UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION I
2100 RENAISSANCE BOULEVARD, SUITE 100
KING OF PRUSSIA, PA 19406-2713

July 28, 2015

Docket No. 05000302 License No. DPR-72

Terry Hobbs
Decommissioning General Manager
Duke Energy Florida, Inc.
Crystal River Unit 3
15760 W. Power Line Road
Crystal River, FL 34428-6708

SUBJECT: NRC INSPECTION REPORT NO. 05000302/2015008, DUKE ENERGY FLORIDA, INC., CRYSTAL RIVER UNIT 3, CRYSTAL RIVER, FLORIDA

Dear Mr. Hobbs:

On June 30, 2015, the U.S. Nuclear Regulatory Commission (NRC) completed its quarterly inspection under Inspection Manual Chapter 2561, “Decommissioning Power Reactor Inspection Program," at the permanently shut down Crystal River Nuclear Plant Unit 3 (CR-3). On-site inspections were performed on May 4-7 and June 22-25, 2015. In-office reviews of information supplied by Duke Energy Florida, Inc. were also performed during the inspection period. The inspection examined activities conducted under your license as they relate to safety and compliance with the Commission's rules and regulations, and the conditions of your license. The inspection consisted of observations by the inspectors, interviews with personnel, and a review of procedures and records. The results of the inspection were discussed with you and other members of the CR-3 staff on July 8, 2015, and are described in the enclosed report.

Based on the results of this inspection, the NRC has determined that one licensee identified Severity Level IV violation of NRC requirements occurred. This violation is being treated as a Non-Cited Violation (NCV), consistent with Section 2.3.2 of the Enforcement Policy. The NCV is described in the subject inspection report. If you contest the violation or significance of the NCV, you should provide a response within 30 days of the date of this inspection report, with the basis for your denial, to the Nuclear Regulatory Commission, ATTN: Document Control Desk, Washington DC 20555-0001, with copies to: (1) the Regional Administrator, Region I; (2) the Director, Office of Enforcement, United States Nuclear Regulatory Commission, Washington, DC 20555-0001; and (3) the NRC Senior Decommissioning Inspector, Region I.

In accordance with 10 CFR 2.390 of the NRC’s "Rules of Practice," a copy of this letter, its enclosure(s), and your response, will be made available electronically for public inspection in the NRC Public Document Room or from the NRC document system (ADAMS), accessible from the NRC website at http://www.nrc.gov/reading-rm/adams.html. To the extent possible, your response should not include any personal privacy, proprietary, or safeguards information so that it can be made available to the Public without redaction.

Current NRC regulations and guidance are included on the NRC's website at www.nrc.gov; select Radioactive Waste; Decommissioning of Nuclear Facilities; then Regulations, Guidance and Communications. The current Enforcement Policy is included on the NRC's website at www.nrc.gov; select About NRC, Organizations & Functions; Office of Enforcement; Enforcement documents; then Enforcement Policy (Under 'Related Information'). You may also obtain these documents by contacting the Government Printing Office (GPO) toll-free at 1-866-512-1800. The GPO is open from 8:00 a.m. to 5:30 p.m. EST, Monday through Friday (except Federal holidays).

No reply to this letter is required. Please contact Steve Hammann, at 610-337-5399, if you have any questions regarding this matter.

Sincerely,

/RA/

Marc S. Ferdas, Chief
Decommissioning and Technical Support Branch
Division of Nuclear Materials Safety

Enclosure: Inspection Report 05000302/2015008

cc w/encl: State of Florida
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Current NRC regulations and guidance are included on the NRC's website at www.nrc.gov; select Radioactive Waste; Decommissioning of Nuclear Facilities; then Regulations, Guidance and Communications. The current Enforcement Policy is included on the NRC's website at www.nrc.gov; select About NRC, Organizations & Functions; Office of Enforcement; Enforcement documents; then Enforcement Policy (Under 'Related Information'). You may also obtain these documents by contacting the Government Printing Office (GPO) toll-free at 1-866-512-1800. The GPO is open from 8:00 a.m. to 5:30 p.m. EST, Monday through Friday (except Federal holidays).

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U.S. NUCLEAR REGULATORY COMMISSION
REGION I

INSPECTION REPORT

Inspection No.	05000302/2015008
Docket No.	05000302
License No.	DPR-72
Licensee:	Duke Energy Florida, Inc. (Duke Energy)
Facility:	Crystal River Unit 3 (CR-3)
Location:	15760 W. Power Line Road Crystal River, FL 34428-6708
Inspection Dates:	April 1, 2015 to June 30, 2015
Inspectors:	Stephen Hammann, Senior Health Physicist Decommissioning and Technical Support Branch Division of Nuclear Materials Safety, Region I Keith Young, Senior Reactor Inspector Engineering Branch 3 Division of Reactor Safety, Region I
Approved By:	Marc Ferdas, Chief Decommissioning and Technical Support Branch Division of Nuclear Materials Safety, Region I

EXECUTIVE SUMMARY

Duke Energy
Crystal River Nuclear Plant
NRC Inspection Report No. 05000302/2015008

An announced quarterly inspection was completed at CR-3 on June 30, 2015. On-site inspections were conducted on May 4-7, and June 22-25, 2015. In-office reviews of information supplied by Duke Energy were also performed during the inspection period. The inspection included a review of operations, management oversight, adverse weather preparations, radioactive effluent control, radiological environmental monitoring program (REMP), fire protection, and plant support activities. The inspection consisted of observations by the inspectors, interviews with Duke Energy personnel, a review of procedures and records, and plant walk-downs. The NRC's program for overseeing the safe operation of a shut-down nuclear power reactor is described in Inspection Manual Chapter (IMC) 2561, "Decommissioning Power Reactor Inspection Program."

Based on the results of this inspection, a licensee-identified Severity Level (SL) IV Non-Cited Violation (NCV) of CR-3 Technical Specifications (TS) 5.6.2.17, "Technical Specifications Bases Control Program," was identified. Duke Energy did not submit any TS Bases updates to the NRC from April 30, 2009 to May 27, 2015. This violation is being treated as a NCV, consistent with Section 2.3.2 of the Enforcement Policy. Because this violation resulted in no potential safety consequences and it was entered into CR-3's corrective action program (Condition Report (CR) 749664), this violation is being treated as an NCV, consistent with the NRC Enforcement Policy.

REPORT DETAILS

1.0 Background

On February 20, 2013, Duke Energy sent a letter (Agency Documentation and Management System (ADAMS) Accession Number: ML13056A005) to the NRC certifying the permanent cessation of activities and certifying that the fuel had been permanently removed from the reactor. This met the requirements of 10 CFR 50.82(a)(1)(i) and 50.82(a)(1)(ii). As of the end of the reporting period, CR-3 was in the "Post Operation Transitional Phase" of decommissioning as described in IMC 2561.

2.0 Post Operation Transition Phase Performance and Status Review

a. Inspection Scope (Inspection Procedures (IPs) 36801, 37801, 40801, 60801, 71801, 84750)

The inspectors performed an on-site inspection the week of June 22-26, 2015. In-office reviews of information supplied by Duke Energy were also performed during the inspection period. The inspection consisted of observations by the inspectors, interviews with Duke Energy personnel, a review of procedures and records, and plant walk-downs.

The inspectors assessed management oversight of the site's transition to decommissioning status. The inspectors reviewed the sites organization, staffing levels, and personnel qualifications to ensure the safe storage of radioactive materials would not be adversely impacted by organization changes. The inspectors reviewed schedule and scope changes to assess the potential impact on the information contained in CR-3's Post-Shutdown Decommissioning Activities Report (PSDAR) and to ensure changes were consistent with NRC regulations and guidance.

The inspectors reviewed the status of system and equipment abandonment activities and plant modifications being made to support CR-3's transition to SAFSTOR status. The inspectors also reviewed CR-3's proposed plans for changes to the reactor building ventilation system (REG-0100, Attachment 1 – Screen, "Partial Abandonment of the RB Purge System," Revision 3).

The inspectors reviewed the CR-3 requirements for submitting changes to the TS bases to the NRC as required by the regulations and TS 5.6.2.17, "Technical Specifications Bases Control Program." The inspectors also reviewed corrective action reports to determine if issues were being properly assessed, reviewed, prioritized, and corrective actions were being appropriately implemented.

The inspectors observed spent fuel sipping activities being performed. The inspectors reviewed the equipment setup and evaluated CR-3's adherence to procedures, supervisory oversight, and communication and coordination. The inspector reviewed procedures, move sheets, and radiation work permit (RWP) associated with the spent fuel sipping campaign.

The inspectors evaluated CR-3's readiness for adverse weather (flooding and hurricanes). The inspectors performed a walk-down of the site, reviewed procedures and completed work orders to verify the CR-3 flooding mitigation plans and supplies were consistent with design requirements and risk analysis assumptions. The inspectors verified that sandbags, equipment, and supplies were properly staged as required by CR-3 procedure EM-220D, "Violent Weather (Permanently Defueled)."

The inspectors reviewed documentation associated with radioactive effluent control and site radiological environmental monitoring program (REMP) to determine the effectiveness of site radiological programs. The inspectors reviewed procedures, diagrams of effluent monitoring systems, liquid effluent release analysis (SDT-1, L-2015-0014; WDT-11A, and L-2015-0012), annual REMP report, annual effluent report, and the Off-Site Dose Calculation Manual (ODCM). The inspectors performed a walkdown of groundwater monitoring wells and site liquid and gaseous effluent monitoring points.

b. Observations and Findings

The inspectors noted that CR-3 made several schedule changes that impacted the information contained in their PSDAR. The changes were related to the timeline for the disposal of legacy waste, the timeline for the demolition of the Ready Warehouse, and an increase in overall construction costs for the Independent Spent Fuel Storage Installation (ISFSI) due to additional security modifications. The changes were documented in a letter to the U.S. Nuclear Regulatory Commission (NRC), dated June 11, 2015 (ML15175A188).

In preparation for the dry cask storage of spent fuel scheduled to start in 2017, 358 spent fuel assemblies were in the process of undergoing sipping. Sipping was being performed to assess material condition of spent fuel assemblies in preparation for dry cask storage. Previous to this sipping campaign a majority of the fuel assemblies had undergone sipping or ultrasonic (UT) testing. CR-3 was also going to perform visual inspections of a portion of the fuel assemblies in the SFP to further characterize their condition.

The inspectors noted that CR-3 recently issued a new adverse weather procedure, EM-220D. The procedure was revised to better reflect the permanently shutdown status of the site. The inspectors verified that sandbags, equipment, and supplies were properly staged as required by CR-3's procedure in preparation for adverse weather conditions that may arise at the site.

The inspectors verified that the annual radiological effluent and the annual REMP reports demonstrated that calculated doses were below regulatory dose criteria of 10 CFR 50, Appendix I. The inspectors also determined that waste treatment systems were maintained and operated in accordance with procedures. The inspector verified that liquid and gaseous effluent releases to the environment were being properly controlled, monitored, and quantified as required by NRC regulations. The inspectors noted that the reactor vessel was being maintained and filled with water for shielding purposes. The inspectors confirmed that the water level was being monitored by operations personnel. The inspectors confirmed that abandoned radiation monitors that were no

longer necessary to support plant operations were properly abandoned and the radiation monitors that remained in-service were properly being operated and maintained.

CR-3 personnel identified that the site had not submitted changes they had made to various bases sections within their TSs as required by TS 5.6.2.17 since 2009. The issue was placed in their corrective action program (CR 749664), evaluated, and appropriate corrective actions were implemented.

c. Conclusions

Based on the results of this inspection, one licensee-identified SL IV NCV of CR-3 TS 5.6.2.17 was identified.

CR-3 TS 5.6.2.17, "Technical Specifications (TS) Bases Control Program" states, in part, that changes to the bases implemented without prior NRC approval shall be provided to the NRC on a frequency consistent with 10 CFR 50.71, "Maintenance of Records, Making of Reports." 10 CFR 50.71(e)(4) states, in part, that subsequent revisions must be filed annually or 6 months after each refueling outage provided the interval between successive updates does not exceed 24 months; and the revisions must reflect all changes up to a maximum of 6 months prior to the date of filling. It also states that for nuclear power reactor facilities that have submitted the certifications required by 50.82(a)(1), subsequent revisions must be filed every 24 months. Contrary to the requirement, Duke Energy did not submit changes to the bases of the TS to the NRC at the required time intervals. Specifically, Duke Energy did not submit any TS Bases updates to the NRC from April 30, 2009 to May 27, 2015.

This violation is being treated as a NCV, consistent with Section 2.3.2 of the Enforcement Policy. Because this violation resulted in no potential safety consequences and it was entered into CR-3's corrective action program (Condition Report (CR) 749664), this violation is being treated as an NCV, consistent with the NRC Enforcement Policy. **(NCV 05000302/2015008-01; TS Bases Updates Not Submitted At Required Frequencies)**

3.0 Decommissioning Fire Protection Program (FPP)

a. Inspection Scope (IP 64704)

The inspectors reviewed CR-3's FPP to ensure Duke Energy adequately implemented their program in accordance with 10CFR50.48(f), "Fire Protection," the current CR-3 fire protection plan, final safety analysis report (FSAR), and Regulatory Guide 1.191, "Fire Protection Program for Nuclear Power Plants During Decommissioning and Permanent Shutdown." The inspectors also verified the FPP had been maintained in a state of operational readiness and changes made to the FFP program continue to meet commitments, NRC requirements, and have not negatively affected the overall state of the FPP at CR-3.

The inspectors performed an inspection of the FPP at CR-3 to determine whether it had been maintained in a state of operational readiness and whether changes made to the FPP continued to meet commitments, the NRC requirements, and had not negatively affected the overall state of the FPP at CR-3. Specifically, the inspectors:

- verified that the licensee had developed and implemented technically adequate procedures to implement the FPP;
- verified the proper installation, operability, and maintenance of fire protection systems and equipment; and
- reviewed the adequacy and implementation of the quality assurance program for the FPP.

The inspection consisted of interviews with Duke Energy personnel, a review of procedures and records, and plant walk-downs.

The inspectors reviewed changes made to CR-3's FPP. The inspectors also reviewed a sample of FPP procedures used for the storage of combustibles and flammables, performance of hot-work, control of ignition sources, and control of transient combustibles to ensure they were in compliance with the CR-3 FPP and adequately reflected the current decommissioning status of the facility. Additionally, the inspectors reviewed fire brigade training, training with off-site responders, qualifications, and responsibilities to ensure they were qualified to participate in firefighting activities. The inspectors performed a walkdown of firefighting equipment and equipment carts to ensure that they were properly maintained, inventoried and ready for use. Pre-fire plans were reviewed to ensure that they were up-dated and reflected the plants decommissioning status.

The inspectors reviewed the installed fire detection, suppression systems, and fire barriers in fire areas associated with the SFP, SFP cooling equipment, SFP power supply, and SFP inventory to ensure that they were maintained, surveillances were performed on a periodic basis, and capable of performing their intended function. This included review of fire pump capability and testing to ensure adequate water and water pressure could be supplied to the necessary systems and standpipes for fire suppression/firefighting activities. Part of this review included review of detection and suppression systems for the cable spreading room which contains a gaseous suppression system (Halon) and the "B" diesel generator room which contains a sprinkler suppression system. The inspectors also conducted a walk-down of plant detection systems, suppression systems, fire barriers, and fire pumps/water sources to ensure material condition was maintained.

The inspectors reviewed a sample of self-assessments and corrective action documents to ensure the licensee was identifying and placing FPP decommissioning deficiencies into the corrective action program.

b. Observations and Findings

The inspectors noted that CR-3 maintained the FPP within NRC requirements and the FPP. Fire protection detection systems, suppression systems, barriers, and fire pumps are being maintained and appropriately tested and in a state of operational readiness. The fire brigade training, qualifications and conduct of fire drills (announced and unannounced) were being properly implemented. The inspectors determined that proper FPP emphasis was placed on SFP systems and components to minimize the potential for radiological releases in the event of a fire at the plant.

The inspectors also noted that changes made to the CR-3 FPP did not reduce its effectiveness and that screenings and evaluations contained the appropriate level of detail and sufficient basis to support the changes. However, this review is not a formal safety evaluation and does not constitute formal NRC approval of the changes. Therefore, these changes remain subject to future NRC inspections.

c. Conclusions

Based on the results of this inspection, no findings of safety significance were identified.

3.0 Exit Meeting Summary

On July 8, 2015, the inspectors presented the inspection results, via teleconference, to Terry Hobbs, General Manager, and other members of Duke Energy's staff. The inspectors confirmed that proprietary information was not removed from the site.

PARTIAL LIST OF PERSONS CONTACTED

Licensee
B. Akins, Radiation Protection and Chemistry Manager
W. Basnight, Procedures
D. Demontfort, Maintenance
A. Doruff, Fire Protection Specialist
F. Frando, Technical Specialist II, Licensing
P. Gerardin, Nuclear Oversight, Lead Assessor
R. Ivey, Nuclear Oversight, Corporate Director of Audit & Programs
G. McCallum, Operations
P. Rose, Licensing
I. Wilson, General Manager Decommissioning (Acting)
T. Worthington, Engineering Manager
B. Wunderly, Plant Manager

ITEMS OPEN, CLOSED, AND DISCUSSED

05000302/2015008-01 NCV TS Bases Updates Not Submitted At Required Frequencies

LIST OF DOCUMENTS REVIEWED

Action Request (Condition Reports)
664047, 673234, 680894, 685436, 686427, 686967, 690389, 691739, 693222, 694092, 698726, 700871, 701925, 707158, 708318, 709581, 709689, 714369, 719375, 720264, 721384, 721580, 722097, 724401, 728589, 729925, 730549, 734834, 736107, 737953, 738620, 738973, 740234, 741929, 746903, 747654, 747844, 749664, 753559

Audits and Reports
2014-CR3-FP-01, (NOS) Crystal River Fire Protection Audit, 4/3/14
708467, Self-Assessment Report, B.5.b Spent Fuel Pool Makeup with Two Man Response Team, 9/16/14
Annual Radioactive Effluent Release Report 2014, Crystal River Unit 3
Annual Radiological Environmental Operating Report 2014, Crystal River Unit 3
Self-Assessment Report, Review of NEIL Standards Prior to Visit, 1/14

Calculations
F13-0003, CR-3 Spent Fuel Pool Time to Uncover Fuel Analysis, Revision 0
N13-0003, Public Dose from a Radioactive Waste Event, Revision 0

Design Basis Documents (DBD)
DBD, Electrical Support Systems Fire Detection System, Revision 3
Enhanced DBD, Fire Service System, Revision 20

Drawings
B-208-031-FS-00, Elementary Diagram, Fire Service System Index, Revision 9
B-208-031-FS-01, Elementary Diagram, Diesel Driven Fire Pump FSP-2A, Revision 20
B-208-031-FS-02, Elementary Diagram, Diesel Driven Fire Pump FSP-2B, Revision 13
B-208-031-FS-03, Elementary Diagram, Motor Driven Fire Service Pump FSP-1, Revision 19
FD-302-2321, Fire Service Water, Revision 93
FD-302-621, Spent Fuel Cooling, Revision 35
FD-302-751, Revision 64
FD-302-752, Revision 39
FD-302-785, Spent Fuel Chilled Water System CH-SF, Revision 2
L-001-022, Revision 50
L-001-023, Revision 29
P-304-783, Addition to Halon System to Cable Spreading Room, Revision 4

Evaluations
AI-9003, Attachment 2 (Hazard Identification), System Evaluation, Categorization and Abandonment Fire Service Water System, Revision 5
AI-9003, Attachment 2 (SSC Category Determination), System Evaluation, Categorization and Abandonment Fire Service Water System, Revision 5
Basic Cause Evaluation, Technical Specification Bases Control Program, CR-3 Has Not Submitted Updates
REG-0100, Attachment 1 – Screen, Partial Abandonment of the RB Purge System, Revision 3

Pre-Fire Plans
AI-2205A, Pre-Fire Plan-Control Complex, Revision 10
AI-2205B, Pre-Fire Plan-Turbine Building, Revision 12
AI-2205C, Pre-Fire Plan-Auxiliary Building, Revision12
AI-2205D, Pre-Fire Plan-Intermediate Building, Revision 6
AI-2205E, Pre-Fire Plan-Reactor Building, Revision 7
AI-2205F, Pre-Fire Plan-Miscellaneous Building and Components, Revision 18

Procedures

AAG-005, Contingencies for Loss of SF Pool Level, Revision 3
AI-151, Reporting Requirement Program, Revision 3
AI-604, Administrative Control of Structures, Material and Equipment Laydown Areas, Revision 17
AI-1000, Housekeeping/Material Condition Program, Revision 48
AI-2200, Guidelines for Handling use and Control of Transient Combustibles, Revision 13
AI-2205, Administration of CR-3 Fire Brigade Organization and Duties of the Fire Brigade, Revision 26
AI-2210, Fire Watch Program, Rev. 13
AI-9003, System Evaluation, Categorization and Abandonment, Revision 5
AP-880, Fire Protection, Revision 32
Areva, FS-156, Fuel Assembly Visual Examination using the Mobile Video Rig, Revision 005
Areva, FS-181, Vacuum Sipping Setup and Operation, Revision 008
CP-9300, Fire Protection – Minimum Requirements, Compensatory Measures, and Surveillance Requirements, Revision 3
EM-216, Duties of the Fire Brigade, Revision 28
EM-220D, Violent Weather (Permanently Defueled), Revision 0
FIR-0003, Hot Work Permit, Revision 1
OPS-1000, Conduct of Operations during Decommissioning, Revision 7
PM-185, Diesel Fire Pump Engine Inspection and Maintenance, Revision 10
REG-0010, 10CFR50.59 and Selected Regulatory Reviews, Rev. 3
SP-363, Fire Protection System Tests, Revision 45
SP-365A, Electric Fire Service Pump, FSP-1 Operability, Revision 25
SP-365B, Diesel Fire Service Pump, FSP-2A Operability, Revision 28
SP-408, Fire System Flow Test, Revision 21
SP-502, Fire Pump Diesel Batteries Weekly Inspection, Revision 23
SP-503, Fire Pump Diesel Batteries Quarterly Check, Revision 21
SP-731A, Auxiliary Building Ventilation Continuous Release, Revision 20
SP-736L, Liquid Releases to the Discharge Canal Via RM-L2, Revision 8
SP-736M, Liquid Releases to the Discharge Canal Via RM-L7, Revision 5

Procedures-Completed Surveillance Procedures

AI-9003, Revision 6, Attachment 2, SSC Category Determination Document, Reactor Building Purge (AH-XC), Revision 1
PM-175, Fire Door Maintenance, Revision 12, Completed 1/19/15
SP-190J, Cable Spreading Room Fire Detection System Test, Revision 22, Completed 8/6/14
SP-363, Fire Protection System (Fire Pump Tests), Revision 44, Completed 5/6/14
SP-365A, Electric Fire Service Pump, FSP-1 Operability, Revision 25, Completed 3/26/15
SP-365B, Diesel Fire Service Pump FSP-2A Operability, Revision 28, Completed 3/18/15
SP-407, Fire and Flood Barrier Penetration Seals Inspection, Revision 41, Completed 7/15/14
SP-408, Fire System Flow Test, Revision 19, Completed 9/23/11
SP-408, Fire System Flow Test, Revision 21, Completed 9/22/14
SP-411D, Fire Protection for Emergency Diesel Generators, Revision 27, Completed 1/6/15 & 1/9/14
SP-501A, Halon ASC Weight and Pressure Check, Revision 18, Completed 6/17/14
SP-501B, Halon System Functional Test, Revision 24, Completed 2/10/15
SP-502, Fire Pump Diesel Batteries Weekly Inspection, Revision 23, Completed 3/9/15
SP-503, Fire Pump Diesel Batteries Quarterly Check, Revision 20, Completed 1/19/15
SP-607, Fire Barrier Inspection, Revision 34, Completed 3/18/14
SP-800, Monthly Fire Extinguisher Inspections, Revision 52, Completed 1/25/15
SP-802, Fire Hose Hydro Test and Hose Reel Inspection, Revision 39, Completed 9/4/14
SP-804, Surveillance of Plant Fire Brigade Equipment, Revision 59, Completed 3/8/15
SP-805A, Annual Inspection of Plant Fire Doors, Revision 22, Completed 3/17/14

Miscellaneous

Crystal River Unit 3, Licensing Correspondence List
Degraded/OOS Equipment Log (Fire Protection), 5/5/15
EC 0000099162R1, Evaluation of External Flood Re-Definition
Effluent Release SDT-1, L-2015-0014; WDT-11A, L-2015-0012
Fire Brigade Roster, 4/16/15
Fire Team Leader (Incipient) Roster, 4/15/15
Key Safety Function Status Sheet, SF Pool, Work Week 15W15 (4/6/15 – 4/12/15)
MOU for Providing Citrus County Fire Protection Support to the CR-3 Complex, 5/5/14
Off-Site Dose Calculation Manual, Crystal River Unit 3, Revision 35
RWP 1060, Fuel Sipping Activities, Revision 2

Screenings and Reviews

664047, 50.59 Screen, Remove Appendix R and Safe Shutdown from FSAR (92014-007)
680894, 50.59 Screen, Fire Protection Transition to 50.48(f)
693222, 50.59 Screen, Revise Fire Brigade to Reflect SAFSTOR Plant Status
693270, 50.59 Screen, Revise Fire Brigade to Reflect SAFSTOR Plant Status
707158, 50.59 Screen, Changes to Fire Brigade Equipment
719375, 50.59 Screen, Partial Fire System Abandonment
721580, 50.59 Screen, Reg. Review of Fire Barrier Scope Reduction
730549, 50.59 Screen, Misc. Fire Service Procedure Changes for SAFSTOR
740234, 50.59 Screen, Review of Fire Protection Documents for SAFSTOR

Training-Fire Brigade Lessons and Plans
FWB01G/FPQ0001G, Hot work Fire Watch Training, Revision 14
TRF-30, CR-3, Off Site Responder Awareness Training, Revision 3
TRF-050, Initial Incipient Fire Brigade Lesson Plan, Revision 0
TRF-052, Combustion and Fire Behavior, Revision 0
TRF-053, Portable Fire Extinguisher 1, Revision 0
TRF-054, Portable Fire Extinguisher 2, Revision 0
TRF-055, Fire Alarm Systems, Revision 0
TRF-056, Fixed Extinguishing Systems, Revision 0
TRF-057, Fire Hose, Revision 0
TRF-058, Fire Streams and Nozzles, Revision 0
TRF-063, Plant Walk Trough's and Drills, Revision 0
TRF-064, Delta Training, Incipient Fire Brigade, Revision 0

Training-Fire Brigade Drills and Critiques
Unannounced Drill, Control Complex, 4160V ES Switchgear Room "A", Completed 5/10/14
Unannounced Drill, Control Complex, 4160V ES Switchgear Room "B", Completed 8/16/14
Unannounced Drill, 119' Auxiliary Building (SFP-1A), Completed 9/12/14
Announced Drill, 119' Auxiliary Building (SFP-1B), Completed 10/20/14
Unannounced Drill, CC/Spent Fuel Chiller Systems (CHHE-4A), Completed 1/20/15

Training-Competed Records for Fire Brigade and Fire Watch Qualifications
TRF000, Initial Fire Brigade Training, 1/12/15
TRF001, Fire Brigade Training, 1/12/15
TRF020, Initial Fire Team Leader Training, 1/12/15
TRF025, FTL Fire Suppression Techniques, 1/12/15
TRF026, FTL Fire Prevention Techniques, 1/12/15
TRF064, Incipient Fire Brigade Delta Training, 4/20/15
TRFA, Annual Hands On Fire Brigade Training, 4/20/15
2014 Fire Brigade Annual Drill Completions, 4/20/15
2015 Fire Brigade Drill Completions, 4/20/15
2015 Fire Brigade Annual Training Completions, 4/20/15
2015 Fire Watch Training Completions, 4/14-4/15
2015 Incipient Fire Brigade Qualification Completions, 4/20/15
2015 Incipient Fire Brigade Leader Qualification Completions, 4/20/15

Work Orders
00219232, 00219604, 00219605, 01885333, 01935802, 02216993, 02217532, 02273978, 02276244, 02284984, 13349913, 13313841, 13355289, 13377003, 13395423, 13455854

LIST OF ACRONYMS USED

CFR	Code of Federal Regulations
CR	Condition Report
CR-3	Crystal River Unit 3
Duke Energy	Duke Energy Florida, Inc.
FPP	Fire Protection Program
FSAR	Final Safety Analysis Report
IMC	Inspection Manual Chapter
ISFSI	Independent Spent Fuel Storage Installation
IP	Inspection Procedure
NRC	U.S. Nuclear Regulatory Commission
NCV	Non-Cited Violation
ODCM	Off-Site Dose Calculation Manual
PSDAR	Post Shutdown Decommissioning Activities Report
REMP	Radiological Environmental Monitoring Program
RWP	Radiation Work Permit
SFP	Spent Fuel Pool
SL	Severity Level
TS	Technical Specifications
UT	Ultrasonic Testing