

NRR-PMDAPEm Resource

From: Dietrich, Allison
Sent: Tuesday, July 14, 2015 2:52 PM
To: Terry L Curtiss (tlcurtiss@aep.com)
Cc: 'hikish@aep.com'; Pelton, David; Jackson, Christopher; Palmrose, Donald; Dickson, Elijah
Subject: D.C. COOK UNITS 1 AND 2 - SRXB RAI CONCERNING LAR TO ADOPT TSTF-490 AND IMPLEMENT FULL-SCOPE AST (TAC NOS. MF5184 AND MF5185)
Attachments: SRXB RAI regarding Alternate Source Term MF5184 MF5185.pdf

By letter dated November 14, 2014 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML14324A209), as supplemented by letter dated February 12, 2015 (ADAMS Accession No. ML15050A247), Indiana Michigan Power Company (I&M) submitted a license amendment request for the Donald C. Cook Nuclear Plant, Units 1 and 2. The proposed amendment consists of adoption of Technical Specifications Task Force (TSTF)-490, Revision 0, and implementation of a full scope alternate source term radiological analysis methodology.

The U.S. Nuclear Regulatory Commission (NRC) staff in the Reactor Systems Branch of the Office of Nuclear Reactor Regulation is currently reviewing your submittal, as supplemented. The staff has determined that additional information is needed in order to complete the review, as described in the attached Request for Additional Information (RAI). The draft RAI was sent to I&M via electronic mail on July 7, 2015. A clarification telephone conference was held on July 10, 2015. Based on our discussion, we understand that a response to the RAI will be provided by August 26, 2015. A follow-up audit by the NRC staff will be scheduled for September 2015.

Please let me know if you have any questions or concerns.

Sincerely,

Allison W. Dietrich, Project Manager
Plant Licensing Branch III-1
Division of Operating Reactor Licensing
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Subject: D.C. COOK UNITS 1 AND 2 - SRXB RAI CONCERNING LAR TO ADOPT TSTF-490 AND IMPLEMENT FULL-SCOPE AST (TAC NOS. MF5184 AND MF5185)
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REQUEST FOR ADDITIONAL INFORMATION REGARDING
ADOPTION OF TSTF-490, REVISION 0
AND IMPLEMENTATION OF FULL-SCOPE ALTERNATE SOURCE TERM
DONALD C. COOK NUCLEAR PLANT, UNITS 1 AND 2
DOCKET NOS. 50-315 AND 50-316
TAC NOS. MF5184 AND MF5185

RAI-SRXB-1

The license amendment request (LAR) dated November 14, 2014 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML14324A209) listed in Enclosure 12 the various new alternative source term (AST) input parameter values, including those based on reactor coolant system (RCS) performance, for offsite and control room (CR) habitability doses for each accident analysis. The supplement dated February 12, 2015 (ADAMS Accession No. ML15050A247) provided information about the thermal-hydraulic (TH) analyses that were applied as the basis for the values.

As stated in the supplement, "The current licensing basis (CLB) thermal hydraulic (TH) calculations were used to provide input to most of the new dose analyses, although some of those inputs are different from previous inputs that were derived from the same TH calculations." The supplement also stated that the majority of the input parameters originated from calculations performed for previous license amendments for Donald C. Cook Nuclear Plant Units 1 and 2, such as license amendment Nos. 271 and 252 for implementation of AST for CR habitability, and license amendment Nos. 256 and 239 to address steam generator tube rupture (SGTR) overfill. Other inputs were obtained from projects implemented under Title 10 of the *Code of Federal Regulations* (10 CFR), Section 50.59, from information obtained from actual plant post-trip data, and from simulator data representing a Unit 1 SGTR transient.

The supplement provided additional descriptions of the TH analysis for each accident considered in the AST analysis. The source documents for most of the accident analyses, such as the volume data provided by Westinghouse Electric Company during a steam generator replacement project, are not available for the NRC staff to verify the proper incorporation of the values as input parameters to the AST analysis. Thus, the NRC staff cannot verify the authenticity of the RCS input parameter values to their source documentation, along with whether the sources, other than the previous license amendments, were reviewed and approved by the NRC.

- a. Provide information for all of the input parameter values provided in Enclosure 12 of the LAR connecting each value to its respective source documentation. Additionally, provide the source documentation that produced the input parameter values that were applied in each accident analysis. The source documentation can be submitted on the docket or made available for staff inspection via an audit. If placed on the docket and considered a proprietary document, submit a redacted version along with the proprietary version in accordance with 10 CFR 2.390.