NEXTera ENERGY SEABROOK

July 9, 2015

10CFR50.46

Docket No. 50-443
SBK-L-15140

United States Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, D.C. 20555-0001

Seabrook Station
Annual Reporting of Changes to, or Errors in
Emergency Core Cooling System Models or Applications

In accordance with the requirements of 10 CFR 50.46(a)(3)(ii), NextEra Energy Seabrook, LLC submits a tabulation of the Large Break and Small Break LOCA PCT margin utilization tables applicable to Seabrook Station summarizing the changes or errors discovered during calendar year 2014. Compliance with 10 CFR 50.46 requirements is demonstrated by the current composite Large Break LOCA PCT of 1939 °F remaining well below the limit of 2200 °F. The cumulative change in the Large Break LOCA PCT is +155 °F from the analysis of record value. The current Small Break LOCA PCT of 1373 °F also remains well below the limit of 2200 °F. There is currently no Small Break LOCA PCT cumulative change.

Should you have any questions regarding this report, please contact Mr. Kevin J. Randall, Reactor Engineering Supervisor, at (603) 773-7992.

Sincerely,

NextEra Energy Seabrook, LLC

Michael Ossing
Licensing Manager

cc: D. Dorman, NRC Region I Administrator
J. Lamb, NRC Project Manager, Project Directorate I-2
P. Cataldo, NRC Senior Resident Inspector

A002
NRR

ENCLOSURE TO SBK-L-15140

Seabrook SBLOCA and LBLOCA PCTs

LBLOCA	Peak Clad Temperature	Cumulative Change
2013 10 CFR 50.46 Annual Report [1]	1919 °F	135 °F
Errors in 2014		
- Error in Burst Strain Application [2]	+20 °F	20 °F
- Grid Heat Transfer Enhancement [3]	0 °F	0 °F
- Changes to Grid Blockage Ratio and Porosity [3]	0 °F	0 °F
- Changes in MONTECF Total Uncertainty [4]	0 °F	0 °F
2014 10 CFR 50.46 Annual Report	**1939 °F**	**155 °F**
SBLOCA		
2013 10 CFR 50.46 Annual Report [1]	1373 °F	0 °F
Errors in 2014		
- Fuel Rod Gap Conductance [5]	0 °F	0 °F
- Radiation Heat Transfer Model [5]	0 °F	0 °F
- SBLOCTA Pre-DNB Cladding Surface Heat Transfer Coefficient [5]	0 °F	0 °F
2014 10 CFR 50.46 Annual Report	**1373 °F**	**0 °F**

References

1. Letter from M. Ossing to U. S. Nuclear Regulatory Commission, "Seabrook Station Annual Reporting of Changes to, or Errors in Emergency Core Cooling System Models or Applications," SBK-L-14136, Accession No. ML1420A443, July 17, 2014.

2. Letter from M. Ossing to U. S. Nuclear Regulatory Commission, "Seabrook Station Best Estimate Large Break Loss of Coolant Accident (BE LOCA) 10 CFR 50.46 30-day Report," SBK-L-14040, Accession No. ML055A399, February 18, 2014.

3. Letter from M. Ossing to U. S. Nuclear Regulatory Commission, "Best Estimate Large Break Loss of Coolant Accident 10 CFR 50.46 30-day Report," SBK-L-14073, Accession No. ML14111A402, April 16, 2014.

4. Letter from M. Ossing to U. S. Nuclear Regulatory Commission, "Best Estimate Large Break Loss of Coolant Accident 10 CFR 50.46 30-day Report," SBK-L-14192, Accession No. ML14297A091, October 20, 2014.

5. Westinghouse letter, M. W. James to J. Perryman, "NextEra Energy Seabrook Station 10 CFR 50.46 Annual Notification and Reporting for 2014," NF-NA-15-47, March 9, 2015.