



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

September 24, 2015

LICENSEE: Duke Energy Progress, Inc.

FACILITY: Shearon Harris Nuclear Power Plant, Unit 1

SUBJECT: SUMMARY OF JULY 15, 2015, MEETING WITH DUKE ENERGY PROGRESS, INC. TO DISCUSS THE SHEARON HARRIS NUCLEAR POWER PLANT, UNIT 1, PROPOSED LICENSE AMENDMENT REQUEST REGARDING TECHNICAL SPECIFICATIONS FOR THE PRESSURIZER AND MAIN STEAM LINE SAFETY VALVES (TAC NO. MF6233)

On July 15, 2015, a Category 1 public meeting was held between the U.S. Nuclear Regulatory Commission (NRC) and representatives of Duke Energy Progress, Inc. (Duke Energy, the licensee) that was conducted as a public teleconference. The purpose of the meeting was to have followup discussions to a June 11, 2015, meeting between Duke Energy and the NRC staff, including discussions regarding pressurizer water level technical specifications (TSs) and analysis. At the June 11, 2015, meeting, Duke Energy discussed a proposed license amendment request (LAR) for Shearon Harris Nuclear Power Plant, Unit 1 (Harris) regarding TSs for the pressurizer safety valves (PSVs) and main steam line safety valves (MSLSVs) lift setting setpoints. The meeting notice and agenda, dated July 1, 2015, are available in the Agencywide Documents Access and Management System (ADAMS) at Accession No. ML15183A266. There were no presentation materials for the meeting. A list of attendees is enclosed.

During the meeting, the licensee and the NRC staff discussed two main topics:

1. The impact of modifying the PSV and MSLSV TS lift setting tolerances on the pressurizer water level TS and associated analyses; and
2. The impact of modifying the PSV and MSLSV TS lift setting tolerances with regard to meeting Three Mile Island (TMI) Item II.D.1.

Regarding topic 1 above, the licensee discussed the historical and licensing basis for the pressurizer water level TSs for Westinghouse plants, in general, and for the Harris plant, in particular. The licensee referred to the following publicly available documents:

1. Section 3.4.9 of NUREG-1431, "Standard Technical Specifications, Westinghouse Plants, Revision, Volume 1, Specifications," and "Standard Technical Specifications, Westinghouse Plants, Revision 4.0, Volume 2, Bases" (ADAMS Accession Nos. ML12100A222 and ML12100A228, respectively)
2. Harris TS 3.4.3, included in Amendment Nos. 93 and 109 to the Facility Operating License for Harris (ADAMS Accession Nos. ML993570130 and ML020370248, respectively)

3. Technical Specification Task Force Traveler-162, "Maximum Pressurizer Water Level Limit Bases" (ADAMS Accession No. ML040490213)
4. Section 2.4.19 of the Safety Evaluation for Amendment No. 107 to the Facility Operating License for Harris (ADAMS Accession No. ML012830516)
5. SLNRC 84-0087, Letter from N. Petrick to H. Denton, dated May 25, 1984, "SNUPPS [Standardized Nuclear Unit Power Plant System] Technical Specifications Reactor Systems Branch Issues" (ADAMS Legacy Library No. 8405310209)

The NRC staff discussed a sensitivity study of the pressurizer water level using acceptable methods and reflecting the allowable tolerance changes associated with PSVs and MSLSVs setpoints for the applicable events discussed in the updated final safety analysis report (UFSAR), Chapter 15 for the proposed LAR. The staff discussed conformance to the acceptance criterion specified in various sections of Chapter 15, "Transient and Accident Analysis," of NUREG-0800, "Standard Review Plan for the Review of Safety Analysis Reports for Nuclear Power Plants: LWR Edition" (hereafter SRP) (see <http://www.nrc.gov/reading-rm/doc-collections/nuregs/staff/sr0800/ch15/>), which states that an incident of moderate frequency should not generate a more serious plant condition without other faults occurring independently.<sup>1</sup> The NRC staff noted that the events discussed in UFSAR, Chapter 15, which credited PSVs and MSLSVs in the analysis for consequence mitigation, may be considered the "applicable events" to be analyzed in support of TS changes on the allowable lift setting tolerances.

The NRC staff also indicated that since the pressurizer water level is an input parameter to the safety analyses being evaluated regarding the change to the PSVs and MSLSVs lift setting TS tolerances, the NRC staff review would include, in part, confirming that the pressurizer water level TS continue to meet the requirements of Title 10 of the *Code of Federal Regulations* (10 CFR), Section 50.36, and in particular, 10 CFR 50.36(c)(2)(ii).

Regarding topic 2 above, the licensee described the considerations it was addressing regarding TMI Item II.D.1. TMI Item II.D.1 is associated with the qualification of RCS relief and safety valves and, for pressurized water reactors, power-operated relief valve block valves for all fluid conditions expected under operating conditions, transients, and accidents. The NRC staff noted that the treatment of non-safety systems in the analysis of operating conditions, transients, and accidents should be conservative. The conservative assumption of whether a non-safety system actuates or not will depend on the operating conditions, transients, and accidents.

No regulatory decisions were made at the meeting. No members of the public were in attendance. No Public Meeting Feedback forms were received.

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<sup>1</sup> For example, see SRP Sections 15.2.1-15.2.5, "Loss of External Load; Turbine Trip; Loss of Condenser Vacuum; Closure of Main Steam Isolation Valve (BWR); and Steam Pressure Regulator Failure (Closed)," Revision 2, dated March 2007 (ADAMS Accession No. ML070300702), Subsection II, SRP Acceptance Criterion 2.C).

Please direct any inquiries to Dennis Galvin at 301-415-6256 or [Dennis.Galvin@nrc.gov](mailto:Dennis.Galvin@nrc.gov).



Martha Batillas, Project Manager  
Plant Licensing Branch II-2  
Division of Operator Reactor Licensing  
Office of Nuclear Reactor Regulation

Docket No. 50-400

Enclosure:  
List of Attendees

cc w/enclosure: Distribution via Listserv

LIST OF ATTENDEES  
JULY 15, 2015, MEETING WITH DUKE ENERGY PROGRESS, INC.  
SHEARON HARRIS NUCLEAR POWER PLANT, UNIT 1  
PROPOSED LICENSE AMENDMENT REQUEST REGARDING TECHNICAL  
SPECIFICATIONS FOR THE PRESSURIZER AND MAIN STEAM LINE SAFETY VALVES

**U.S. Nuclear Regulatory Commission**

C. Jackson  
S. Sun  
R. Beaton  
M. Hamm  
M. Barillas  
D. Galvin

**Duke Energy Progress, Inc.**

C. Ioppolo  
D. Earp  
N. Mirza  
M. Karki  
R. Harvey  
S. Thomas  
D. Tibbitts

Enclosure

Please direct any inquiries to Dennis Galvin at 301-415-6256 or [Dennis.Galvin@nrc.gov](mailto:Dennis.Galvin@nrc.gov).

Martha Barillas, Project Manager /RA/  
Plant Licensing Branch II-2  
Division of Operator Reactor Licensing  
Office of Nuclear Reactor Regulation

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\* concurred by email

OFFICE/	NRR/DORL/LPL2-2/PM	NRR/DORL/LPL2-2/PM	NRR/DORL/LPL2-2/LA	NRR/DSS/SRXB/BC
NAME	DGalvin	MBarillas	BClayton (LRonewicz for)	CJackson
DATE	09/24/15	08/10/15	08/07/15	08/26/15
OFFICE	NRR/DSS/STSB/BC	NRR/DORL/LPL2-2/BC	NRR/DORL/LPL2-2/PM	
NAME	RElliott *	SHelton	MBarillas	
DATE	08/26/15	09/21/15	09/24/15	

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