

May 7, 2015

MEMORANDUM TO: Nathan Sanfilippo, Chief
Performance Assessment Branch
Division of Inspection and Regional Support
Office of Nuclear Reactor Regulation

FROM: Andrew Waugh, Reactor Operations Engineer */RA/*
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SUBJECT: SUMMARY OF THE REACTOR OVERSIGHT PROCESS WORKING
GROUP PUBLIC MEETING HELD ON APRIL 16, 2015

On April 16, 2015, the U.S. Nuclear Regulatory Commission (NRC) staff hosted the Reactor Oversight Process (ROP) Working Group (WG) public meeting with the Nuclear Energy Institute (NEI) ROP Task Force and other industry representatives. Enclosure 1 contains the meeting attendance list; and Enclosure 2 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML15127A244) contains the Frequently Asked Questions (FAQs) Log and the FAQs discussed during the meeting. Meeting attendees discussed topics related to assessment and performance indicators (PIs).

In the area of the PI program, staff and industry addressed the following items:

- (1) Industry representatives encouraged the staff to update the "Archived FAQs" file available on the NRC's public website. The staff indicated that it will work with NEI on this matter and expects to update the corresponding file in the public website by the May ROP public meeting.
- (2) Industry inquired about the status of the "Exclusion of Low Risk Trains/ Segments from Unavailability Monitoring" white paper discussed in the previous ROP public meeting (ADAMS Accession No. ML15103A245). The staff indicated that there was no change in status from the previous meeting, but that it expects to provide an update during the next ROP public meeting.

Staff and industry discussed the following PI FAQs (see Enclosure 2):

- FAQ 14-08: This is a site-specific FAQ for Prairie Island. This FAQ addresses a diesel generator failure counted under the Emergency AC MSPI. The licensee proposes to retract a diesel generator failure because it resulted from a reverse power condition

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that would not occur while the diesel generator is performing its monitored function and it is not indicative of the reliability of the equipment. The staff acknowledged that this FAQ was made final in the previous ROP meeting.

- FAQ 14-10: This is a site-specific FAQ for Indian Point Units 2 and 3. This FAQ requests guidance on the data reporting for the Protected Area Equipment PI. The licensee proposes to maintain two separate indicators for Unit 2 and Unit 3, but to use the same value for both Units starting early 2015. The NRC staff reviewed the information provided by the licensee and provided comments on the licensee's approach. The staff is aligned with the licensee's proposed treatment for future Protected Area Equipment PI data at this site. The staff acknowledged that this FAQ was made final in the previous ROP meeting.
- FAQ 15-01: This is a site-specific FAQ for Perry. During an unplanned scram, the Full Core Display indicated dual rod indication, both red and green lights for Rods Out and In, respectively. Operators utilized a second indication on the Rod Action Control System to verify rods were indeed inserted before proceeding with the normal scram response procedure. A question about the NEI guidance existed whether this should then count as a complicated scram (IE04). The staff agreed with the licensee that in this particular case, the complicated scram PI is not applicable because the operators were able to verify the condition of the reactor without exiting the normal scram response procedure. NEI provided an update to NEI 99-02 incorporating guidance about the use of alternative indications to clarify control rods status. The NRC staff agreed with the proposed guidance and expects to provide a response during the next ROP public meeting to make this FAQ final. This FAQ is tentative final.
- FAQ 15-01: This is a site-specific FAQ for Perry. During an unplanned scram, the Full Core Display indicated dual rod indication, both red and green lights for Rods Out and In, respectively. Operators utilized a second indication on the Rod Action Control System to verify rods were indeed inserted before proceeding with the normal scram response procedure. A question about the NEI guidance existed whether this should then count as a complicated scram (IE04). The staff agreed with the licensee that in this particular case, the complicated scram PI is not applicable because the operators were able to verify the condition of the reactor without exiting the normal scram response procedure. NEI provided an update to NEI 99-02 incorporating guidance about the use of alternative indications to clarify control rods status. The NRC staff agreed with

the proposed guidance and expects to provide a response during the next ROP public meeting to make this FAQ final. This FAQ is tentative final.

The next ROP WG public meeting is scheduled to be held on May 20, 2015.

Enclosures:

Attendance List – April 16, 2015
Reactor Oversight Process Task
Force FAQ Log – April 16, 2015

the proposed guidance and expects to provide a response during the next ROP public meeting to make this FAQ final. This FAQ is tentative final.

The next ROP WG public meeting is scheduled to be held on May 20, 2015.

Enclosures:

Attendance List – April 16, 2015
 Reactor Oversight Process Task
 Force FAQ Log – April 16, 2015

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***concurred via email**

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DATE	5/07/15	5/06/15	5/7/15

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**REACTOR OVERSIGHT PROCESS PUBLIC MEETING
ATTENDANCE LIST
April 16, 2015**

Nathan Sanfilippo	NRC
Andrew Waugh	NRC
Luis Cruz	NRC
Ami Patel	NRC
Joanna Bridge	NRC
Steve Roberts	NRC
Jim Slider	NEI
Larry Parker	STARS Alliance
Tony Zimmerman	Duke Energy
Suzanne Leblang	Entergy
Lenny Sueper	Xcel Energy
Robin Ritzman	First Energy
Shirelle Allen	Duke Energy
Ken Heffner	Certrec
Steve Catron	NextEra Energy
Adrienne Driver	Duke Energy
Bill Ketchum	Wolf Creek
Dave Kline	
Marvin Lewis	