August 15, 1978

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50-270
and 50-287

Duke Power Company
ATTN: Mr. William O. Parker, Jr.
Vice President - Steam
Production
P. O. Box 2178
422 South Church Street
Charlotte, North Carolina 28242

Gentlemen:

By letter dated May 16, 1978, Duke Power Company submitted a proposed program for the installation of a limited number of tube sleeves in the Oconee Station once-through steam generator tubes. The purpose of these sleeves is to reduce dynamic stress in the region of previously indicated tube abnormalities.

We have previously reviewed the document submitted with your May 16 letter, entitled "Technical Justification for Installing and Testing Tube Sleeves in Once-Through Steam Generator - October 3, 1977 (Rev. 1 February 17, 1978)" on the Three Mile Island Unit No. 2 docket. The results of our review are provided in the Safety Evaluation Report, Supplement 2, for TMI-2, dated February 1978. Based on the above review we have no objection to your proposed installation and test program. A copy of page 5-4 of the TMI-2 Safety Evaluation Report is enclosed.

Sincerely,

Robert W. Reid, Chief
Operating Reactors Branch #4
Division of Operating Reactors

Enclosure:
Page 5-4 of TMI-2 SER

cc w/enclosure: See next page

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cc: Mr. William L. Porter
Duke Power Company
P. O. Box 2178
422 South Church Street
Charlotte, North Carolina 28242

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DeBevoise & Liberman
700 Shoreham Building
806-15th Street, NW.,
Washington, D.C. 20005

Oconee Public Library
201 South Spring Street
Walhalla, South Carolina 29691

Tube sleeving test program

Both Babcock & Wilcox and Combustion Engineering have designed test sleeves for insertion in the upper end of a number of steam generator tubes, with appropriate instrumentation, to assess their effect in stiffening the tubes to improve their capability to resist flow-induced vibration, the most likely cause of tube damage found in some other Babcock & Wilcox steam generators.

Safety evaluations have been submitted by the applicant for both designs, including descriptions of the sleeves, locations in the steam generator, method of installation, instrumentation, bench test results, stress analyses, and an evaluation of the effects of the test equipment on the safety of the plant.

Based on a review of the information supplied by the applicant and the similarity of the sleeves to installations in some Combustion Engineering steam generators, we conclude that neither the structural integrity nor the operational characteristics of safety-related equipment involved will be affected by the test program modifications, nor will the consequences of an accident be increased as a result thereof, and therefore that these modifications are acceptable.

After completion of all the above programs we will determine what modifications, if any, we will require to be made to Three Mile Island Unit 2.