### Administrative Topics Outline - RO DRAFT

Form ES-301-1

Facility: <u>PLANT E. I. HATC</u> Exam Level: <u>RO Ø</u> SRO	<u>H ILT 9</u> -  🔲 S	Date of Examination: <u>03/23/2015</u> <u>BRO-U </u> Operating Test No.: <u>2015-301</u>							
Administrative Topic (see Note)	Type Code*	Describe activity to be performed							
Conduct of Operations Admin 1	M, R	U1 - Correct Reactor Water Level for high drywell temperatures. <b>G2.1.35</b> (3.9/4.2) <b>ALL</b>							
Conduct of Operations Admin 2	D, R	Perform Att. 15 of 34GO-OPS-001-1, "IRM Alternate Power Check" and then determine any additional actions. <b>G2.1.23</b> (4.3/4.4) <b>ALL</b>							
Equipment Control Admin 3	N, R	Review RCIC Pump Operability Surveillance G2.2.12 (3.7/4.1) ALL							
Emergency Procedures/Plan Admin 4	M, R	Determine the Evacuation Route During an Emergency G2.4.39 (3.9/3.8) RO ONLY							
NOTE: All items (5 total) are required for SROs. RO applicants require only 4 items unless they are retaking only the administrative topics, when all 5 are required.									
<ul> <li>* Type Codes &amp; Criteria: (C)ontrol room, (S)imulator, or Class(R)oom (D)irect from bank (≤ 3 for ROs; ≤ 4 for SROs &amp; RO retakes) (N)ew or (M)odified from bank (≥ 1) (P)revious 2 exams (≤ 1; randomly selected)</li> </ul>									

### Administrative Topics Outline – SRO-I DRAFT

Form ES-301-1

Facility:       PLANT E. I. HATCH ILT 9       Date of Examination: 03/23/2015         Exam Level:       RO I       SRO-I I       SRO-U       Operating Test No.: 2015-301										
Administrative Topic (see Note)	Type Code*	Describe activity to be performed								
Conduct of Operations Admin 1	M, R	U1 - Correct Reactor Water Level for high drywell temperatures. <b>G2.1.35</b> (3.9/4.2) <b>ALL</b>								
Conduct of Operations Admin 2	D, R	Perform Att. 15 of 34GO-OPS-001-1, "IRM Alternate Power Check" and then determine any additional actions. <b>G2.1.23</b> (4.3/4.4) <b>ALL</b>								
Equipment Control Admin 3	N, R	Review RCIC Pump Operability Surveillance G2.2.12 (3.7/4.1) ALL								
Radiation Control Admin 5	M, R	Evaluate Venting DW Irrespective of Offsite Release rates IAW PCG G2.3.11 (4.3) SRO ONLY								
Emergency Procedures/Plan Admin 6	D, S, R	Given Plant Conditions, Determine the Emergency Classification and complete EN Form/ G2.4.29 (4.0) SRO ONLY								
NOTE: All items (5 total) are required for SROs. RO applicants require only 4 items unless they are retaking only the administrative topics, when all 5 are required.										
<ul> <li>* Type Codes &amp; Criteria: (C)ontrol room, (S)imulator, or Class(R)oom (D)irect from bank (≤ 3 for ROs; ≤ 4 for SROs &amp; RO retakes) (N)ew or (M)odified from bank (≥ 1) (P)revious 2 exams (≤ 1; randomly selected)</li> </ul>										

#### Control Room/In-Plant Systems Outline - RO DRAFT Form ES-301-2

### Facility: PLANT E. I. HATCH ILT 9

Date of Examination: 03/23/2015

Exam Level: RO 🗹 SRO-I 🗆 SRO-U 🗆

Operating Test No.: 2015-301

#### Control Room Systems<sup>@</sup> (8 for RO); (7 for SRO-I); (2 or 3 for SRO-U, including 1 ESF)

System / JPM Title	Type Code*	Safety Function
<b>CR/SIM 1</b> – Normal Start of Recirc ASD With Cooling Water Failure	A, L, D, S	SF-1 Reactivity Control 202001K6.02 (3.1/3.2) ALL
<b>CR/SIM 2 –</b> Lower RWL Using The RHR System, LR-JP-06.11 (1 <sup>st</sup> Loop fails to start)	A, L, M, S	SF-2 Reactor Water Inventory Control 20300A4.01 (4.3/4.1) ALL
<b>CR/SIM 3</b> – Roll the Main Turbine	A, L, D, S	SF-3 Reactor Pressure Control 245000A4.06 (2.7/2.6) ALL
<b>CR/SIM 4 –</b> HPCI / Place HPCI in Pressure Control Mode	D, S	SF-4 Heat Removal From The Core 206000A4.06 (4.3/4.3) ALL
<b>CR/SIM 5</b> – Initiate Emergency Torus Venting using the Emergency Vent Path (Alt path)	A, M, S	SF-5 Containment Integrity 223001A4.07 (4.2/4.1) ALL
<b>CR/SIM 6 –</b> Re-Energize Bus with Diesel Generator	A, D, S	SF-6 Partial or Complete Loss of AC 295003A1.02 (4.2/4.3) ALL
<b>CR/SIM 7</b> – Loss of Air Actions for Rx Bldg Ventilation	A, D, S	SF-8 Plant Service System 295019AK2.08 (2.8/2.9) ALL
<b>CR/SIM 8 –</b> Place Control Room HVAC Systems in the Purge Mode, JPM 25026	C , L, D	SF-9 Radioactivity Release 290003A4.01 (3.9/4.0) <b>RO ONLY</b>
In-Plant Systems <sup>@</sup> (3 for RO); (3 for SRO-I); (3 o	or 2 for SRO-U)	
PLANT 1 – JP20024 Actuate DG CO <sub>2</sub> Sys	D, E	SF-8 Plant Service System 286000A2.08 (3.2/3.3) ALL
PLANT 2 – Loss of Air / Align Emergency Nitrogen to drywell Pneumatics – JPM 25028	D, R, E	SF-3 Reactor Pressure Control 218000A2.03 (RO 3.4/SRO 3.6) ALL
PLANT 3 – JP02718-07.1 Transfer LPCI Bus	D	SF-6 Electrical 262001A4.03 (3.2/3.4) ALL

## Control Room/In-Plant Systems Outline - SRO-I DRAFT Form ES-301-2

# Facility: PLANT E. I. HATCH ILT 9

Date of Examination: 03/23/2015 SRO-U

Exam Level: RO D SRO-I Ø

Operating Test No.: 2015-301

#### Control Room Systems<sup>@</sup> (8 for RO); (7 for SRO-I); (2 or 3 for SRO-U, including 1 ESF)

System / JPM Title	Type Code*	Safety Function
<b>CR/SIM 1</b> – Normal Start of Recirc ASD With Cooling Water Failure	A, L, D, S	SF-1 Reactivity Control 202001K6.02 (3.1/3.2) ALL
<b>CR/SIM 2 –</b> Lower RWL Using The RHR System, LR-JP-06.11 (1 <sup>st</sup> Loop fails to start)	A, L, M, S	SF-2 Reactor Water Inventory Control 20300A4.01 (4.3/4.1) ALL
<b>CR/SIM 3 –</b> Roll the Main Turbine	A, L, D, S	SF-3 Reactor Pressure Control 245000A4.06 (2.7/2.6) ALL
<b>CR/SIM 4 –</b> HPCI / Place HPCI in Pressure Control Mode	D, S	SF-4 Heat Removal From The Core 206000A4.06 (4.3/4.3) ALL
<b>CR/SIM 5</b> – Initiate Emergency Torus Venting using the Emergency Vent Path (Alt path)	A, M, S	SF-5 Containment Integrity 223001A4.07 (4.2/4.1) ALL
<b>CR/SIM 6</b> – Re-Energize Bus with Diesel Generator	A, D, S	SF-6 Partial or Complete Loss of AC 295003A1.02 (4.2/4.3) ALL
<b>CR/SIM 7</b> – Loss of Air Actions for Rx Bldg Ventilation	A, D, S	SF-8 Plant Service System 295019AK2.08 (2.8/2.9) ALL
In-Plant Systems <sup>@</sup> (3 for RO); (3 for SRO-I); (3 o	or 2 for SRO-U)	
PLANT 1 – JP20024 Actuate DG CO <sub>2</sub> Sys	D, E	SF-8 Plant Service System 286000A2.08 (3.2/3.3) ALL
PLANT 2 – Loss of Air / Align Emergency Nitrogen to drywell Pneumatics – JPM 25028	D, R, E	SF-3 Reactor Pressure Control 218000A2.03 (RO 3.4/SRO 3.6) ALL
PLANT 3 – JP02718-07.1 Transfer LPCI Bus	D	SF-6 Electrical 262001A4.03 (3.2/3.4) <b>ALL</b>

ES-401, Rev. 9

#### **BWR Examination Outline**

Form ES-401-1

Facility     Hatch     Date of Exam:     March 2015																			
Tier Group			RO K/A Category Points													SRO-Only Points			
		к 1	к 2	к 3	к 4	K 5	К 6	A 1	A 2	A 3	A 4	G *	Total	1	<b>\2</b>	0	G*	Total	
1.	1	3	4	3				3	3			4	20		4	6	3	7	
Emergency & Abnormal Plant	2	1	1	1		N/A		2	1	N	A	1	7		2		1	3	
Evolutions	Tier Totals	4	5	4				5	4			5	27		6	4	•	10	
	1	3	2	2	2	3	2	2	2	3	3	2	26		3	2	2	5	
2. Plant	2	1	1	1	1	1	1	2	1	1	1	1	12	0	2	1		3	
Systems	Tier Totals	4	3	3	3	4	3	4	3	4	4	3	38		5		3	8 -	
3. Generic K	nowledge and	d Ab	ilitie	s		1	:	2	:	3	4	4	10	1	2	3	4	7	
	Categories					2	1	2		3	3	3		1	2	2	2		
<ol> <li>Ensure that at least two topics from every applicable K/A category are sampled within each tier of the RO and SRO-only outlines (i.e., except for one category in Tier 3 of the SRO-only outline, the ATier Totals<sup>©</sup> in each K/A category shall not be less than two).</li> <li>The point total for each group and tier in the proposed outline must match that specified in the table. The final point total for each group and tier may deviate by ∀1 from that specified in the table based on NRC revisions. The final RO exam must total 75 points and the SRO-only exam must total 25 points.</li> <li>Systems/evolutions within each group are identified on the associated outline; systems or evolutions that do not apply at the facility should be deleted and justified; operationally important, site-specific systems that are not included on the outline should be added. Refer to ES-401, Attachment 2, for guidance regarding the elimination of inappropriate K/A statements.</li> <li>Select topics from as many systems and evolutions as possible; sample every system or evolution in the group before selecting a second topic for any system or evolution.</li> <li>Absent a plant-specific priority, only those K/As having an importance rating (IR) of 2.5 or higher shall be selected. Use the RO and SRO ratings for the RO and SRO-only portions, respectively.</li> <li>Select SRO topics for Tiers 1 and 2 from the shaded systems and K/A categories.</li> <li>*The generic (G) K/As in Tiers 1 and 2 shall be selected from Section 2 of the K/A Catalog, but the topics must be relevant to the applicable evolution or system.</li> <li>On the following pages, enter the K/A numbers, a brief description of each topic, the topics= importance ratings (IRs) for the applicable license level, and the point totals (#) for each system and category. Enter the group and the rotals for each category in the table bove; if fuel handling equipment is sampled in other than Category A2 or G* on the SRO-only exam.</li> <li>For Tier</li></ol>																			

ES-401, REV 9			T1G	<b>31 BWR EXAMINATION OUTLINE</b>	FORM ES-401-1
KA	NAME / SAFETY FUNCTION:	ļ	IR	K1 K2 K3 K4 K5 K6 A1 A2 A3 A4	G TOPIC:
		RO	SRO	)	
295001AA2.06	Partial or Complete Loss of Forced Core Flow Circulation / 1 & 4	3.2	3.3		Nuclear boiler instrumentation
295003AK2.04	Partial or Complete Loss of AC / 6	3.4	3.5		A.C. electrical loads
295004AK2.01	Partial or Total Loss of DC Pwr / 6	3.1	3.1		Battery charger
295005AK3.02	Main Turbine Generator Trip / 3	3.4	3.5		Recirculation pump downshift/trip: Plant-Specific
295006G2.1.31	SCRAM / 1	4.6	4.3		Ability to locate control room switches, controls and indications and to determine that they are correctly reflecting the desired plant lineup.
295016G2.4.11	Control Room Abandonment / 7	4.0	4.2		Knowledge of abnormal condition procedures.
295018AA2.02	Partial or Total Loss of CCW / 8	3.1	3.2		Cooling water temperature
295019AA1.02	Partial or Total Loss of Inst. Air / 8	3.3	3.1		Instrument air system valves: Plant-Specific
295021AK3.01	Loss of Shutdown Cooling / 4	3.3	3.4		Raising reactor water level
295023AK1.03	Refueling Acc Cooling Mode / 8	3.7	4.0		Inadvertent criticality
295024EA2.08	High Drywell Pressure / 5	3.6	4.0		Drywell radiation levels

ES-401, RE	V 9		T1G	<b>1 BWR EXAMINATION OUTLINE</b>	FORM ES-401-		
KA	NAME / SAFETY FUNCTION:		IR	K1 K2 K3 K4 K5 K6 A1 A2 A3 A4 G	TOPIC:		
		RO	SRO	)			
295025EK2.01	High Reactor Pressure / 3	4.1	4.1		RPS		
295026EK1.02	Suppression Pool High Water Temp. / 5	3.5	3.8		Steam condensation		
295028G2.1.25	High Drywell Temperature / 5	3.9	4.2		Ability to interpret reference materials such as graphs, monographs and tables which contain performance data.		
295030EA1.03	Low Suppression Pool Wtr Lvl / 5	3.4	3.4		HPCS: Plant-Specific		
295031EK2.09	Reactor Low Water Level / 2	3.3	3.4		Recirculation system: Plant-Specific		
295037EK3.03	SCRAM Condition Present and Power Above APRM Downscale or Unknown / 1	4.1	4.5		Lowering reactor water level		
295038EA1.01	High Off-site Release Rate / 9	3.9	4.2		Stack-gas monitoring system: Plant-Specific		
600000AK1.02	Plant Fire On Site / 8	2.9	3.1		Fire Fighting		
700000G2.2.44	Generator Voltage and Electric Grid Distrurbancecs	4.2	4.4		Ability to interpret control room indications to verify the status and operation of a system, and understand how operator actions and directives affect plant and system conditions		

ES-401, RE	EV 9	-	T1G2 BWR EXAMINATION OUTLINE	FORM ES-401-		
KA	NAME / SAFETY FUNCTION:	IB	R K1 K2 K3 K4 K5 K6 A1 A2 A3 A4 G	TOPIC:		
		RO	SRO			
295008G2.1.20	High Reactor Water Level / 2	4.6	4.6	Ability to execute procedure steps.		
295014AA2.04	Inadvertent Reactivity Addition / 1	4.1	4.4	Violation of fuel thermal limits		
295015AK1.02	Incomplete SCRAM / 1	3.9	4.1	Cooldown effects on reactor power		
295020AA1.02	Inadvertent Cont. Isolation / 5 & 7	3.2	3.2	Drywell ventilation/cooling system		
295032EK3.03	High Secondary Containment Area Temperature / 5	3.8	3.9	Isolating affected systems		
295033EK2.02	High Secondary Containment Area Radiation Levels / 9	3.8	4.1	Process radiation monitoring system		
295036EA1.04	Secondary Containment High Sump/Area Water Level / 5	3.1	3.4	Radiation monitoring: Plant-Specific		

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ES-401, REV 9			T2G	<b>1 BWR EXAMINATION OUTLINE</b>	FORM ES-401-1		
KA	NAME / SAFETY FUNCTION:		IR	K1 K2 K3 K4 K5 K6 A1 A2 A3 A4 G	TOPIC:		
		RO	SRC				
203000A2.15	RHR/LPCI: Injection Mode	4.2	4.2		Loop selection logic failure: Plant-Specific		
205000K3.02	Shutdown Cooling	3.2	3.3		Reactor water level: Plant-Specific		
206000A1.04	HPCI	3.7	3.8		Suppression pool level: BWR-2,3,4		
206000A4.03	HPCI	3.1	3		Turbine temperatures: BWR-2,3,4		
209001K2.01	LPCS	3.0	3.1		Pump power		
211000K1.05	SLC	3.4	3.6		RWCU		
212000K6.05	RPS	3.5	3.8		RPS sensor inputs		
215003A2.05	IRM	3.3	3.5		Faulty or erratic operation of detectors/system		
215004A3.01	Source Range Monitor	3.2	3.2		Meters and recorders		
				5			
215004K2.01	Source Range Monitor	2.6	2.8		SRM channels/detectors		
215005K5.03	APRM / LPRM	2.9	3.3		Control rod symmetrical patterns		

ES-401, REV 9			T2G	<b>1 BWR EXAMINATION OUTLINE</b>	FORM ES-401-		
KA	NAME / SAFETY FUNCTION:		IR	K1 K2 K3 K4 K5 K6 A1 A2 A3 A4 G	TOPIC:		
		RO	SRO				
217000K3.03	RCIC	3.5	3.5		Decay heat removal		
218000K1.04	ADS	3.9	4.2		Drywell/containment pressure: Plant-Specific		
218000K5.01	ADS	3.8	3.8		ADS logic operation		
223002A4.01	PCIS/Nuclear Steam Supply Shutoff	3.6	3.5		Valve closures		
223002K6.02	PCIS/Nuclear Steam Supply Shutoff	3.0	3.2		D.C. electrical distribution		
239002K1.07	SRVs	3.6	3.8		Suppression pool		
259002A3.04	Reactor Water Level Control	3.2	3.2		Changes in reactor feedwater flow		
261000A4.02	SGTS	3.1	3.1		Suction valves		
262001A1.04	AC Electrical Distribution	2.7	2.9		Load currents		
262002K4.01	UPS (AC/DC)	3.1	3.4		Transfer from preferred power to alternate power supplies		
263000G2.1.19	DC Electrical Distribution	3.9	3.8		Ability to use plant computer to evaluate system or component status.		

ES-401, RE	:V 9		T2G	<b>1 BWR EXAMINATION OUTLINE</b>	FORM ES-401-1		
KA	NAME / SAFETY FUNCTION:		IR	K1 K2 K3 K4 K5 K6 A1 A2 A3 A4 G	TOPIC:		
		RO	SRO	· · · · · · · · · · · · · · · · · · ·			
263000K5.01	DC Electrical Distribution	2.6	2.9		Hydrogen generation during battery charging		
264000A3.06	EDGs	3.1	3.2		Cooling water system operation		
300000G2.2.42	Instrument Air	3.9	4.6		Ability to recognize system parameters that are entry- level conditions for Technical Specifications		
400000K4.01	Component Cooling Water	3.4	3.9		Automatic start of standby pump		

ES-401, REV 9			T20	<b>2 BWR EXAMINATION OUTLINE</b>	FORM ES-401-1
KA	NAME / SAFETY FUNCTION:		IR	K1 K2 K3 K4 K5 K6 A1 A2 A3 A4 G	TOPIC:
		RO	SRC	)	
201002K3.03	RMCS	2.9	3.0		Ability to process rod block signals
201003A1.02	Control Rod and Drive Mechanism	2.8	2.8		CRD drive pressure
201006A1.03	RWM	2.9	3.0		Latched group indication: P-Spec(Not-BWR6)
219000A2.13	RHR/LPCI: Torus/Pool Cooling Mode	3.5	3.7		High suppression pool temperature
230000K2.02	RHR/LPCI: Torus/Pool Spray Mode	2.8	2.9		Pumps
245000K5.07	Main Turbine Gen. / Aux.	2.6	2.9		Generator operations and limitations
256000K6.01	Reactor Condensate	2.8	2.8		Plant air systems
259001G2.4.35	Reactor Feedwater	3.8	4.0		Knowledge of local auxiliary operator tasks during emergency and the resultant operational effects
271000K4.01	Offgas	2.9	3.3		Dilution of hydrogen gas concentration
272000A3.02	Radiation Monitoring	3.6	3.7		Offgas system isolation indications
288000K1.04	Plant Ventilation	2.6	2.6		Applicable component cooling water system: Plant- Specific

ES-401, REV 9			T2G	2 BWR EXAMINATION OUTLINE	FORM ES-401-1
KA	NAME / SAFETY FUNCTION:	I	R	K1 K2 K3 K4 K5 K6 A1 A2 A3 A4 G TOPIC:	
		RO	SRO		
290001A4.04	Secondary CTMT	2.6	2.7	Auxiliary building area temper	ture: Plant-Specific

ES-401, REV 9			Т3	BWR EXAMINATION OUTLINE	FORM ES-401-1	
KA	NAME / SAFETY FUNCTION:		IR	K1 K2 K3 K4 K5 K6 A1 A2 A3 A4 G	TOPIC:	
		RO	SRO	)		
G2.1.23	Conduct of operations	4.3	4.4		Ability to perform specific system and integrated plant procedures during all modes of plant operation.	
G2.1.40	Conduct of operations	2.8	3.9		Knowledge of refueling administrative requirements	
G2.2.21	Equipment Control	2.9	4.1		Knowledge of pre- and post-maintenance operability requirements.	
G2.2.36	Equipment Control	3.1	4.2		Ability to analyze the effect of maintenance activities, such as degraded power sources, on the status of limiting conditions of operations	
G2.3.11	Radiation Control	3.8	4.3		Ability to control radiation releases.	
G2.3.13	Radiation Control	3.4	3.8		Knowledge of radiological safety procedures pertaining to licensed operator duties	
G2.3.14	Radiation Control	3.4	3.8		Knowledge of radiation or contamination hazards that may arise during normal, abnormal, or emergency conditions or activities	
G2.4.21	Emergency Procedures/Plans	4.0	4.6		Knowledge of the parameters and logic used to assess the status of safety functions	
G2.4.32	Emergency Procedures/Plans	3.6	4.0		Knowledge of operator response to loss of all annunciators.	
G2.4.49	Emergency Procedures/Plans	4.6	4.4		Ability to perform without reference to procedures those actions that require immediate operation of system components and controls.	

ES-401, RE	:V 9	SRO	T1G1 BWR EXAMINATION OUTLINE	FORM ES-401-1	
КА	NAME / SAFETY FUNCTION:	IR	K1 K2 K3 K4 K5 K6 A1 A2 A3 A4 G	TOPIC:	
		RO SR	0		
295001G2.1.7	Partial or Complete Loss of Forced Core Flow Circulation / 1 & 4	4.4 4.7		Ability to evaluate plant performance and make operational judgments based on operating characteristics, reactor behavior and instrument interpretation.	
295006AA2.02	SCRAM / 1	4.3 4.4		Control rod position	
295024EA2.02	High Drywell Pressure / 5	3.9 4.(		Drywell temperature	
295026EA2.01	Suppression Pool High Water Temp. / 5	4.1 4.2		Suppression pool water temperature	
295028EA2.05	High Drywell Temperature / 5	3.6 3.8		Torus/suppression chamber pressure: Plant-Specific	
295038G2.4.8	High Off-site Release Rate / 9	3.8 4.5		Knowledge of how abnormal operating procedures are used in conjunction with EOPs.	
600000G2.4.47	Plant Fire On Site / 8	4.2 4.2		Ability to diagnose and recognize trends in an accurate and timely manner utilizing the appropriate control room reference material.	

ES-401, REV 9			RO T	T1G2 BWR EXAMINATION OUTLINE	FORM ES-401-1	
KA	NAME / SAFETY FUNCTION:	IR		K1 K2 K3 K4 K5 K6 A1 A2 A3 A4 G	TOPIC:	
		RO	SRC	0		
295008AA2.05	High Reactor Water Level / 2	2.9	3.1		Swell	
295014G2.4.11	Inadvertent Reactivity Addition / 1	4.0	4.2		Knowledge of abnormal condition procedures.	
295035EA2.01	Secondary Containment High Differential Pressure / 5	3.8	3.9		Secondary containment pressure: Plant-Specific	

ES-401, REV 9			RO T	2G1 BWR EXAMINATION OUTLINE	FORM ES-401-1
KA	NAME / SAFETY FUNCTION:	1	R	K1 K2 K3 K4 K5 K6 A1 A2 A3 A4 G	TOPIC:
		RO	SRO	)	
205000A2.07	Shutdown Cooling	2.7	2.7		Loss of motor cooling: Plant-Specific
206000A2.07	HPCI	3.4	3.6		Low suppression pool level: BWR-2,3,4
				. 6	
259002G2.4.20	Reactor Water Level Control	3.8	4.3		Knowledge of operational implications of EOP warnings, cautions and notes.
262002G2.4.46	UPS (AC/DC)	4.2	4.2		Ability to verify that the alarms are consistent with the plant conditions.
223002A2.01	PCIS/Nuclear Steam Suppy Shutoff	3.2	3.5		AC Electrical Distribution failures

ES-401, REV 9		S	RO T	T2G2 BWR EXAMINATION OUTLINE	FORM ES-401-1	
KA	NAME / SAFETY FUNCTION:	I	R	K1 K2 K3 K4 K5 K6 A1 A2 A3 A4 G TOPIC:		
		RO	SRO	RO		
202001A2.26	Recirculation	2.9	3.1	1 Incomplete start sequence: Plant-S	pecific	
215002G2.1.19	RBM	3.9	3.8	8 Ability to use plant computer to evalue component status.	uate system or	
268000A2.02	Radwaste	2.3	2.7	7		

ES-401, REV 9		1	SRO	<b>T3 BWR EXAMINATION OUTLINE</b>	FORM ES-401-1	
KA	NAME / SAFETY FUNCTION:	J	IR	K1 K2 K3 K4 K5 K6 A1 A2 A3 A4 G	TOPIC:	
		RO	SRO			
G2.1.39	Conduct of operations	3.6	4.3		Knowledge of conservative decision making practices	
G2.2.14	Equipment Control	3.9	4.3		Knowledge of the process for controlling equipment configuration or status	
G2.2.25	Equipment Control	3.2	4.2		Knowledge of the bases in Technical Specifications for limiting conditions for operations and safety limits.	
G2.3.12	Radiation Control	3.2	3.7		Knowledge of radiological safety principles pertaining to licensed operator duties	
G2.3.15	Radiation Control	2.9	3.1		Knowledge of radiation monitoring systems	
G2.4.31	Emergency Procedures/Plans	4.2	4.1		Knowledge of annunciators alarms, indications or response procedures	
G2.4.9	Emergency Procedures/Plans	3.8	4.2		Knowledge of low power / shutdown implications in accident (e.g. LOCA or loss of RHR) mitigation strategies.	