



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

July 1, 2015

Mr. Lawrence J. Weber  
Senior Vice President and  
Chief Nuclear Officer  
Indiana Michigan Power Company  
Nuclear Generation Group  
One Cook Place  
Bridgman, MI 49106

SUBJECT: DONALD C. COOK NUCLEAR PLANT, UNIT 1 - PUBLIC NOTICE OF  
APPLICATION FOR AMENDMENT TO RENEWED FACILITY OPERATING  
LICENSE (TAC NO. MF6390)

Dear Mr. Weber:

The enclosed announcement was forwarded to *The Herald-Palladium* newspaper for publication. The announcement relates to your application dated June 29, 2015, for amendment to Renewed Facility Operating License No. DPR-58 for the Donald C. Cook Nuclear Plant, Unit 1. The proposed amendment would revise Unit 1 Technical Specification 3.3.2, "Engineered Safety Feature Actuation System (ESFAS) Instrumentation."

If you have any questions, please contact me at 301-415-3049 or via e-mail at [Terry.Beltz@nrc.gov](mailto:Terry.Beltz@nrc.gov).

Sincerely,

A handwritten signature in black ink, appearing to read "Terry A. Beltz", with a long horizontal flourish extending to the right.

Terry A. Beltz, Senior Project Manager  
Plant Licensing Branch III-1  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Docket No. 50-315

Enclosure:  
Public Notice

cc w/encl: Distribution via Listserv

PUBLIC NOTICE

NRC STAFF PROPOSES TO AMEND OPERATING LICENSE AT THE  
DONALD C. COOK NUCLEAR PLANT, UNIT 1

The U.S. Nuclear Regulatory Commission (NRC) staff has received an application dated June 29, 2015 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML15181A002), from Indiana Michigan Power Company (the licensee), for an exigent amendment to the Unit 1 renewed facility operating license at the Donald C. Cook Nuclear Plant, located in Berrien County, Michigan.

The proposed amendment would permanently revise the Donald C. Cook Nuclear Plant, Unit 1 (CNP-1) Technical Specification (TS) 3.3.2, "Engineered Safety Feature Actuation System (ESFAS) Instrumentation," by adding a new Condition for one or more inoperable Required Channels for main feedwater (MFW) pump trips, changing Table 3.3.2-1 to add a footnote to the Applicable Mode Column for Mode 2 and to reflect the new Condition, and renumbering existing Conditions.

NRC Information Notice (IN) 2015-05, "Inoperability of Auxiliary and Emergency Feedwater Auto-Start Circuits on Loss of Main Feedwater Pumps," dated May 12, 2015 (ADAMS Accession No. ML15008A493) discusses several instances between 2006 and 2012 where licensees operated their MFW systems in such a manner that the automatic initiation of auxiliary feedwater (AFW) or emergency feedwater on loss of all MFW pumps was disabled. During its review of IN 2015-05, the licensee determined that operation of the CNP-1 MFW pumps resulted in a condition similar to those described in IN 2015-05. The licensee concluded that there is not a corresponding provision in the CNP-1 TSs that allows for the normal startup and shutdown of an MFW pump due to the design of the MFW pump trip channels and the MFW pump operational requirements for startup and shutdown of an MFW pump.

The licensee requested that the proposed amendment be processed on an exigent basis, in accordance with the provisions in Title 10 of the *Code of Federal Regulations* (10 CFR) Section 50.91(a)(6). Under 10 CFR 50.91(a)(6)(i)(B), where the Commission finds that exigent circumstances exist, in that a licensee and the Commission must act quickly and that time does not permit the Commission to publish a *Federal Register* notice allowing 30 days for prior public comment, and it also determines that the amendment involves no significant hazards considerations, the Commission will use local media to provide reasonable notice to the public in the area surrounding a licensee's facility of the licensee's amendment and of its proposed determination that no significant hazards consideration is involved, consulting with the licensee on the proposed media release and on the geographical area of its coverage.

The licensee's claim of exigent circumstances is based on the considerations below. On May 31, 2015, CNP-1 was shut down to repair an inoperable emergency diesel generator (EDG), which experienced a bearing failure during post-maintenance testing. The EDG bearing failure that resulted in the shutdown condition of CNP-1 could not have been reasonably foreseen. The upcoming startup of CNP-1 will require a normal startup of the CNP-1 MFW pumps, which currently have no TS provisions that reflect the design of the normal operation of the MFW pumps and the pump trip inputs for auto-actuation of the AFW pumps. The operating limitations of the CNP-1 TSs have existed since initial plant operation and were recently identified through the issuance of IN 2015-05, and could not have been reasonably foreseen. Based on the above, the licensee could not have avoided the situation that has resulted in the need for an exigent amendment.

The licensee requests approval of this proposed license amendment prior to Mode 2 entry during restart of CNP-1 from the current forced outage, where Mode 2 is defined by TSs as less than or equal to 5 percent rated thermal power. The licensee projects startup of CNP-1

to commence on or about July 12, 2015. Therefore, this does not allow time for a 30-day public comment period as specified in 10 CFR 50.91(a)(2)(ii) prior to issuance.

As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration using the standards in 10 CFR 50.92. The licensee and NRC have evaluated this proposed change with regard to the determination of whether or not a significant hazards consideration is involved.

Operation of CNP-1 in accordance with the proposed amendment will not involve a significant increase in the probability or consequences of an accident previously evaluated. The design basis events which impose initiation of the AFW system requirements are loss of normal main feedwater, main steam line break, loss of offsite power (LOOP), and a small-break loss-of-coolant accident. These design bases event evaluations assume actuation of the AFW system due to LOOP signal, a steam generator water level low-low, or a safety injection signal. The anticipatory motor-driven AFW pump auto-start signals from the MFW pumps are not credited in any design-basis accidents and are, therefore, not part of the primary success path for postulated accident mitigation as defined by 10 CFR 50.36(c)(2)(ii), Criterion 3. Modifying the Completion Time clock activation requirements, providing a Condition and Required Actions for more than one inoperable channel for this function, and modifying Modes 1 and 2 Applicability for this function will not impact any previously evaluated design-basis accidents. Therefore, the proposed change does not involve a significant increase in the probability or consequences of an accident previously evaluated.

The proposed amendment will not create the possibility of a new or different kind of accident from any previously analyzed. The TS change allows for one or more MFW pump channels to be inoperable during Modes 1 and 2, and has an operational allowance during Modes 1 and 2 for placing MFW pumps in service or securing MFW pumps. This change

involves an anticipatory AFW auto-start function that is not credited in the accident analysis. Since this change only affects the conditions at which this auto-start function needs to be operable and does not affect the function that actuates AFW due to LOOP, low-low steam generator level, or a safety injection signal, it will not be an initiator to a new or different kind of accident from any accident previously evaluated. Therefore, the proposed amendment does not create the possibility of a new or different kind of accident from any accident previously evaluated.

The proposed amendment will not involve a significant reduction in a margin of safety. This TS change involves the automatic start of the AFW pumps due to trip of both MFW pumps, which is not an assumed start signal for design basis events. This change does not modify any values or limits involved in a safety related function or accident analysis. Therefore, the proposed amendment does not involve a significant reduction in a margin of safety.

Following an initial review of this application, the requested amendment has been evaluated against the standards in 10 CFR 50.92 and the NRC staff has made a proposed (preliminary) determination that the requested amendment involves no significant hazards consideration. The changes do not significantly increase the probability or consequences of any accident previously considered, nor create the possibility of an accident of a different kind, nor significantly decrease any margin of safety.

The NRC is providing opportunity for the public to comment on the no significant hazards determination. The NRC may inform the licensee of the public's comments, as necessary and appropriate. If the Commission determines that the proposed license amendment request should be granted, then the NRC will publish a notice of issuance in the *Federal Register* under 10 CFR 2.106; and a hearing will be provided after issuance if one has been requested by a person who satisfies the provisions for intervention specified in 10 CFR 2.309.

Comments on the proposed determination of no significant hazards consideration may be (1) telephoned to David L. Pelton, Chief, Plant Licensing Branch III-1, by collect call to 301-415-2307, or by facsimile to 301-415-2102, (2) e-mailed to [David.Pelton@nrc.gov](mailto:David.Pelton@nrc.gov), or (3) submitted in writing to the Chief, Rules, Announcements and Directives Branch, Division of Administrative Services, Office of Administration, Mail Stop: OWFN-12-H08, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001. All comments received by 5:00 p.m. on July 8, 2015, will be considered in reaching a final determination. A copy of the application may be examined electronically through ADAMS in the NRC Library at <http://www.nrc.gov/reading-rm/adams.html> and at the Commission's Public Document Room (PDR), located at One White Flint North, Public File Area O1-F21, 11555 Rockville Pike (first floor), Rockville, Maryland. Persons who do not have access to ADAMS or who encounter problems in accessing the documents located in ADAMS should contact the NRC PDR Reference staff by telephone at 1-800-397-4209, or 301-415-4737, or by e-mail to [pdr.resource@nrc.gov](mailto:pdr.resource@nrc.gov).

July 1, 2015

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Senior Vice President and  
Chief Nuclear Officer  
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Bridgman, MI 49106

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Sincerely,

*/RA/*

Terry A. Beltz, Senior Project Manager  
Plant Licensing Branch III-1  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Docket No. 50-315

Enclosure:  
Public Notice

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**ADAMS Accession Nos.: Letter - ML15180A384      Public Notice - ML15180A369**

OFFICE	DORL/LPL3-1/PM	DORL/LPL4-1/LA	OGC	DORL/LPL3-1/PM	DORL/LPL3-1/PM
NAME	TBeltz	JBurkhardt	DRoth	DPelton	TBeltz
DATE	06/30/2015	6/30/2015	06/30/2015	07/01/2015	07/01/2015

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