

## **NRR-PMDAPEm Resource**

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**From:** Purnell, Blake  
**Sent:** Thursday, May 07, 2015 9:13 AM  
**To:** Lisa Simpson (Lisa.Simpson@exeloncorp.com)  
**Cc:** David Gullott (David.Gullott@exeloncorp.com); Tate, Travis  
**Subject:** LaSalle County Station, Units 1 and 2 - Request for Additional Information Regarding Relief Request I3R-14 (TAC Nos. MF5654 and MF5655)  
**Attachments:** LaSalle RR I3R-14 RAI.docx

Ms. Lisa Simpson:

On January 29, 2015, Exelon Generation Company, LLC (the licensee) submitted relief request (RR) I3R-14 for LaSalle County Station (LSCS), Units 1 and 2 (Agencywide Documents Access and Management System Accession No. ML15030A175). RR I3R-14 is intended to apply American Society of Mechanical Engineers Boiler and Pressure Vessel Code, Code Case N-702, "Alternative Requirements for Boiling Water Reactor (BWR) Nozzle Inner Radius and Nozzle-to-Shell Welds," to reactor pressure vessel nozzles at LSCS.

The U.S. Nuclear Regulatory Commission staff has reviewed the RR and determined that it needs additional information to complete its review. The licensee is requested to respond to the attached request for additional information within 30 days of this email. If you have any questions about this request, contact me at (301) 415-1380.

Sincerely,

Blake Purnell, Project Manager  
Plant Licensing Branch III-2 and  
Planning and Analysis Branch  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation  
U.S. Nuclear Regulatory Commission

Docket Nos. 50-373 and 50-374

**Hearing Identifier:** NRR\_PMDA  
**Email Number:** 2046

**Mail Envelope Properties** (Blake.Purnell@nrc.gov20150507091300)

**Subject:** LaSalle County Station, Units 1 and 2 - Request for Additional Information  
Regarding Relief Request I3R-14 (TAC Nos. MF5654 and MF5655)  
**Sent Date:** 5/7/2015 9:13:12 AM  
**Received Date:** 5/7/2015 9:13:00 AM  
**From:** Purnell, Blake

**Created By:** Blake.Purnell@nrc.gov

**Recipients:**

"David Gullott (David.Gullott@exeloncorp.com)" <David.Gullott@exeloncorp.com>

Tracking Status: None

"Tate, Travis" <Travis.Tate@nrc.gov>

Tracking Status: None

"Lisa Simpson (Lisa.Simpson@exeloncorp.com)" <Lisa.Simpson@exeloncorp.com>

Tracking Status: None

**Post Office:**

<b>Files</b>	<b>Size</b>	<b>Date &amp; Time</b>
MESSAGE	1174	5/7/2015 9:13:00 AM
LaSalle RR I3R-14 RAI.docx	23806	

**Options**

**Priority:** Standard

**Return Notification:** No

**Reply Requested:** No

**Sensitivity:** Normal

**Expiration Date:**

**Recipients Received:**

## REQUEST FOR ADDITIONAL INFORMATION

### RELIEF REQUESTS I3R-14

### LASALLE COUNTY STATION, UNITS 1 AND 2

### TAC NOS. MF5654 AND MF5655

On January 29, 2015, Exelon Generation Company, LLC (the licensee) submitted relief request (RR) I3R-14 for LaSalle County Station (LSCS), Units 1 and 2 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML15030A175). RR I3R-14 is intended to apply American Society of Mechanical Engineers (ASME) Boiler and Pressure Vessel Code (ASME Code), Code Case N-702, "Alternative Requirements for Boiling Water Reactor (BWR) Nozzle Inner Radius and Nozzle-to-Shell Welds," to reactor pressure vessel nozzles at LSCS.

The U.S. Nuclear Regulatory Commission (NRC) staff has reviewed the RR and determined that it needs additional information to complete its review.

#### **Background**

By letter dated April 19, 2013 (ADAMS Accession No. ML13071A240), the NRC staff approved the Boiling Water Reactor Vessel Internals Project (BWRVIP) topical report, BWRVIP-241, "Fracture Mechanics Evaluation for the Boiling Water Reactor Nozzle-to-Vessel Shell Welds and Nozzle Blend Radii," submitted on April 26, 2011 (ADAMS Accession No. ML11119A041). The staff's safety evaluation (SE) states that BWRVIP-241 may be referenced as the technical basis for the use of ASME Code Case N-702 as an alternative. The staff's SE further states that the licensee should address the conditions and limitations specified in the SE to demonstrate that BWRVIP-241 is applicable to the plant.

The RR states that the recirculating outlet nozzles for LSCS Unit 2 did not meet Condition 4 of the SE. Section 6, "Summary and Conclusions," of BWRVIP-241 states:

For plants having recirculation outlet nozzles with Condition 4 greater than 1.15 [i.e., does not meet Condition 4 of SE], a plant specific analysis following the approach described in this report may be able to justify values greater than 1.15. The analysis would need to be submitted to the NRC as part of a relief request to implement Code Case N-702 for the subject nozzles.

The RR does not provide an analysis to justify a value for Condition 4 greater than 1.15, but instead provides only the summary and results of the analysis. The RR indicates that the analysis is contained in Design Analysis L-003976, "Probability of Failure Analysis for Reactor Pressure Vessel Nozzles."

#### **Request**

Provide a plant-specific analysis for the recirculating outlet nozzles for LSCS Unit 2 to justify a value for Condition 4 greater than 1.15.