



**UNITED STATES
NUCLEAR REGULATORY COMMISSION**
REGION II
245 PEACHTREE CENTER AVENUE NE, SUITE 1200
ATLANTA, GEORGIA 30303-1257

June 11, 2015

Mr. David R. Vineyard
Vice President
Southern Nuclear Operating Company, Inc.
Edwin I. Hatch Nuclear Plant
11028 Hatch Parkway North
Baxley, GA 31513

**SUBJECT: NOTIFICATION OF EDWIN I. HATCH NUCLEAR PLANT - NRC
COMPONENT DESIGN BASES INSPECTION - INSPECTION REPORT
05000321/2015007 AND 05000366/2015007**

Dear Mr. Vineyard:

The purpose of this letter is to notify you that the U.S. Nuclear Regulatory Commission (NRC) Region II staff will conduct a component design bases inspection at your Edwin I. Hatch Nuclear Plant during the weeks of September 21 - 25, October 05 - 09, and October 19 - 23, 2015. Theodore Fanelli, a Senior Reactor Inspector from the NRC's Region II Office, will lead the inspection team. This inspection will be conducted in accordance with the baseline inspection procedure, Procedure 71111.21, Component Design Bases Inspection, issued November 29, 2013.

The inspection will evaluate the capability of risk significant/low margin components to function as designed and to support proper system operation. The inspection will also include a review of selected operator actions, operating experience, and modifications.

During a telephone conversation on June 10, 2015, Mr. Fanelli confirmed with Mr. Collins, of your staff, arrangements for an information-gathering site visit, and the three-week onsite inspection. The schedule is as follows:

- Information-gathering visit: Week of August 31 - September 05, 2015
- Onsite weeks: September 21 - 25, October 05 - 09, and October 19 - 23, 2015

The purpose of the information-gathering visit is to meet with members of your staff to identify risk-significant components and operator actions. Information and documentation needed to support the inspection will also be identified. Mr. John Hanna, a Region II Senior Reactor Analyst, will accompany Mr. Fanelli during the information-gathering visit to review probabilistic risk assessment data and identify risk significant components that will be examined during the inspection.

The Enclosure lists documents that will be needed prior to the information-gathering visit. Please provide the referenced information to the Region II office by August 24, 2015. Contact Mr. Fanelli with any questions concerning the requested information. The inspectors will try to

minimize your administrative burden by specifically identifying only those documents required for inspection preparation.

Additional documents will be requested during the information-gathering visit. The additional information will need to be available to the team in the Region II office prior to the first day of the preparation week, September 14, 2015. Mr. Fanelli will also discuss the following inspection support administrative details: availability of knowledgeable plant engineering and licensing personnel to serve as points of contact during the inspection; method of tracking inspector requests during the inspection; licensee computer access; working space; arrangements for site access; and other applicable information.

In accordance with 10 CFR 2.390 of the NRC's "Rules of Practice," a copy of this letter and its Enclosure will be available electronically for public inspection in the NRC Public Document Room or from the Publicly Available Records (PARS) component of NRC's document system (ADAMS). ADAMS is accessible from the NRC Web site at <http://www.nrc.gov/reading-rm/adams.html> (the Public Electronic Reading Room).

Thank you for your cooperation in this matter. If you have any questions regarding the information requested or the inspection, please contact Mr. Fanelli at 404-997-4433 or me at 404-997-4607.

Sincerely,

/RA/

Jonathan H. Bartley, Chief
Engineering Branch 1
Division of Reactor Safety

Docket Nos. 50-321, 50-366
License Nos. DPR-57, NPF-5

Enclosure:
Information Request for Edwin I. Hatch Nuclear Plant –
Component Design Bases Inspection

cc: Distribution via Listserv

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☒ PUBLICLY AVAILABLE ☐ NON-PUBLICLY AVAILABLE ☐ SENSITIVE ☒ NON-SENSITIVE
ADAMS: ☒ Yes ACCESSION NUMBER: _____ ☒ SUNSI REVIEW COMPLETE ☐ FORM 665 ATTACHED

OFFICE	RII:DRS	RII:DRS					
SIGNATURE	TNF1	JHB1					
NAME	TFANELLI	JBARTLEY					
DATE	6/11/2015	6/11/2015	6/ /2015	6/ /2015	6/ /2015	6/ /2015	6/ /2015
E-MAIL COPY?	YES NO	YES NO	YES NO	YES NO	YES NO	YES NO	YES NO

OFFICIAL RECORD COPY DOCUMENT NAME: S:\DRS\ENG BRANCH 1\BRANCH INSPECTION FILES\2014-2015-2016
CYCLE INSPECTION FOLDER FOR ALL SITES\HATCH\CDBI 2015\HATCH CDBI 2015007 NOTIFICATION LETTER.DOCX

INFORMATION REQUEST FOR EDWIN I. HATCH NUCLEAR PLANT COMPONENT DESIGN BASES INSPECTION

Please provide the information electronically in searchable “.pdf” files, Excel, or other searchable format on CDROM (or FTP site, SharePoint, etc.) The CDROM (or website) should be indexed and hyperlinked to facilitate ease of use.

1. From your most-recent Probabilistic Safety Analysis (PSA) excluding external events and fires:
 - a. Two risk rankings of components from your site-specific PSA
 - i) One sorted by Risk Achievement Worth (RAW)
 - ii) The other sorted by Birnbaum Importance
 - b. A list of the top 500 cut-sets
2. From your most-recent PSA including external events and fires:
 - a. Two risk rankings of components from your site-specific PSA
 - i) one sorted by RAW
 - ii) the other sorted by Birnbaum Importance
 - b. A list of the top 500 cut-sets
3. Risk ranking of operator actions from your site specific PSA sorted by RAW. Provide human reliability worksheets for these items.
4. List of time-critical operator actions with a brief description of each action.
5. List of Emergency and Abnormal Operating Procedures revised (significant) since April 1, 2010 with a brief description of each revision.
6. List of components with low design margins (e.g., pumps closest to the design limit for flow or pressure, diesel generator close to design required output, heat exchangers close to rated design heat removal, motor operated valve risk-margin rankings, etc.) and associated margin evaluations, or margin calculations.
7. List of station operating experience evaluations/reviews performed and documented in the station’s corrective action program for industry events and safety related equipment failures/vulnerabilities [as communicated by NRC generic communications, industry communications, 10 CFR part 21 Notifications, etc.] since June 1, 2012.
8. List and brief description of safety related SSC design modifications implemented since June 1, 2012.
9. List and brief description of common-cause component failures that have occurred since June 1, 2012.

Enclosure

10. List and brief description of operability evaluations completed since June 1, 2012.
11. List of equipment on the site's Station Equipment Reliability Issues List, including a description of the reason(s) why each component is on that list and summaries (if available) of your plans to address the issue(s).
12. List and brief description of equipment currently in degraded or nonconforming status as described in NRC Inspection Manual Chapter 0326, Operability Determinations and Functionality Assessments for Conditions Adverse to Quality or Safety, issued January 31, 2014.
13. List and reason for equipment classified in maintenance rule (a)(1) status since June 1, 2012.
14. Copies of System Descriptions (or the like design basis documents) for Safety-Related Systems.
15. Copy of the current UFSAR(s).
16. Copy of Technical Specification(s).
17. Copy of Technical Specifications Bases.
18. Copy of Technical Requirements Manual(s).
19. List and brief description of Root Cause Evaluations that have been performed since June 1, 2012.
20. In-service Testing Program Procedure(s).
21. Corrective Action Program Procedure(s).
22. Electrical one line diagrams of plant with Index and legend (electronic and full size – hard copy week of August 31).
23. Primary AC calculation(s) for safety-related buses.
24. Primary DC calculation(s) for safety-related buses.
25. Piping and instrumentation diagrams (P&IDs) for safety-related systems with Index and legend (electronic and 1/2 size – hard copy week of August 31).
26. Copy of any self-assessments performed in preparation for this inspection and a copy of any prior self-assessments performed in preparation for the 2012 CDBI inspection.
27. Copy of Operability determination procedure(s).
28. Copies of condition reports generated from previous CDBI (2012).

29. Index (procedure number, titles, and current revision) of station Emergency Operating Procedures (EOPs), Abnormal Operating Procedures (AOPs), and Annunciator Response Procedures (ARPs).
30. Contact information for a person to discuss PRA information prior to the information-gathering trip (name, title, phone number, and e-mail address).
- 31 Copy of the Quality Assurance Program Manual

LIST OF ACRONYMS

AC	Alternating Current
AOP	Abnormal Operating Procedure
ARP	Annunciator Response Procedure
CDBI	Component Design Basis Inspection
DC	Direct Current
ECCS	Emergency Core Cooling System
EOP	Emergency Operating Procedure
FTP	File Transfer Protocol
NRC	Nuclear Regulatory Commission
OE	Operating Experience
P&ID	Piping and Instrumentation Diagram
PRA	Probabilistic Risk Assessment
PSA	Probabilistic Safety Assessment
RAW	Risk Achievement Worth
RIS	Regulatory Issue Summary
SSC	Structure, System or Component
UFSAR	Updated Final Safety Analysis Report