1989

DOCKET NO(S). 50-269
 50-270
 50-287

SEE ATTACHED LIST

SUBJECT: OCONEE NUCLEAR STATION, UNITS 1, 2, AND 3 (DUKE POWER COMPANY)

The following documents concerning our review of the subject facility are transmitted for your information.

✓	DESCRIPTION OF DOCUMENT	DATED
	Notice of Receipt of Application	
	Draft/Final Environmental Statement	
	Notice of Availability of Draft/Final Environmental Statement	
	Safety Evaluation Report, or Supplement No. _____	
	Environmental Assessment and Finding of No Significant Impact	
	Notice of Issuance of Environmental Assessment	
	Notice of Consideration of Issuance of Facility Operating License or Amendment to Facility Operating License	
	Biweekly Notice; Applications and Amendments to Operating Licenses Involving No Significant Hazards Conditions See Page(s) _____	
	Exemption	
	Construction Permit No. CPPR- _____, Amendment No. _____	
	Facility Operating License No. _____, Amendment No. _____	
	Order	
X	Monthly Operating Report for <u>July 1989</u> transmitted by Letter	8/15/89
	Annual/Semi-Annual Report: _____ transmitted by Letter	
	Other _____	

Office of Nuclear Reactor Regulation

Enclosures:
 As Stated

cc: See next page

AERS-1
Ca

OFFICE▶	PD II-3						
SURNAME▶	MRood:sa						
DATE▶	8/29/89						

September 27, 1989

DISTRIBUTION:
 Docket File
 PDII-3 r/f
 Ringram
 Lwiens

DOCKET NO(S). 50-269, 50-270,
 and 50-287

SEE ATTACHED LIST

SUBJECT: OCONEE NUCLEAR STATION, UNITS 1, 2 AND 3

The following documents concerning our review of the subject facility are transmitted for your information.

	DESCRIPTION OF DOCUMENT	DATED
✓	Notice of Receipt of Application	
	Draft/Final Environmental Statement	
	Notice of Availability of Draft/Final Environmental Statement	
	Safety Evaluation Report, or Supplement No. _____	
	Environmental Assessment and Finding of No Significant Impact	
	Notice of Issuance of Environmental Assessment	
	Notice of Consideration of Issuance of Facility Operating License or Amendment to Facility Operating License	
	Biweekly Notice; Applications and Amendments to Operating Licenses Involving No Significant Hazards Conditions See Page(s) _____	
	Exemption	
	Construction Permit No. CPPR- _____, Amendment No. _____	
	Facility Operating License No. _____, Amendment No. _____	
	Order	
	Monthly Operating Report for _____ transmitted by Letter	
X	Annual/Semi-Annual Report: Radioactive Effluent Release Rpt. for January- June 1989 _____ transmitted by Letter	08/30/89
	Other _____	

Office of Nuclear Reactor Regulation

Enclosures:
 As Stated

cc: See next page

ACRS-1
cc

OFFICE▶	PDII-3						
SURNAME▶	Ringram:cb						
DATE▶	09/24/89						

Chief
Division of Ecological Services
Bureau of Sport Fisheries & Wildlife
U.S. Department of the Interior
Washington, DC 20240

Dr. William Cunningham
FDA Research Chemist
National Bureau of Standards
Reactor Bldg. 235, Room B108
Gaithersburg, MD 20899

U.S. Environmental Protection Agency
Region IV Office
ATTN: EIS Coordinator
345 Courtland Street, NE
Atlanta, GA 30365

Distribution:

November 1, 1989

Docket File
PDII-3 R/F
R. Ingram
L. Wiens

DOCKET NO(S). 50-269, 50-270
and 50-287

SEE ATTACHED LIST

SUBJECT: OCONEE NUCLEAR STATION, UNITS 1, 2 AND 3 (DUKE POWER COMPANY)

The following documents concerning our review of the subject facility are transmitted for your information.

✓	DESCRIPTION OF DOCUMENT	DATED
	Notice of Receipt of Application	
	Draft/Final Environmental Statement	
	Notice of Availability of Draft/Final Environmental Statement	
	Safety Evaluation Report, or Supplement No. _____	
	Environmental Assessment and Finding of No Significant Impact	
	Notice of Issuance of Environmental Assessment	
	Notice of Consideration of Issuance of Facility Operating License or Amendment to Facility Operating License	
	Biweekly Notice; Applications and Amendments to Operating Licenses Involving No Significant Hazards Conditions See Page(s) _____	
	Exemption	
	Construction Permit No. CPPR- _____, Amendment No. _____	
	Facility Operating License No. _____, Amendment No. _____	
	Order	
X	Monthly Operating Report for <u>August 1989</u> transmitted by Letter	09/15/89
	Annual/Semi-Annual Report: _____ transmitted by Letter	
	Other _____	

Office of Nuclear Reactor Regulation

Enclosures:
As Stated

cc:

OFFICE▶	PDII-3 <i>n</i>						<i>AURS1</i> <i>cc</i>
SURNAME▶	RIngram:sa						
DATE▶	10/30/89						

Chief
Division of Ecological Services
Bureau of Sport Fisheries & Wildlife
U.S. Department of the Interior
Washington, DC 20240

Dr. William Cunningham
FDA Research Chemist
National Bureau of Standards
Reactor Bldg. 235, Room B108
Gaithersburg, MD 20899

U.S. Environmental Protection Agency
Region IV Office
ATTN: EIS Coordinator
345 Courtland Street, NE
Atlanta, GA 30365

March 31, 1990

DISTRIBUTION
Docket File
Oconee File
PD23 r/f
LWiens
RIngram

DOCKET NO(S). 50-269
50-270
50-287

SUBJECT: OCONEE NUCLEAR STATION, UNITS 1, 2, AND 3, DUKE POWER COMPANY

The following documents concerning our review of the subject facility are transmitted for your information.

✓	DESCRIPTION OF DOCUMENT	DATED
	Notice of Receipt of Application	
	Draft/Final Environmental Statement	
	Notice of Availability of Draft/Final Environmental Statement	
	Safety Evaluation Report, or Supplement No. _____	
	Environmental Assessment and Finding of No Significant Impact	
	Notice of Issuance of Environmental Assessment	
	Notice of Consideration of Issuance of Facility Operating License or Amendment to Facility Operating License	
	Biweekly Notice; Applications and Amendments to Operating Licenses Involving No Significant Hazards Conditions See Page(s) _____	
	Exemption	
	Construction Permit No. CPPR- _____, Amendment No. _____	
	Facility Operating License No. _____, Amendment No. _____	
	Order	
X	Monthly Operating Report for October 1989 transmitted by Letter	11/15/89
	Annual/Semi-Annual Report: _____ transmitted by Letter	
	Other _____	

Office of Nuclear Reactor Regulation

Enclosures:
As Stated

cc: See next page

Accessed

OFFICE ▶	PDII-3						
SURNAME ▶	RIngram						
DATE ▶	3/31/90						

cc: Chief
Division of Ecological Services
Bureau of Sport Fisheries and Wildlife
U. S. Department of the Interior
Washington, D. C. 20240

Dr. William Cunningham
FDA Research Chemist
National Bureau of Standards
Reactor Bldg 235, Room B108
Gaithersburg, Maryland 20899

U. S. Environmental Protection Agency
Region IV Office
Regional Radiation Representative
345 Courtland Street, N. E.
Atlanta, Georgia 30365

March 31, 1990

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RIngram

DOCKET NO(S). 50-269
50-270
50-287

SUBJECT: OCONEE NUCLEAR STATION, UNITS 1, 2, AND 3, DUKE POWER COMPANY

The following documents concerning our review of the subject facility are transmitted for your information.

✓	DESCRIPTION OF DOCUMENT	DATED
	Notice of Receipt of Application	
	Draft/Final Environmental Statement	
	Notice of Availability of Draft/Final Environmental Statement	
	Safety Evaluation Report, or Supplement No. _____	
	Environmental Assessment and Finding of No Significant Impact	
	Notice of Issuance of Environmental Assessment	
	Notice of Consideration of Issuance of Facility Operating License or Amendment to Facility Operating License	
	Biweekly Notice; Applications and Amendments to Operating Licenses Involving No Significant Hazards Conditions See Page(s) _____	
	Exemption	
	Construction Permit No. CPPR- _____, Amendment No. _____	
	Facility Operating License No. _____, Amendment No. _____	
	Order	
X	Monthly Operating Report for September 1989 _____ transmitted by Letter	10/13/89
	Annual/Semi-Annual Report: _____ _____ transmitted by Letter	
	Other _____	

Office of Nuclear Reactor Regulation

Enclosures:
As Stated

cc: See next page

ACKS-1 ent

OFFICE▶	PDII-3 <i>AK</i>						
SURNAME▶	RIngram						
DATE▶	3/31/90						

cc: Chief
Division of Ecological Services
Bureau of Sport Fisheries and Wildlife
U. S. Department of the Interior
Washington, D. C. 20240

Dr. William Cunningham
FDA Research Chemist
National Bureau of Standards
Reactor Bldg 235, Room B108
Gaithersburg, Maryland 20899

U. S. Environmental Protection Agency
Region IV Office
Regional Radiation Representative
345 Courtland Street, N. E.
Atlanta, Georgia 30365

March 31, 1990

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DOCKET NO(S). 50-269
50-270
50-287

SUBJECT: OCONEE NUCLEAR STATION, UNITS 1, 2, AND 3, DUKE POWER COMPANY

The following documents concerning our review of the subject facility are transmitted for your information.

✓	DESCRIPTION OF DOCUMENT	DATED
	Notice of Receipt of Application	
	Draft/Final Environmental Statement	
	Notice of Availability of Draft/Final Environmental Statement	
	Safety Evaluation Report, or Supplement No. _____	
	Environmental Assessment and Finding of No Significant Impact	
	Notice of Issuance of Environmental Assessment	
	Notice of Consideration of Issuance of Facility Operating License or Amendment to Facility Operating License	
	Biweekly Notice; Applications and Amendments to Operating Licenses Involving No Significant Hazards Conditions See Page(s) _____	
	Exemption	
	Construction Permit No. CPPR- _____, Amendment No. _____	
	Facility Operating License No. _____, Amendment No. _____	
	Order	
	Monthly Operating Report for _____ transmitted by Letter	
	Annual/Semi-Annual Report: _____ transmitted by Letter	
X	Other <u>Add'l info. for NPDES Renewal</u> transmitted by letter	11/7/89

Office of Nuclear Reactor Regulation

Enclosures:
As Stated

cc: See next page

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OFFICE▶	PDII-3 <i>M</i>						
SURNAME▶	RIngram:sw						
DATE▶	3/3/90						

cc: Chief
Division of Ecological Services
Bureau of Sport Fisheries and Wildlife
U. S. Department of the Interior
Washington, D. C. 20240

Dr. William Cunningham
FDA Research Chemist
National Bureau of Standards
Reactor Bldg 235, Room B108
Gaithersburg, Maryland 20899

U. S. Environmental Protection Agency
Region IV Office
Regional Radiation Representative
345 Courtland Street, N. E.
Atlanta, Georgia 30365

March 31, 1990

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DOCKET NO(S). 50-269
50-270
50-287

SUBJECT: OCONEE NUCLEAR STATION, UNITS 1, 2, AND 3, DUKE POWER COMPANY

The following documents concerning our review of the subject facility are transmitted for your information.

✓	DESCRIPTION OF DOCUMENT	DATED
	Notice of Receipt of Application	
	Draft/Final Environmental Statement	
	Notice of Availability of Draft/Final Environmental Statement	
	Safety Evaluation Report, or Supplement No. _____	
	Environmental Assessment and Finding of No Significant Impact	
	Notice of Issuance of Environmental Assessment	
	Notice of Consideration of Issuance of Facility Operating License or Amendment to Facility Operating License	
	Biweekly Notice; Applications and Amendments to Operating Licenses Involving No Significant Hazards Conditions See Page(s) _____	
	Exemption	
	Construction Permit No. CPPR-- _____, Amendment No. _____	
	Facility Operating License No. _____, Amendment No. _____	
	Order	
X	Monthly Operating Report for December 1989 transmitted by Letter	1/15/90
	Annual/Semi-Annual Report: _____ transmitted by Letter	
	Other _____	

Office of Nuclear Reactor Regulation

Enclosures:
As Stated

cc: See next page

ACRS-1
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OFFICE▶	PDII-3						
SURNAME▶	RIngram:sw						
DATE▶	3/31/90						

cc: Chief
Division of Ecological Services
Bureau of Sport Fisheries and Wildlife
U. S. Department of the Interior
Washington, D. C. 20240

Dr. William Cunningham
FDA Research Chemist
National Bureau of Standards
Reactor Bldg 235, Room B108
Gaithersburg, Maryland 20899

U. S. Environmental Protection Agency
Region IV Office
Regional Radiation Representative
345 Courtland Street, N. E.
Atlanta, Georgia 30365

March 31, 1990

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DOCKET NO(S). 50-269
50-270
50-287

SUBJECT: OCONEE NUCLEAR STATION, UNITS 1, 2, AND 3, DUKE POWER COMPANY

The following documents concerning our review of the subject facility are transmitted for your information.

	DESCRIPTION OF DOCUMENT	DATED
✓	Notice of Receipt of Application	
	Draft/Final Environmental Statement	
	Notice of Availability of Draft/Final Environmental Statement	
	Safety Evaluation Report, or Supplement No. _____	
	Environmental Assessment and Finding of No Significant Impact	
	Notice of Issuance of Environmental Assessment	
	Notice of Consideration of Issuance of Facility Operating License or Amendment to Facility Operating License	
	Biweekly Notice; Applications and Amendments to Operating Licenses Involving No Significant Hazards Conditions See Page(s) _____	
	Exemption	
	Construction Permit No. CPPR- _____, Amendment No. _____	
	Facility Operating License No. _____, Amendment No. _____	
	Order	
X	Monthly Operating Report for Nov. 1989 & corrected rept for Oct. transmitted by Letter	12/15/89
	Annual/Semi-Annual Report: _____ _____ transmitted by Letter	
	Other _____	

Office of Nuclear Reactor Regulation

Enclosures:
As Stated

cc: See next page

ACRS-ent

OFFICE▶	PDII-3 ^c						
SURNAME▶	RIngram: sw						
DATE▶	2/31/90						

cc: Chief
Division of Ecological Services
Bureau of Sport Fisheries and Wildlife
U. S. Department of the Interior
Washington, D. C. 20240

Dr. William Cunningham
FDA Research Chemist
National Bureau of Standards
Reactor Bldg 235, Room B108
Gaithersburg, Maryland 20899

U. S. Environmental Protection Agency
Region IV Office
Regional Radiation Representative
345 Courtland Street, N. E.
Atlanta, Georgia 30365

February 27, 1991

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Oconee File
PD23 r/f
BClayton

DOCKET NO(S). 50-269, 59-270
and 50-287

SEE ATTACHED LIST

SUBJECT: OCONEE NUCLEAR STATION

The following documents concerning our review of the subject facility are transmitted for your information.

✓	DESCRIPTION OF DOCUMENT	DATED
	Notice of Receipt of Application	
	Draft/Final Environmental Statement	
	Notice of Availability of Draft/Final Environmental Statement	
	Safety Evaluation Report, or Supplement No. _____	
	Environmental Assessment and Finding of No Significant Impact	
	Notice of Issuance of Environmental Assessment	
	Notice of Consideration of Issuance of Facility Operating License or Amendment to Facility Operating License	
	Biweekly Notice; Applications and Amendments to Operating Licenses Involving No Significant Hazards Conditions See Page(s) _____	
	Exemption	
	Construction Permit No. CPPR- _____, Amendment No. _____	
	Facility Operating License No. _____, Amendment No. _____	
	Order	
X	Monthly Operating Report for June - December 1990 transmitted by Letter	
X	Annual/Semi-Annual Report: Jan. - Dec. Radioactive Effluent Release Rpt. transmitted by Letter	
	Other _____	

Office of Nuclear Reactor Regulation

Enclosures:
As Stated

cc:

ST
[Handwritten signature]

OFFICE ▶	PDII-3						
SURNAME ▶	BClayton:sw						
DATE ▶	2/27/91						

cc: Chief
Division of Ecological Services
Bureau of Sport Fisheries and Wildlife
U. S. Department of the Interior
Washington, D. C. 20240

Dr. William Cunningham
FDA Research Chemist
National Bureau of Standards
Reactor Bldg 235, Room B108
Gaithersburg, Maryland 20899

U. S. Environmental Protection Agency
Region IV Office
Regional Radiation Representative
345 Courtland Street, N. E.
Atlanta, Georgia 30365

March 8, 1991

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 BClayton

DOCKET NO(S). 50-269
 50-270
 50-287

SEE ATTACHED LIST

SUBJECT: OCONEE NUCLEAR STATION

The following documents concerning our review of the subject facility are transmitted for your information.

	DESCRIPTION OF DOCUMENT	DATED
	Notice of Receipt of Application	
	Draft/Final Environmental Statement	
	Notice of Availability of Draft/Final Environmental Statement	
	Safety Evaluation Report, or Supplement No. _____	
	Environmental Assessment and Finding of No Significant Impact	
	Notice of Issuance of Environmental Assessment	
	Notice of Consideration of Issuance of Facility Operating License or Amendment to Facility Operating License	
	Biweekly Notice; Applications and Amendments to Operating Licenses Involving No Significant Hazards Conditions See Page(s) _____	
	Exemption	
	Construction Permit No. CPPR- _____, Amendment No. _____	
	Facility Operating License No. _____, Amendment No. _____	
	Order	
	Monthly Operating Report for _____ transmitted by Letter	
X	Annual/Semi-Annual Report: Radioactive Release Rept. _____ _____ transmitted by Letter	2/28/91
	Other _____	

Office of Nuclear Reactor Regulation

Enclosures:
 As Stated

cc:

OFFICE ▶	PDII-3						
SURNAME ▶	BClayton: SW						
DATE ▶	3/8/91						

cc:

Chief
Division of Ecological Services
Bureau of Sport Fisheries and Wildlife
U. S. Department of the Interior
Washington, D. C. 20240

Dr. William Cunningham
FDA Research Chemist
National Bureau of Standards
Reactor Bldg 235, Room B108
Gaithersburg, Maryland 20899

U. S. Environmental Protection Agency
Region IV Office
Regional Radiation Representative
345 Courtland Street, N. E.
Atlanta, Georgia 30365

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 PD23 r/f
 BClayton

March 8, 1991

DOCKET NO(S). 50-269
 50-270
 50-287

SEE ATTACHED LIST

SUBJECT: OCONEE NUCLEAR STATION

The following documents concerning our review of the subject facility are transmitted for your information.

	DESCRIPTION OF DOCUMENT	DATED
✓	Notice of Receipt of Application	
	Draft/Final Environmental Statement	
	Notice of Availability of Draft/Final Environmental Statement	
	Safety Evaluation Report, or Supplement No. _____	
	Environmental Assessment and Finding of No Significant Impact	
	Notice of Issuance of Environmental Assessment	
	Notice of Consideration of Issuance of Facility Operating License or Amendment to Facility Operating License	
	Biweekly Notice; Applications and Amendments to Operating Licenses Involving No Significant Hazards Conditions See Page(s) _____	
	Exemption	
	Construction Permit No. CPPR- _____, Amendment No. _____	
	Facility Operating License No. _____, Amendment No. _____	
	Order	
X	Monthly Operating Report for January 1991 transmitted by Letter	2/15/91
	Annual/Semi-Annual Report: _____ transmitted by Letter	
	Other _____	

Office of Nuclear Reactor Regulation

Enclosures:
 As Stated

cc:

OFFICE▶	PDII-3						
SURNAME▶	BClayton:sw						
DATE▶	3/8/91						

cc:

Chief
Division of Ecological Services
Bureau of Sport Fisheries and Wildlife
U. S. Department of the Interior
Washington, D. C. 20240

Dr. William Cunningham
FDA Research Chemist
National Bureau of Standards
Reactor Bldg 235, Room B108
Gaithersburg, Maryland 20899

U. S. Environmental Protection Agency
Region IV Office
Regional Radiation Representative
345 Courtland Street, N. E.
Atlanta, Georgia 30365

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 PD23 r/f
 BClayton

March 28, 1991

DOCKET NO(S). 50-269
 50-270
 50-287

SEE ATTACHED LIST

SUBJECT: OCONEE NUCLEAR STATION

The following documents concerning our review of the subject facility are transmitted for your information.

✓	DESCRIPTION OF DOCUMENT	DATED
	Notice of Receipt of Application	
	Draft/Final Environmental Statement	
	Notice of Availability of Draft/Final Environmental Statement	
	Safety Evaluation Report, or Supplement No. _____	
	Environmental Assessment and Finding of No Significant Impact	
	Notice of Issuance of Environmental Assessment	
	Notice of Consideration of Issuance of Facility Operating License or Amendment to Facility Operating License	
	Biweekly Notice; Applications and Amendments to Operating Licenses Involving No Significant Hazards Conditions See Page(s) _____	
	Exemption	
	Construction Permit No. CPPR- _____, Amendment No. _____	
	Facility Operating License No. _____, Amendment No. _____	
	Order	
X	Monthly Operating Report for February 1991 transmitted by Letter	3/15/91
	Annual/Semi-Annual Report: _____ transmitted by Letter	
	Other _____	

Office of Nuclear Reactor Regulation

Enclosures:
 As Stated

cc:

ST
[Handwritten signature]

OFFICE▶	PD23						
SURNAME▶	BClayton:sw						
DATE▶	3/28/91						

cc:

Chief
Division of Ecological Services
Bureau of Sport Fisheries and Wildlife
U. S. Department of the Interior
Washington, D. C. 20240

Dr. William Cunningham
FDA Research Chemist
National Bureau of Standards
Reactor Bldg 235, Room B108
Gaithersburg, Maryland 20899

U. S. Environmental Protection Agency
Region IV Office
Regional Radiation Representative
345 Courtland Street, N. E.
Atlanta, Georgia 30365

April 26, 1991

DOCKET NO(S). 50-269
 50-270
 50-287

SEE ATTACHED LIST

SUBJECT: OCONEE NUCLEAR STATION

The following documents concerning our review of the subject facility are transmitted for your information.

	DESCRIPTION OF DOCUMENT	DATED
✓	Notice of Receipt of Application	
	Draft/Final Environmental Statement	
	Notice of Availability of Draft/Final Environmental Statement	
	Safety Evaluation Report, or Supplement No. _____	
	Environmental Assessment and Finding of No Significant Impact	
	Notice of Issuance of Environmental Assessment	
	Notice of Consideration of Issuance of Facility Operating License or Amendment to Facility Operating License	
	Biweekly Notice; Applications and Amendments to Operating Licenses Involving No Significant Hazards Conditions See Page(s) _____	
	Exemption	
	Construction Permit No. CPPR— _____, Amendment No. _____	
	Facility Operating License No. _____, Amendment No. _____	
	Order	
	Monthly Operating Report for _____ transmitted by Letter	
X	Annual/Semi-Annual Report: Radioactive Effluent Release Report _____ transmitted by Letter	2/28/91
	Other _____	

Office of Nuclear Reactor Regulation

Enclosures:
 As Stated

cc:

OFFICE	LA: PDII -3						
SURNAME	BCl: Clayton: SW						
DATE	4/26/91						

cc:

Chief
Division of Ecological Services
Bureau of Sport Fisheries and Wildlife
U. S. Department of the Interior
Washington, D. C. 20240

Dr. William Cunningham
FDA Research Chemist
National Bureau of Standards
Reactor Bldg 235, Room B108
Gaithersburg, Maryland 20899

U. S. Environmental Protection Agency
Region IV Office
Regional Radiation Representative
345 Courtland Street, N. E.
Atlanta, Georgia 30365

September 12, 1991

DOCKET NO(S). 50-269/270/287

TO THOSE ON ATTACHED LIST

SUBJECT: OCONEE NUCLEAR STATION, UNITS 1/2/3

The following documents concerning our review of the subject facility are transmitted for your information.

✓	DESCRIPTION OF DOCUMENT	DATED
	Notice of Receipt of Application	
	Draft/Final Environmental Statement	
	Notice of Availability of Draft/Final Environmental Statement	
	Safety Evaluation Report, or Supplement No. _____	
	Environmental Assessment and Finding of No Significant Impact	
	Notice of Issuance of Environmental Assessment	
	Notice of Consideration of Issuance of Facility Operating License or Amendment to Facility Operating License	
	Biweekly Notice; Applications and Amendments to Operating Licenses Involving No Significant Hazards Conditions See Page(s) _____	
	Exemption	
	Construction Permit No. CPPR- _____, Amendment No. _____	
	Facility Operating License No. _____, Amendment No. _____	
	Order	
X	Monthly Operating Report for Semiannual Radioactive Effluent Release Report transmitted by Letter	06/30/91
	Annual/Semi-Annual Report: _____ transmitted by Letter	
	Other _____	

Office of Nuclear Reactor Regulation

Enclosures:
As Stated

cc: See next page

OFFICE▶	PDI 173/LA						
SURNAME▶	LBerry						
DATE▶	9/12/91						

September 7, 1991

DOCKET NO(S). 50-269, 270 and
287

DISTRIBUTION
Docket File
NRC & L PDR
LBerry

See attached

SUBJECT:

The following documents concerning our review of the subject facility are transmitted for your information.

✓	DESCRIPTION OF DOCUMENT	DATED
	Notice of Receipt of Application	
	Draft/Final Environmental Statement	
	Notice of Availability of Draft/Final Environmental Statement	
	Safety Evaluation Report, or Supplement No. _____	
	Environmental Assessment and Finding of No Significant Impact	
	Notice of Issuance of Environmental Assessment	
	Notice of Consideration of Issuance of Facility Operating License or Amendment to Facility Operating License	
	Biweekly Notice; Applications and Amendments to Operating Licenses Involving No Significant Hazards Conditions See Page(s) _____	
	Exemption	
	Construction Permit No. CPPR- _____, Amendment No. _____	
	Facility Operating License No. _____, Amendment No. _____	
	Order	
X	Monthly Operating Report for _____ transmitted by Letter	
	Annual/Semi-Annual Report: _____ _____ transmitted by Letter	
	Other _____	

Office of Nuclear Reactor Regulation

Enclosures:
As Stated

cc: **See Next page**

OFFICE ▶	PD/16-LA						
SURNAME ▶	LBerry						
DATE ▶	9/9/91						

November 18, 1991

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Docket File ✓
PDII-3 Reading
LBerry

DOCKET NO(S). 50-269, 50-270 and 50-287

TO THOSE ON ATTACHED LIST

SUBJECT: OCOONEE NUCLEAR STATION, UNITS 1,2, AND 3

The following documents concerning our review of the subject facility are transmitted for your information.

✓	DESCRIPTION OF DOCUMENT	DATED
	Notice of Receipt of Application	
	Draft/Final Environmental Statement	
	Notice of Availability of Draft/Final Environmental Statement	
	Safety Evaluation Report, or Supplement No. _____	
	Environmental Assessment and Finding of No Significant Impact	
	Notice of Issuance of Environmental Assessment	
	Notice of Consideration of Issuance of Facility Operating License or Amendment to Facility Operating License	
	Biweekly Notice; Applications and Amendments to Operating Licenses Involving No Significant Hazards Conditions See Page(s) _____	
	Exemption	
	Construction Permit No. CPPR- _____, Amendment No. _____	
	Facility Operating License No. _____, Amendment No. _____	
	Order	
X	Monthly Operating Report for <u>FOR SEPTEMBER 1991 AND REVISED</u> transmitted by Letter	10/15/91
	Annual/Semi-Annual Report: <u>REPORTS FOR AUGUST 1991</u> _____ transmitted by Letter	
	Other _____	

Office of Nuclear Reactor Regulation
Division of Reactor Projects I/II

Enclosures:
As Stated

cc: See next page

OFFICE▶	PDII-3/LA LBerry						
SURNAME▶							
DATE▶	11/18/91						

ST
[Signature]

September 25, 1991

DOCKET NO(S). 50-269/270;287

To Those on Attached List

SUBJECT: OCONEE NUCLEAR STATION, UNITS 1/2/3

The following documents concerning our review of the subject facility are transmitted for your information.

✓	DESCRIPTION OF DOCUMENT	DATED
	Notice of Receipt of Application	
	Draft/Final Environmental Statement	
	Notice of Availability of Draft/Final Environmental Statement	
	Safety Evaluation Report, or Supplement No. _____	
	Environmental Assessment and Finding of No Significant Impact	
	Notice of Issuance of Environmental Assessment	
	Notice of Consideration of Issuance of Facility Operating License or Amendment to Facility Operating License	
	Biweekly Notice; Applications and Amendments to Operating Licenses Involving No Significant Hazards Conditions See Page(s) _____	
	Exemption	
	Construction Permit No. CPPR-- _____, Amendment No. _____	
	Facility Operating License No. _____, Amendment No. _____	
	Order	
x	Monthly Operating Report for August 1991 and Monthly Operating Status Report for July 1991 transmitted by Letter	9/13/91
	Annual/Semi-Annual Report: _____ transmitted by Letter	
	Other _____	

Office of Nuclear Reactor Regulation

Enclosures:
As Stated

cc: See next page

OFFICE▶	PDI-8/LA						
SURNAME▶	LBerry: 1b						
DATE▶	9/ 25/91						

December 18, 1991

DOCKET NO(S). 50-269/270/287

TO THOSE ON ATTACHED LIST

SUBJECT: OCOONEE NUCLEAR STATION, UNITS 1/2/3

The following documents concerning our review of the subject facility are transmitted for your information.

✓	DESCRIPTION OF DOCUMENT	DATED
	Notice of Receipt of Application	
	Draft/Final Environmental Statement	
	Notice of Availability of Draft/Final Environmental Statement	
	Safety Evaluation Report, or Supplement No. _____	
	Environmental Assessment and Finding of No Significant Impact	
	Notice of Issuance of Environmental Assessment	
	Notice of Consideration of Issuance of Facility Operating License or Amendment to Facility Operating License	
	Biweekly Notice; Applications and Amendments to Operating Licenses Involving No Significant Hazards Conditions See Page(s) _____	
	Exemption	
	Construction Permit No. CPPR- _____, Amendment No. _____	
	Facility Operating License No. _____, Amendment No. _____	
	Order	
	Monthly Operating Report for June 1991 & Revised Reports for May 1991 transmitted by Letter	07/15/91
	Monthly Operating Report for Oct 1991 & Revised Reports for Sept 1991 transmitted by Letter	11/15/91
	Annual/Semi-Annual Report: _____ transmitted by Letter	
	Other _____	

Office of Nuclear Reactor Regulation
 Division of Reactor Projects I/II

Enclosures:
 As Stated

cc: See next page

ST
[Handwritten signature]

OFFICE ▶	PD 113						
SURNAME ▶	LBerry						
DATE ▶	12/18/91						

March 5, 1992

DOCKET NO(S). 50-269/270/287

TO THOSE ON ATTACHED LIST

SUBJECT: OCONEE NUCLEAR STATION, UNITS 1/2/3

The following documents concerning our review of the subject facility are transmitted for your information.

✓	DESCRIPTION OF DOCUMENT	DATED
	Notice of Receipt of Application	
	Draft/Final Environmental Statement	
	Notice of Availability of Draft/Final Environmental Statement	
	Safety Evaluation Report, or Supplement No. _____	
	Environmental Assessment and Finding of No Significant Impact	
	Notice of Issuance of Environmental Assessment	
	Notice of Consideration of Issuance of Facility Operating License or Amendment to Facility Operating License	
	Biweekly Notice; Applications and Amendments to Operating Licenses Involving No Significant Hazards Conditions See Page(s) _____	
	Exemption	
	Construction Permit No. CPPR- _____, Amendment No. _____	
	Facility Operating License No. _____, Amendment No. _____	
x	Order Performance and Operating Status for month of January 1992	2/14/92
	Monthly Operating Report for _____ transmitted by Letter	
	Annual/Semi-Annual Report: _____ _____ transmitted by Letter	
	Other _____	

Office of Nuclear Reactor Regulation
Division of Reactor Projects I/II

Enclosures:
As Stated

cc:

OFFICE ▶	... P.D. 1.3						
SURNAME ▶	... LBerry						
DATE ▶	... 3/5/92						

March 5, 1992

DOCKET NO(S). 50-269/270/287

TO THOSE ON ATTACHED LIST

SUBJECT: OCONEE NUCLEAR STATION, UNITS 1/2/3

The following documents concerning our review of the subject facility are transmitted for your information.

✓	DESCRIPTION OF DOCUMENT	DATED
	Notice of Receipt of Application	
	Draft/Final Environmental Statement	
	Notice of Availability of Draft/Final Environmental Statement	
	Safety Evaluation Report, or Supplement No. _____	
	Environmental Assessment and Finding of No Significant Impact	
	Notice of Issuance of Environmental Assessment	
	Notice of Consideration of Issuance of Facility Operating License or Amendment to Facility Operating License	
	Biweekly Notice; Applications and Amendments to Operating Licenses Involving No Significant Hazards Conditions See Page(s) _____	
	Exemption	
	Construction Permit No. CPPR— _____, Amendment No. _____	
	Facility Operating License No. _____, Amendment No. _____	
X	Order ^{Other} Performance and Operating Status for Month of December 1991	2/15/92
	Monthly Operating Report for _____ transmitted by Letter	
	Annual/Semi-Annual Report: _____ _____ transmitted by Letter	
	Other _____	

Office of Nuclear Reactor Regulation
 Division of Reactor Projects I/II

Enclosures:
 As Stated

cc: See next page

OFFICE▶	PDYI-3 <i>LBerry</i>						
SURNAME▶	LBerry						
DATE▶	3/5/92						

March 5, 1992

DOCKET NO(S). 50-269/279/287

TO THOSE ON ATTACHED LIST

SUBJECT: OCONEE NUCLEAR STATION, UNITS 1/2/3

The following documents concerning our review of the subject facility are transmitted for your information:

✓	DESCRIPTION OF DOCUMENT	DATED
	Notice of Receipt of Application	
	Draft/Final Environmental Statement	
	Notice of Availability of Draft/Final Environmental Statement	
	Safety Evaluation Report, or Supplement No. _____	
	Environmental Assessment and Finding of No Significant Impact	
	Notice of Issuance of Environmental Assessment	
	Notice of Consideration of Issuance of Facility Operating License or Amendment to Facility Operating License	
	Biweekly Notice; Applications and Amendments to Operating Licenses Involving No Significant Hazards Conditions See Page(s) _____	
	Exemption	
	Construction Permit No. CPPR- _____, Amendment No. _____	
	Facility Operating License No. _____, Amendment No. _____	
	Order	
	Monthly Operating Report for _____ transmitted by Letter	
x	Annual/Semi-Annual Report: 1991 Radioactive Effluent Release Report for July-December _____ transmitted by Letter	12/31/92
	Other _____	

Office of Nuclear Reactor Regulation
Division of Reactor Projects I/II

Enclosures:
As Stated

cc: See next page

*ST
LJB*

OFFICE ▶	PDI 3 <i>LBerry</i>						
SURNAME ▶	LBerry						
DATE ▶	3/5/92						

March 5, 1992

DOCKET NO(S). 50-269/270/287

TO THOSE ON ATTACHED LIST

SUBJECT: OCONEE ~~NUCLEAR~~ STATION, UNITS 1/2/3

The following documents concerning our review of the subject facility are transmitted for your information.

	DESCRIPTION OF DOCUMENT	DATED
✓	Notice of Receipt of Application	
	Draft/Final Environmental Statement	
	Notice of Availability of Draft/Final Environmental Statement	
	Safety Evaluation Report, or Supplement No. _____	
	Environmental Assessment and Finding of No Significant Impact	
	Notice of Issuance of Environmental Assessment	
	Notice of Consideration of Issuance of Facility Operating License or Amendment to Facility Operating License	
	Biweekly Notice; Applications and Amendments to Operating Licenses Involving No Significant Hazards Conditions See Page(s) _____	
	Exemption	
	Construction Permit No. CPPR— _____, Amendment No. _____	
	Facility Operating License No. _____, Amendment No. _____	
x	Order ^{OTHER} Performance and Operating Station for month of November 1991	12/13/91
	Monthly Operating Report for _____ transmitted by Letter	
	Annual/Semi-Annual Report: _____	
	_____ transmitted by Letter	
	Other _____	

Office of Nuclear Reactor Regulation
division of Reactor Projects I/II

Enclosures:
As Stated

cc: See next page

OFFICE▶	POLI-3						
SURNAME▶	LBerry						
DATE▶	3/5/92						

APRIL 28, 1992

DOCKET NO(S). 50-269/270/289
50-369/370
50-413/414

TO THOSE ~~AN~~ ATTACHED LIST

SUBJECT: **XX** OCONEE NUCLEAR STATION, UNITS 1/2/3, WILLIAM B. MCGUIRE NUCLEAR STATION, UNITS 1/2, AND CATAWBA NUCLEAR STATION, UNITS 1/2

The following documents concerning our review of the subject facility are transmitted for your information.

✓	DESCRIPTION OF DOCUMENT	DATED
	Notice of Receipt of Application	
	Draft/Final Environmental Statement	
	Notice of Availability of Draft/Final Environmental Statement	
	Safety Evaluation Report, or Supplement No. _____	
	Environmental Assessment and Finding of No Significant Impact	
	Notice of Issuance of Environmental Assessment	
	Notice of Consideration of Issuance of Facility Operating License or Amendment to Facility Operating License	
	Biweekly Notice; Applications and Amendments to Operating Licenses Involving No Significant Hazards Conditions See Page(s) _____	
	Exemption	
	Construction Permit No. CPPR- _____, Amendment No. _____	
	Facility Operating License No. _____, Amendment No. _____	
	Order	
	Monthly Operating Report for _____ transmitted by Letter	
	Annual/Semi-Annual Report: _____ transmitted by Letter	
	Other NONCOMPLIANCE NOTIFICATION	3/19/92

Office of Nuclear Reactor Regulation
Division of Reactor Projects I/II

Enclosures:
As Stated

cc: See next page

OFFICE ▶	PDII-3/7LA						
SURNAME ▶	L. BERRY						
DATE ▶	4/28/92						

APRIL 28, 1992

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PDII-3 READING
OCONEE READING
L. BERRY

DOCKET NO(S). 50-269/270/287

TO THOSE ON ATTACHED LIST

SUBJECT: OCONEE NUCLEAR STATION UNITS 1/2/3

The following documents concerning our review of the subject facility are transmitted for your information:

✓	DESCRIPTION OF DOCUMENT	DATED
	Notice of Receipt of Application	
	Draft/Final Environmental Statement	
	Notice of Availability of Draft/Final Environmental Statement	
	Safety Evaluation Report, or Supplement No. _____	
	Environmental Assessment and Finding of No Significant Impact	
	Notice of Issuance of Environmental Assessment	
	Notice of Consideration of Issuance of Facility Operating License or Amendment to Facility Operating License.	
	Biweekly Notice; Applications and Amendments to Operating Licenses Involving No Significant Hazards Conditions See Page(s) _____	
	Exemption	
	Construction Permit No. CPPR- _____, Amendment No. _____	
	Facility Operating License No. _____, Amendment No. _____	
	Order	
	Monthly Operating Report for MARCH 1992 & REVISED MONTHLY REPORT transmitted by Letter	4/15/92
	Annual/Semi-Annual Report: FOR FEBRUARY 1992 _____ transmitted by Letter	
	Other _____	

Office of Nuclear Reactor Regulation
DIVISION OF REACTOR PROJECTS I/II

Enclosures:
As Stated

cc:

OFFICE ▶	... PDII-871A						
SURNAME ▶	... L. BERRY						
DATE ▶	... 4/28/92						

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 OCONEE READING
 L. BERRY

APRIL 28, 1992

DOCKET NO(S). 50-269/270/287

TO THOSE ON ATTACHED LIST

SUBJECT: OCONEE NUCLEAR STATION, UNITS 1/2/3

The following documents concerning our review of the subject facility are transmitted for your information.

✓	DESCRIPTION OF DOCUMENT	DATED
	Notice of Receipt of Application	
	Draft/Final Environmental Statement	
	Notice of Availability of Draft/Final Environmental Statement	
	Safety Evaluation Report, or Supplement No. _____	
	Environmental Assessment and Finding of No Significant Impact	
	Notice of Issuance of Environmental Assessment	
	Notice of Consideration of Issuance of Facility Operating License or Amendment to Facility Operating License	
	Biweekly Notice; Applications and Amendments to Operating Licenses Involving No Significant Hazards Conditions See Page(s) _____	
	Exemption	
	Construction Permit No. CPPR- _____, Amendment No. _____	
	Facility Operating License No. _____, Amendment No. _____	
	Order	
	Monthly Operating Report for _____ transmitted by Letter	
	Annual/Semi-Annual Report: _____ transmitted by Letter	
	Other <u>P:ERFORMANCE & OPERATING STATUS FOR MONTH OF FEB 1992</u>	3/13/92

Office of Nuclear Reactor Regulation
 DIVISION OF REACTOR PROJECTS I/II

Enclosures:
 As Stated

cc:

OFFICE ▶	PDII-3/EA						
SURNAME ▶	L. BERRY						
DATE ▶	4/28/92						

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OCONEE READING
L. BERRY

APRIL 28, 1992

DOCKET NO(S). 50-269/270/287

TO THOSE ON ATTACHED LIST

SUBJECT: OCONEE NUCLEAR STATION, UNITS 1/2/3

The following documents concerning our review of the subject facility are transmitted for your information:

✓	DESCRIPTION OF DOCUMENT	DATED
	Notice of Receipt of Application	
	Draft/Final Environmental Statement	
	Notice of Availability of Draft/Final Environmental Statement	
	Safety Evaluation Report, or Supplement No. _____	
	Environmental Assessment and Finding of No Significant Impact	
	Notice of Issuance of Environmental Assessment	
	Notice of Consideration of Issuance of Facility Operating License or Amendment to Facility Operating License	
	Biweekly Notice; Applications and Amendments to Operating Licenses Involving No Significant Hazards Conditions See Page(s) _____	
	Exemption	
	Construction Permit No. CPPR- _____, Amendment No. _____	
	Facility Operating License No. _____, Amendment No. _____	
	Order	
	Monthly Operating Report for _____ transmitted by Letter	
	Annual/Semi-Annual Report: _____ transmitted by Letter	
X	Other <u>AMEND TO NPDES PERMIT</u>	4/6/92

Office of Nuclear Reactor Regulation
Division of Reactor Projects I/II

Enclosures:
As Stated

cc:

OFFICE ▶	PDII-3/LA					
SURNAME ▶	L. BERRY					
DATE ▶	4/28/92					

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 OCONEE READING
 L. BERRY

May 7, 1992

DOCKET NO(S). 50-269/270/287

TO THOSE ON ATTACHED LIST

SUBJECT: OCONEE NUCLEAR STATION, UNITS 1/2/3

The following documents concerning our review of the subject facility are transmitted for your information.

✓	DESCRIPTION OF DOCUMENT	DATED
	Notice of Receipt of Application	
	Draft/Final Environmental Statement	
	Notice of Availability of Draft/Final Environmental Statement	
	Safety Evaluation Report, or Supplement No. _____	
	Environmental Assessment and Finding of No Significant Impact	
	Notice of Issuance of Environmental Assessment	
	Notice of Consideration of Issuance of Facility Operating License or Amendment to Facility Operating License	
	Biweekly Notice; Applications and Amendments to Operating Licenses Involving No Significant Hazards Conditions See Page(s) _____	
	Exemption	
	Construction Permit No. CPPR- _____, Amendment No. _____	
	Facility Operating License No. _____, Amendment No. _____	
	Order	
	Monthly Operating Report for _____ transmitted by Letter	
	Annual/Semi-Annual Report: _____ transmitted by Letter	
	Other ANNUAL RADIOLOGICAL ENVIRON OPERATING REPT 1991 _____	4/28/92

Office of Nuclear Reactor Regulation
 DIVISION OF REACTOR PROJECTS I/II

Enclosures:
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cc:

OFFICE ▶	PDII-3/1a						
SURNAME ▶	L. BERRY						
DATE ▶	5/7/92						

AKRS-1
 4/28/92

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 Oconee Reading
 McGuire Reading
 Catawba Reading
 L Berry

July 6, 1992

DOCKET NO(S): 50-269, 50-270, 50-287
 50-369, 50-370
 50-413, 50-414

TO THOSE ON ATTACHED LIST

SUBJECT: OCONEE NUCLEAR STATION, UNITS 1, 2, AND 3
 WILLIAM B. MCGUIRE NUCLEAR STATION, UNITS 1 AND 2
 CATAWBA NUCLEAR STATION, UNITS 1 AND 2

The following documents concerning our review of the subject facility are transmitted for your information.

✓	DESCRIPTION OF DOCUMENT	DATED
	Notice of Receipt of Application	
	Draft/Final Environmental Statement	
	Notice of Availability of Draft/Final Environmental Statement	
	Safety Evaluation Report, or Supplement No. _____	
	Environmental Assessment and Finding of No Significant Impact	
	Notice of Issuance of Environmental Assessment	
	Notice of Consideration of Issuance of Facility Operating License or Amendment to Facility Operating License	
	Biweekly Notice; Applications and Amendments to Operating Licenses Involving No Significant Hazards Conditions See Page(s) _____	
	Exemption	
	Construction Permit No. CPPR- _____, Amendment No. _____	
	Facility Operating License No. _____, Amendment No. _____	
	Order	
	Monthly Operating Report for _____ transmitted by Letter	
	Annual/Semi-Annual Report: _____ transmitted by Letter	
	Other NPDES NONCOMPLIANCE NOTIFICATION	6/11/92

Office of Nuclear Reactor Regulation
 DIVISION OF REACTOR PROJECTS I/II

Enclosures:
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cc:

OFFICE ▶	PDII-3						
SURNAME ▶	L Berry						
DATE ▶	6/7/92						

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 Oconee Reading
 LBedrry

July 6, 1992

DOCKET NO(S). 50-269, 50-270
 and 50-287

TO THOSE ON ATTACHED LIST

SUBJECT: OCONEE NUCLEAR STATION, UNITS 1, 2, AND 3

The following documents concerning our review of the subject facility are transmitted for your information.

✓	DESCRIPTION OF DOCUMENT	DATED
	Notice of Receipt of Application	
	Draft/Final Environmental Statement	
	Notice of Availability of Draft/Final Environmental Statement	
	Safety Evaluation Report, or Supplement No. _____	
	Environmental Assessment and Finding of No Significant Impact	
	Notice of Issuance of Environmental Assessment	
	Notice of Consideration of Issuance of Facility Operating License or Amendment to Facility Operating License	
	Biweekly Notice; Applications and Amendments to Operating Licenses Involving No Significant Hazards Conditions See Page(s) _____	
	Exemption	
	Construction Permit No. CPPR- _____, Amendment No. _____	
	Facility Operating License No. _____, Amendment No. _____	
	Order	
	Monthly Operating Report for MAY 1992 transmitted by Letter	5/31/92
	Annual/Semi-Annual Report: _____ transmitted by Letter	
	Other _____	

Office of Nuclear Reactor Regulation

DIVISION OF REACTOR PROJECTS I/II

Enclosures:
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cc:

ST
LB

OFFICE ▶	PDII-3						
SURNAME ▶	LBedrry						
DATE ▶	6/7/92						

July 23, 1992

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 Oconee Reading
 L Berry

DOCKET NO(S). 50-269, 50-270
 and 50-287

TO THOSE ON ATTACHED LIST

SUBJECT: OCONEE NUCLEAR STATION, UNITS 1, 2, AND 3

The following documents concerning our review of the subject facility are transmitted for your information.

	DESCRIPTION OF DOCUMENT	DATED
	Notice of Receipt of Application	
	Draft/Final Environmental Statement	
	Notice of Availability of Draft/Final Environmental Statement	
	Safety Evaluation Report, or Supplement No. _____	
	Environmental Assessment and Finding of No Significant Impact	
	Notice of Issuance of Environmental Assessment	
	Notice of Consideration of Issuance of Facility Operating License or Amendment to Facility Operating License	
	Biweekly Notice; Applications and Amendments to Operating Licenses Involving No Significant Hazards Conditions See Page(s) _____	
	Exemption	
	Construction Permit No. CPPR- _____, Amendment No. _____	
	Facility Operating License No. _____, Amendment No. _____	
	Order	
<input checked="" type="checkbox"/>	Monthly Operating Report for JUNE 1992 & REVISED MONTHLY REPT FOR transmitted by Letter	7/15/92 XXX
	Annual/Semi-Annual Report: MAY 1992 FOR OCONEE UNIT 1	
	_____ transmitted by Letter	
	Other _____	

Office of Nuclear Reactor Regulation
 Division of Reactor Projects I/II

Enclosures:
 As Stated

cc:

OFFICE▶	PDII-3					
SURNAME▶	L Berry					
DATE▶	7/23/92					

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 Docket File
 PDII-3 Reading
 Oconee Reading
 L. Berry

September 16, 1992

DOCKET NO(S). 50-269/50-270/50-287

TO THOSE ON ATTACHED LIST

SUBJECT: OCONEE NUCLEAR STATION, UNITS 1, 2, AND 3

The following documents concerning our review of the subject facility are transmitted for your information.

	DESCRIPTION OF DOCUMENT	DATED
✓	Notice of Receipt of Application	
	Draft/Final Environmental Statement	
	Notice of Availability of Draft/Final Environmental Statement	
	Safety Evaluation Report, or Supplement No. _____	
	Environmental Assessment and Finding of No Significant Impact	
	Notice of Issuance of Environmental Assessment	
	Notice of Consideration of Issuance of Facility Operating License or Amendment to Facility Operating License	
	Biweekly Notice; Applications and Amendments to Operating Licenses Involving No Significant Hazards Conditions See Page(s) _____	
	Exemption	
	Construction Permit No. CPPR— _____, Amendment No. _____	
	Facility Operating License No. _____, Amendment No. _____	
	Order	
	Monthly Operating Report for _____ transmitted by Letter	
X	Annual/Semi-Annual Report: XXXX RADIOACTIVE EFFLUENT RELEASE & supplemental info transmitted by Letter	8/28/92
	Other _____	

Office of Nuclear Reactor Regulation
 DIVISION OF REACTOR PROJECTS I/II

Enclosures:
 As Stated

cc:

OFFICE ▶	PDII-3					
SURNAME ▶	L. BERRY					
DATE ▶	9/16/92					

DISTRIBUTION
 docket File
 PDII-3 Reading
 Oconee Reading
 L.Berry

September 23, 1992

DOCKET NO(S). 50-269, 50-270, 50-287

TO THOSE ON ATTACHED LIST

SUBJECT: OCONEE NUCLEAR STATION, UNITS 1 AND 2

The following documents concerning our review of the subject facility are transmitted for your information.

✓	DESCRIPTION OF DOCUMENT	DATED
	Notice of Receipt of Application	
	Draft/Final Environmental Statement	
	Notice of Availability of Draft/Final Environmental Statement	
	Safety Evaluation Report, or Supplement No. _____	
	Environmental Assessment and Finding of No Significant Impact	
	Notice of Issuance of Environmental Assessment	
	Notice of Consideration of Issuance of Facility Operating License or Amendment to Facility Operating License	
	Biweekly Notice; Applications and Amendments to Operating Licenses Involving No Significant Hazards Conditions See Page(s) _____	
	Exemption	
	Construction Permit No. CPPR- _____, Amendment No. _____	
	Facility Operating License No. _____, Amendment No. _____	
	Order	
x	Monthly Operating Report for AUGUST 1992 & MONTHLY OPERATING STATUS transmitted by Letter	9/15/92
	Annual/Semi-Annual Report: REPORT FOR JULY 1992 _____ transmitted by Letter	
	Other _____	

Office of Nuclear Reactor Regulation
 DIVISION OF REACTOR PROJECTS I/II

Enclosures:
 As Stated

cc:

OFFICE ▶	PDII-3					
SURNAME ▶	LBerry					
DATE ▶	9/23/92					

January 29, 1993

DOCKET NO(S). 50-269, 50-270
 and 50-287

TO THOSE ON ATTACHED LIST

SUBJECT: OCONEE NUCLEAR STATION, UNITS 1 AND 2

The following documents concerning our review of the subject facility are transmitted for your information.

✓	DESCRIPTION OF DOCUMENT	DATED
	Notice of Receipt of Application	
	Draft/Final Environmental Statement	
	Notice of Availability of Draft/Final Environmental Statement	
	Safety Evaluation Report, or Supplement No. _____	
	Environmental Assessment and Finding of No Significant Impact	
	Notice of Issuance of Environmental Assessment	
	Notice of Consideration of Issuance of Facility Operating License or Amendment to Facility Operating License	
	Biweekly Notice; Applications and Amendments to Operating Licenses Involving No Significant Hazards Conditions See Page(s) _____	
	Exemption	
	Construction Permit No. CPPR- _____, Amendment No. _____	
	Facility Operating License No. _____, Amendment No. _____	
	Order	
X	Monthly Operating Report for <u>December 1992 & Revs'd Rept for Unit 3</u> transmitted by Letter	1/15/93
	Annual/Semi-Annual Report: _____ transmitted by Letter	
	Other _____	

Office of Nuclear Reactor Regulation
 Division of Reactor Projects I/II

Enclosures:
 As Stated

CC:

OFFICE▶	PDII-3						
SURNAME▶	LBerry						
DATE▶	1/29/93						

January 11, 1993

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 Docket File
 PDII-3 Reading
 Oconee Reading
 LBerry

DOCKET NO(S). 50-269, 50-270 and 50-287

TO THOSE ON ATTACHED LIST

SUBJECT: OCONEE NUCLEAR STATION, UNITS 1, 2, AND 3

The following documents concerning our review of the subject facility are transmitted for your information.

✓	DESCRIPTION OF DOCUMENT	DATED
	Notice of Receipt of Application	
	Draft/Final Environmental Statement	
	Notice of Availability of Draft/Final Environmental Statement	
	Safety Evaluation Report, or Supplement No. _____	
	Environmental Assessment and Finding of No Significant Impact	
	Notice of Issuance of Environmental Assessment	
	Notice of Consideration of Issuance of Facility Operating License or Amendment to Facility Operating License	
	Biweekly Notice; Applications and Amendments to Operating Licenses Involving No Significant Hazards Conditions See Page(s) _____	
	Exemption	
	Construction Permit No. CPPR- _____, Amendment No. _____	
	Facility Operating License No. _____, Amendment No. _____	
	Order	
	Monthly Operating Report for _____ transmitted by Letter	
	Annual/Semi-Annual Report: _____ transmitted by Letter	
X	Other <u>SEE ATTACHED</u>	

Office of Nuclear Reactor Regulation

Enclosures:
 As Stated

cc:

OFFICE▶	PDII- ³ LBerry						
SURNAME▶	LBerry						
DATE▶	1/11/93						

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 PDII-3 Reading
 L.Berry

April 5, 1993

DOCKET NO(S). 50-269, 50-270
 and 50-287

TO THOSE ON ATTACHED LIST

SUBJECT: OCONEE NUCLEAR STATION, UNITS 1, 2, AND 3

The following documents concerning our review of the subject facility are transmitted for your information.

✓	DESCRIPTION OF DOCUMENT	DATED
	Notice of Receipt of Application	
	Draft/Final Environmental Statement	
	Notice of Availability of Draft/Final Environmental Statement	
	Safety Evaluation Report, or Supplement No. _____	
	Environmental Assessment and Finding of No Significant Impact	
	Notice of Issuance of Environmental Assessment	
	Notice of Consideration of Issuance of Facility Operating License or Amendment to Facility Operating License	
	Biweekly Notice; Applications and Amendments to Operating Licenses Involving No Significant Hazards Conditions See Page(s) _____	
	Exemption	
	Construction Permit No. CPPR- _____, Amendment No. _____	
	Facility Operating License No. _____, Amendment No. _____	
	Order	
	Monthly Operating Report for _____ transmitted by Letter	
	Annual/Semi-Annual Report: _____ transmitted by Letter	
	Other SEE ATTACHED LIST	

Office of Nuclear Reactor Regulation

Enclosures:
 As Stated

CC:

Handwritten signature/initials

OFFICE▶	PDII-3					
SURNAME▶	L.Berry					
DATE▶	4/5/93					

OCONEE 1/2/3

1. Monthly Operating Report for January 1993. Transmitted by letter dated February 11, 1993.
2. Monthly Operating Report for February 1993, and revised reports for January 1993. Transmitted by letter dated March 15, 1993.
3. Semi-Annual Radioactive Effluent Release Report for July 1 through December 31, 1992. Transmitted by letter dated February 22, 1993.

May 5, 1993

DOCKET NO(S). 50-269/50-270/50-287

TO THOSE ON ATTACHED LIST

SUBJECT: OCONEE NUCLEAR STATION, UNITS 1, 2, AND 3

The following documents concerning our review of the subject facility are transmitted for your information.

✓	DESCRIPTION OF DOCUMENT	DATED
	Notice of Receipt of Application	
	Draft/Final Environmental Statement	
	Notice of Availability of Draft/Final Environmental Statement	
	Safety Evaluation Report, or Supplement No. _____	
	Environmental Assessment and Finding of No Significant Impact	
	Notice of Issuance of Environmental Assessment	
	Notice of Consideration of Issuance of Facility Operating License or Amendment to Facility Operating License	
	Biweekly Notice; Applications and Amendments to Operating Licenses Involving No Significant Hazards Conditions See Page(s) _____	
	Exemption	
	Construction Permit No. CPPR- _____, Amendment No. _____	
	Facility Operating License No. _____, Amendment No. _____	
	Order	
X	Monthly Operating Report for MARCH 1993 & REVSD REPT FOR FEB 1993 transmitted by Letter	4/15/93
X	Annual/Semi-Annual Report: RADIOLOGICAL ENVIRON OPERATING REPT 1992 transmitted by Letter	4/20/93
	Other _____	

Office of Nuclear Reactor Regulation
 Division of Reactor Projects - I/II

Enclosures:
 As Stated

cc:.

ACRS # 1

OFFICE ▶	PDII-3					
SURNAME ▶	L. Berry					
DATE ▶	5/5/.93					

June 14, 1993

DOCKET NO(S). 50-269/270/287

TO THOSE ON ATTACHED LIST

SUBJECT: OCONEE NUCLEAR STATION, UNITS 1, 2, AND 3

The following documents concerning our review of the subject facility are transmitted for your information.

✓	DESCRIPTION OF DOCUMENT	DATED
	Notice of Receipt of Application	
	Draft/Final Environmental Statement	
	Notice of Availability of Draft/Final Environmental Statement	
	Safety Evaluation Report, or Supplement No. _____	
	Environmental Assessment and Finding of No Significant Impact	
	Notice of Issuance of Environmental Assessment	
	Notice of Consideration of Issuance of Facility Operating License or Amendment to Facility Operating License	
	Biweekly Notice; Applications and Amendments to Operating Licenses Involving No Significant Hazards Conditions See Page(s) _____	
	Exemption	
	Construction Permit No. CPPR- _____, Amendment No. _____	
	Facility Operating License No. _____, Amendment No. _____	
	Order	
x	Monthly Operating Report for <u>APRIL 1993</u> transmitted by Letter	5/14/93
	Annual/Semi-Annual Report: _____ transmitted by Letter	
	Other _____	

Office of Nuclear Reactor Regulation
 Division of Reactor Projects - I/II

Enclosures:
 As Stated

cc:

OFFICE▶	PDII-3 LBery					
SURNAME▶	LBery					
DATE▶	6/14/93					

June 24, 1993

DOCKET NO(S). 50-269, 509270,
and 50-287

TO THOSE ON ATTACHED LIST

SUBJECT: OCONEE NUCLEAR STATION, UNITS 1, 2, AND 3

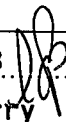
The following documents concerning our review of the subject facility are transmitted for your information.

✓	DESCRIPTION OF DOCUMENT	DATED
	Notice of Receipt of Application	
	Draft/Final Environmental Statement	
	Notice of Availability of Draft/Final Environmental Statement	
	Safety Evaluation Report, or Supplement No. _____	
	Environmental Assessment and Finding of No Significant Impact	
	Notice of Issuance of Environmental Assessment	
	Notice of Consideration of Issuance of Facility Operating License or Amendment to Facility Operating License	
	Biweekly Notice; Applications and Amendments to Operating Licenses Involving No Significant Hazards Conditions See Page(s) _____	
	Exemption	
	Construction Permit No. CPPR- _____, Amendment No. _____	
	Facility Operating License No. _____, Amendment No. _____	
	Order	
x	Monthly Operating Report for MAY 1993 MAY 1993 AND REVS'D REPTS FOR MARCH AND APRIL 1993 transmitted by Letter	6/15/93
	Annual/Semi-Annual Report: _____ transmitted by Letter	
	Other _____	

Office of Nuclear Reactor Regulation
Division of Reactor Projects - I/II

Enclosures:
As Stated

cc:

OFFICE▶	PDII-3 						
SURNAME▶	L. Berry						
DATE▶	6/24/93						

August 12, 1993

DOCKET NO(S). 50-269/270/287

TO THOSE ON ATTACHED LIST

SUBJECT: OCONEE NUCLEAR STATION, UNITS 1, 2, AND 3

The following documents concerning our review of the subject facility are transmitted for your information.

✓	DESCRIPTION OF DOCUMENT	DATED
	Notice of Receipt of Application	
	Draft/Final Environmental Statement	
	Notice of Availability of Draft/Final Environmental Statement	
	Safety Evaluation Report, or Supplement No. _____	
	Environmental Assessment and Finding of No Significant Impact	
	Notice of Issuance of Environmental Assessment	
	Notice of Consideration of Issuance of Facility Operating License or Amendment to Facility Operating License	
	Biweekly Notice; Applications and Amendments to Operating Licenses Involving No Significant Hazards Conditions See Page(s) _____	
	Exemption	
	Construction Permit No. CPPR- _____, Amendment No. _____	
	Facility Operating License No. _____, Amendment No. _____	
	Order	
X	Monthly Operating Report for <u>JUNE 1993</u> transmitted by Letter	7/15/93
	Annual/Semi-Annual Report: _____ transmitted by Letter	
	Other _____	

Office of Nuclear Reactor Regulation
 Division of Reactor Projects - I/II

Enclosures:
 As Stated

cc:

OFFICE▶	PDII-3						
SURNAME▶	L. Berry						
DATE▶	8/12/93						

August 25, 1993

DSITRIBUTION
Docket File
Reading File
L.Berry

DOCKET NO(S). 50-269/270/287

TO THOSE ON ATTACHED LIST

SUBJECT: OCONEE NUCLEAR STATION, UNITS 1,2, and 3

The following documents concerning our review of the subject facility are transmitted for your information.

✓	DESCRIPTION OF DOCUMENT	DATED
	Notice of Receipt of Application	
	Draft/Final Environmental Statement	
	Notice of Availability of Draft/Final Environmental Statement	
	Safety Evaluation Report, or Supplement No. _____	
	Environmental Assessment and Finding of No Significant Impact	
	Notice of Issuance of Environmental Assessment	
	Notice of Consideration of Issuance of Facility Operating License or Amendment to Facility Operating License	
	Biweekly Notice; Applications and Amendments to Operating Licenses Involving No Significant Hazards Conditions See Page(s) _____	
	Exemption	
	Construction Permit No. CPPR- _____, Amendment No. _____	
	Facility Operating License No. _____, Amendment No. _____	
	Order	
x	Monthly Operating Report for <u>JULY 1993 & REVSD MONTHLY OPERATING STATUS REPORTS FOR JUNE 1993</u> transmitted by Letter	8/13/93
	Annual/Semi-Annual Report: _____ transmitted by Letter	
	Other _____	

Office of Nuclear Reactor Regulation
Division of Reactor Projects - I/II

Enclosures:
As Stated

cc:

OFFICE▶	PDII-3						
SURNAME▶	L.Berry						
DATE▶	8/25/93						

October 28, 1993

DOCKET NO(S). 50-269/270,287

TO THOSE ON ATTACHED LLIST

SUBJECT: OCONEE NUCLEAR STATION, UNITS 1, 2, AND 3

The following documents concerning our review of the subject facility are transmitted for your information.

✓	DESCRIPTION OF DOCUMENT	DATED
	Notice of Receipt of Application	
	Draft/Final Environmental Statement	
	Notice of Availability of Draft/Final Environmental Statement	
	Safety Evaluation Report, or Supplement No. _____	
	Environmental Assessment and Finding of No Significant Impact	
	Notice of Issuance of Environmental Assessment	
	Notice of Consideration of Issuance of Facility Operating License or Amendment to Facility Operating License	
	Biweekly Notice; Applications and Amendments to Operating Licenses Involving No Significant Hazards Conditions See Page(s) _____	
	Exemption	
	Construction Permit No. CPPR- _____, Amendment No. _____	
	Facility Operating License No. _____, Amendment No. _____	
	Order	
x	Monthly Operating Report for AUGUST AND SEPTEMBER 1993 transmitted by Letter	9/15/93
x	Annual/Semi-Annual Report: FOR JANUARY - JUNE 1993 transmitted by Letter	10/15/93 8/30/93
	Other _____	

Office of Nuclear Reactor Regulation

Division of Reactor Projects I/II

Enclosures:
As Stated

cc:

OFFICE▶	PDII-3 L. Berry						
SURNAME▶	L. Berry						
DATE▶	10/28/93						

December 2, 1993

DOCKET NO(S). 50-268/270/287

TO THOSE ON ATTACHED LIST

SUBJECT: OCOONEE NUCLEAR STATION, UNITS 1, 2, AND 3

The following documents concerning our review of the subject facility are transmitted for your information.

✓	DESCRIPTION OF DOCUMENT	DATED
	Notice of Receipt of Application	
	Draft/Final Environmental Statement	
	Notice of Availability of Draft/Final Environmental Statement	
	Safety Evaluation Report, or Supplement No. _____	
	Environmental Assessment and Finding of No Significant Impact	
	Notice of Issuance of Environmental Assessment	
	Notice of Consideration of Issuance of Facility Operating License or Amendment to Facility Operating License	
	Biweekly Notice; Applications and Amendments to Operating Licenses Involving No Significant Hazards Conditions See Page(s)_____	
	Exemption	
	Construction Permit No. CPPR- _____, Amendment No. _____	
	Facility Operating License No. _____, Amendment No. _____	
	Order	
X	Monthly Operating Report for OCTOBER 1993 & MONTHLY OPERATING STATUS transmitted by Letter	11/15/93
	Annual/Semi-Annual Report: REPORT-PERSONNEL EXPOSURE FOR SEPTEMBER 1993 transmitted by Letter	
	Other _____	

Office of Nuclear Reactor Regulation
Division of Reactor Projects - I/II

Enclosures:
As Stated

cc:

OFFICE▶	PDII-3						
SURNAME▶	L.Berry						
DATE▶	12/2/93						

June 17, 1994

DOCKET NO(S). 50-269/270/287

Dr. William C. Cunningham
 FDA Research Chemist
 NIST Building 235/B125
 Gaithersburg, Maryland 20899

SUBJECT: OCONEE NUCLEAR STATION, UNITS 1/2/3

The following documents concerning our review of the subject facility are transmitted for your information.

✓	DESCRIPTION OF DOCUMENT	DATED
	Notice of Receipt of Application	
	Draft/Final Environmental Statement	
	Notice of Availability of Draft/Final Environmental Statement	
	Safety Evaluation Report, or Supplement No. _____	
	Environmental Assessment and Finding of No Significant Impact	
	Notice of Issuance of Environmental Assessment	
	Notice of Consideration of Issuance of Facility Operating License or Amendment to Facility Operating License	
	Biweekly Notice; Applications and Amendments to Operating Licenses Involving No Significant Hazards Conditions See Page(s) _____	
	Exemption	
	Construction Permit No. CPPR- _____, Amendment No. _____	
	Facility Operating License No. _____, Amendment No. _____	
	Order	
	Monthly Operating Report for _____ transmitted by Letter	
	Annual/Semi-Annual Report: _____ transmitted by Letter	
X	Other ANNUAL RADIOLOGICAL ENVIRONMENTAL OPERATING REPORT 1993	

Office of Nuclear Reactor Regulation

Enclosures:
 As Stated

cc: Chuck Wakamo
 EPA

OFFICE▶	Chief, Branch of Federal Activities	PDII-3	<i>ACRS-1</i>
SURNAME▶	U.S. Fish and Wildlife Service	L Berry	
DATE▶		6/17/94	

August 9, 1995

DOCKET NO(S). 50-269, 50-270
 and 50-287

Chief, Branch of Federal Activities
 Division of Habitat Conservation
 400 ARLSQ
 U.S. Fish and Wildlife Service
 1849 C Street, NW
 Washington, DC 20240

SUBJECT: OCONEE NUCLEAR STATION, UNITS 1/2/3

The following documents concerning our review of the subject facility are transmitted for your information.

✓	DESCRIPTION OF DOCUMENT	DATED
	Notice of Receipt of Application	
	Draft/Final Environmental Statement	
	Notice of Availability of Draft/Final Environmental Statement	
	Safety Evaluation Report, or Supplement No. _____	
	Environmental Assessment and Finding of No Significant Impact	
	Notice of Issuance of Environmental Assessment	
	Notice of Consideration of Issuance of Facility Operating License or Amendment to Facility Operating License	
	Biweekly Notice; Applications and Amendments to Operating Licenses Involving No Significant Hazards Conditions See Page(s) _____	
	Exemption	
	Construction Permit No. CPPR- _____, Amendment No. _____	
	Facility Operating License No. _____, Amendment No. _____	
	Order	
	Monthly Operating Report for _____ transmitted by Letter	
x	Annual/Semi-Annual Report XXXXXXXXXX Radioactive Effluent Release Report for 1994 transmitted by Letter	4/26/95
	Other _____	

Office of Nuclear Reactor Regulation
 Division of Reactor Projects - I/II

Enclosures:
 As Stated

cc: Chuck Wakamo
 Radiation Program Manager, Region 4

ACRS-1

OFFICE▶ SURNAME▶ DATE▶	Environmental Protection Agency 345 Courtland Street, NE Atlanta, Georgia 30365	PDII-2 L. Berry 8/9/95	
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Distribution
 Docket File
 PDII-2 Reading
 L. Berry

August 9, 1995

DOCKET NO(S). 50-269/270/287

Chief, Branch of Federal Activities
 Division of Habitat Conservation
 400 ARLSQ
 U.S. Fish and Wildlife Service
 1849 C Street, NW
 Washington, DC 20240

SUBJECT: OCONEE NUCLEAR STATION, UNITS 1/2/3

The following documents concerning our review of the subject facility are transmitted for your information.

✓	DESCRIPTION OF DOCUMENT	DATED
	Notice of Receipt of Application	
	Draft/Final Environmental Statement	
	Notice of Availability of Draft/Final Environmental Statement	
	Safety Evaluation Report, or Supplement No. _____	
	Environmental Assessment and Finding of No Significant Impact	
	Notice of Issuance of Environmental Assessment	
	Notice of Consideration of Issuance of Facility Operating License or Amendment to Facility Operating License	
	Biweekly Notice; Applications and Amendments to Operating Licenses Involving No Significant Hazards Conditions See Page(s) _____	
	Exemption	
	Construction Permit No. CPPR- _____, Amendment No. _____	
	Facility Operating License No. _____, Amendment No. _____	
	Order	
	Monthly Operating Report for _____ transmitted by Letter	
x	Annual/Semi-Annual Report: XXXXXX Radiological Environmental Operating Report 1994 transmitted by Letter	4/25/95
	Other _____	

Office of Nuclear Reactor Regulation
 Division of Reactor Projects - I/II

Enclosures:
 As Stated

cc: Dr. William C. Cunningham
 FDA Research Chemist

Chuck Wakamo
 Radiation Program Manager, Region 4
 Environmental Protection Agency
 345 Courtland Street, NE
 Atlanta, Georgia 30365

ACRS-1

OFFICE▶	NIST Building 235/B125 Gaithersburg, Maryland 20899	PDII-2	L. Berry
SURNAME▶		8/9/95	
DATE▶			