



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

June 5, 2015

Mr. Joseph W. Shea
Vice President, Nuclear Licensing
Tennessee Valley Authority
1101 Market Street, LP 3R-C
Chattanooga, TN 37402-2801

SUBJECT: SEQUOYAH NUCLEAR PLANT, UNITS 1 AND 2 – CORRECTIONS TO AMENDMENT NOS. 312 (TECHNICAL SPECIFICATION (TS)-05-08) (TAC NO. MC9774), 306 (TS-05-07) (TAC NO. MC8243), 193 (TS-94-17) (TAC NO. M90781), AND 37 FOR UNIT 1, AND AMENDMENT NOS. 290 (TS-03-13) (TAC NO. MC5214) AND 170 (TS-94-02) (TAC NO. M88735) FOR UNIT 2 (TAC NOS. MD7153 AND MD7154)

Dear Mr. Shea:

The U.S. Nuclear Regulatory Commission (NRC) issued Amendment Nos. 312, 306, 193, and 37 to Facility Operating License No. DPR-77 on October 4, 2006; February 23, 2006; January 3, 1995; and January 24, 1985, respectively, for Sequoyah Nuclear Plant (SQN), Unit 1. In addition, the NRC issued Amendment Nos. 290 and 170 to Facility Operating License No. DPR-79 on April 11, 2005, and March 31, 1994, respectively, for SQN, Unit 2. After these issuances, it came to NRC staff attention that these amendments contained typographical errors introduced inadvertently that were neither considered during NRC review nor addressed in the notice to the public. Therefore, the enclosed pages have been corrected in accordance with the associated approved amendments. Please replace the incorrect pages with the correct pages. A brief description of each amendment and the associated typographical errors are provided below.

1. The NRC issued Amendment No. 312 (Agencywide Documents Access and Management System Accession No. ML061590287) to Facility Operating License No. DPR-77 for SQN, Unit 1 in response to a licensee request dated February 6, 2006 (Technical Specification (TS)-05-08) regarding modifications to the TS requirements for inoperable snubbers by adding Limiting Condition for Operation 3.0.7. This operating license improvement was made available by the NRC on May 4, 2005 (70 *Federal Register* 23252) as part of the consolidated line item improvement process and is consistent with the NRC-approved Technical Specification Task Force (TSTF) Change TSTF-372, Revision 4, "The Addition of Limiting Condition for Operation (LCO) 3.0.8 on the Inoperability of Snubbers."

Amendment No. 312 TS pages reflect all changes discussed in the NRC safety evaluation; however, previously approved changes made by Amendment No. 308 (ADAMS Accession No. ML061360043) to pages 3/4 0-3 and 3/4 0-4 were not included. The corrected pages in Enclosure 1 for Amendment No. 312 to DPR-77 include the approved changes made by Amendment No. 308.

2. The NRC issued Amendment No. 306 (ADAMS Accession No. ML060120099) to Facility Operating License No. DPR-77 for SQN, Unit 1 in response to a licensee request dated August 31, 2005 (TS-05 07). The amendment revises the TSs associated with steam generator tube integrity consistent with Revision 4 to TSTF Standard TS Change Traveler, TSTF-449, "Steam Generator Tube Integrity." A notice of availability for this TS improvement using the consolidated line item improvement process was published in the *Federal Register* on May 6, 2005 (70 FR 24126).

Amendment No. 306 TS page 1-5 reflects changes described in the NRC safety evaluation; however, the numbering for the item "PRESSURE BOUNDARY LEAKAGE" is not correct. This error was neither considered during NRC review nor addressed in the notice to the public, and therefore, is corrected in Enclosure 2.

3. The NRC issued Amendment No. 193 (ADAMS Accession No. ML013320071) to Facility Operating License No. DPR-77 for SQN, Unit 1 in response to a licensee request dated November 2, 1994. The amendment added Operating License Condition 2.C.(25) to provide a limited extension of the surveillance test intervals for certain specified instrumentation on Unit 1 to coincide with the Cycle 7 refueling outage.

Amendment No. 193 amended page 12b of Facility Operating License No. DPR-77; however, the statement, "Additional exemptions are listed in Attachment 1," was inadvertently included. This statement was neither considered during NRC review nor addressed in the notice to the public, and therefore, should be removed. Page 12b has been amended by Amendment Nos. 10, 53, 73, 193, 200, 213, 223, 292, 329, 330, and 333. Approved amendment pages up to, and including, Amendment No. 73 (Legacy Library Accession No. 8807070556) do not include the sentence, "Additional exemptions are listed in Attachment 1," while amendments including and after Amendment No. 193 do include this sentence. Therefore, this correction letter corrects all amendments that inadvertently included the above sentence on page 12b after, and including, Amendment No. 193. A corrected page 12b is found in Enclosure 3.

4. The NRC issued Amendment No. 37 (ADAMS Accession No. ML013250179) to Facility Operating License No. DPR-77 for SQN, Unit 1 in response to licensee requests dated July 21 and July 26, 1983; and August 20, August 27, and August 28, 1984, regarding changes to the TSs related to containment isolation valves, vital batteries, fire detectors, and steam generator low-low level instrumentation.

Amendment No. 37, TS page 3/4 8-11 includes a typographical error in which a "V" was inadvertently deleted in the page footnote. This error was neither considered during NRC review nor addressed in the notice to the public, and therefore, is corrected in Enclosure 4.

5. The NRC issued Amendment No. 290 (ADAMS Accession No. ML050700397) to Facility Operating License No. DPR-79 for SQN, Unit 2 in response to a licensee request dated December 2, 2004 (TS-03-13). The amendment modified the TS requirements for mode change limitations in Limiting Condition for Operation 3.0.4 and Surveillance Requirement 4.0.4, consistent with the TSTF Standard TS Change Traveler, TSTF-359, Revision 9, "Increased Flexibility in Mode Restraints." A notice of availability for this TS improvement using the Consolidated Line Item Improvement Process was published in the *Federal Register* (FR) on April 4, 2003 (68 FR 16579).

Amendment No. 290 TS page 3/4 3-19a includes a typographical error in which the number listed under the action column for row ii.a of Table 3.3-3 was inadvertently moved up one row. This error was neither considered during NRC review nor addressed in the notice to the public, and therefore, is corrected in Enclosure 5.

6. The NRC issued Amendment No. 170 (ML013320583) to Facility Operating License No. DPR-79 for SQN, Unit 2 in response to a licensee request dated September 8, 1993. The amendment added Operating License Condition 2.C.(17) to provide a limited extension of the surveillance test intervals for certain specified instrumentation on Unit 2 to coincide with the Cycle 5 refueling outage that was scheduled to start in July 1994.

Amendment No. 170 amended page 12a of the DPR-79; however, the statement, "Additional exemptions are listed in Attachment 2," was inadvertently included. This statement was neither considered during NRC review nor addressed in the notice to the public, and therefore, should be removed. Page 12a has been amended by Amendment Nos. 2, 65, 162, 170, 204, 214, 267, 273, 282, 322, and 326. Approved amendment pages up to, and including, Amendment No. 65 (Legacy Library Accession No. 8807070556) do not include the sentence, "Additional exemptions are listed in Attachment 2," while amendments including, and after, Amendment No. 170 do include this sentence. Therefore, this correction letter corrects all amendments that inadvertently included the above sentence on page 12a after, and including, Amendment No. 170. A corrected page 12a is found in Enclosure 6.

J. Shea

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We regret any inconvenience these errors may have caused. If you have any questions, please contact me at (301) 415-8480 or andrew.hon@nrc.gov.

Sincerely,



Andrew Hon, Project Manager
Plant Licensing Branch II-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket Nos. 50-327 and 50-328

Enclosures:

1. Corrected Pages for Amendment No. 312 to DPR-77
2. Corrected Page for Amendment No. 306 to DPR-77
3. Corrected Page 12b that includes Amendment No. 193 to DPR-77
4. Corrected Page for Amendment No. 37 to DPR-77
5. Corrected Page for Amendment No. 293 to DPR-79
6. Corrected Page 12a that includes Amendment No. 170 to DPR-79

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ENCLOSURE 1

Corrected Pages for Amendment No. 312 to DPR-77

SURVEILLANCE REQUIREMENTS (Continued)

4.0.3 (Continued)

If the Surveillance is not performed within the delay period, the Limiting Condition for Operation must immediately be declared not met, and the applicable ACTION(s) must be entered. When the Surveillance is performed within the delay period and the Surveillance is not met, the Limiting Condition for Operation must immediately be declared not met, and the applicable ACTION(s) must be entered.

4.0.4 Entry into a MODE or other specified condition in the Applicability of an LCO shall only be made when the LCO's Surveillances have been met within their specified Frequency, except as provided by SR 4.0.3. When an LCO is not met due to Surveillances not having been met, entry into a MODE or other specified condition in the Applicability shall only be made in accordance with LCO 3.0.4.

This provision shall not prevent entry into MODES or other specified conditions in the Applicability that are required to comply with ACTIONS or that are part of a shutdown of the unit.

4.0.5 Surveillance Requirements for inservice inspection and testing of ASME Code Class 1, 2 and 3 components shall be as follows:

Inservice Inspection Program

This program provides controls for inservice inspection of ASME Code Class 1, 2, and 3 components, including applicable supports. The program shall include the following:

- a. Provisions that inservice testing of ASME Code Class 1, 2 and 3 components shall be performed in accordance with Section XI of the ASME Boiler and Pressure Vessel Code and applicable Addenda as required by 10 CFR 50.55a;
- b. The provisions of SR 4.0.2 are applicable to the frequencies for performing inservice inspection activities;
- c. Inspection of each reactor coolant pump flywheel per the recommendation of Regulation Position c.4.b of Regulatory Guide 1.14, Revision 1, August 1975 or in lieu of Position c.4.b(1) and c.4.b(2), a qualified in-place ultrasonic examination over the volume from the inner bore of the flywheel to the circle one-half of the outer radius or a surface examination (magnetic particle and/or liquid penetrant) of exposed surfaces of the removed flywheels may be conducted at 20-year intervals (the provisions of SR 4.0.2 are not applicable); and
- d. Nothing in the ASME Boiler and Pressure Vessel Code shall be construed to supersede the requirement of any TS.

Inservice Testing Program

This program provides controls for inservice testing of ASME Code Class 1, 2, and 3 components including applicable supports. The program shall include the following:

- a. Provisions that inservice testing of ASME Code Class 1, 2, and 3 pumps and valves shall be performed in accordance with the ASME Code for Operation and Maintenance of Nuclear Power Plants (ASME OM Code) and applicable Addenda as required by 10 CFR 50.55a;

APPLICABILITY

SURVEILLANCE REQUIREMENTS (Continued)

4.0.5 (Continued)

b. Testing Frequencies applicable to the ASME OM Code and applicable Addenda as follows:

ASME OM Code and applicable Addenda terminology for inservice testing activities	Required frequencies for performing inservice testing activities
Weekly	At least once per 7 days
Monthly	At least once per 31 days
Quarterly or every 3 months	At least once per 92 days
Semiannually or every 6 months	At least once per 184 days
Every 9 months	At least once per 276 days
Yearly or annually	At least once per 366 days
Biennially or every 2 years	At least once per 731 days

c. The provisions of SR 4.0.2 are applicable to the above required Frequencies and other normal and accelerated frequencies specified as 2 years or less in the Inservice Testing Program for performing inservice testing activities;

d. The provisions of SR 4.0.3 are applicable to inservice testing and activities; and

e. Nothing in the ASME OM Code shall be construed to supersede the requirements of any TS.

ENCLOSURE 2

Corrected Page for Amendment No. 306 to DPR-77

PRESSURE BOUNDARY LEAKAGE

1.22 PRESSURE BOUNDARY LEAKAGE shall be leakage (except primary to secondary leakage) through a non-isolable fault in a Reactor Coolant System component body, pipe wall or vessel wall.

PRESSURE AND TEMPERATURE LIMITS REPORT (PTLR)

1.23 The PTLR is the unit specific document that provides the reactor vessel pressure and temperature limits, including heatup and cooldown rates and the LTOP arming temperature, for the current reactor vessel fluence period. These pressure and temperature limits shall be determined for each fluence period in accordance with Specification 6.9.1.15.

PROCESS CONTROL PROGRAM (PCP)

1.24 DELETED

PURGE - PURGING

1.25 PURGE or PURGING is the controlled process of discharging air or gas from a confinement to maintain temperature, pressure, humidity, concentration or other operating condition, in such a manner that replacement air or gas is required to purify the confinement.

QUADRANT POWER TILT RATIO

1.26 QUADRANT POWER TILT RATIO shall be the ratio of the maximum upper excore detector calibrated output to the average of the upper excore detector calibrated outputs, or the ratio of the maximum lower excore detector calibrated output to the average of the lower excore detector calibrated outputs, whichever is greater.

RATED THERMAL POWER (RTP)

1.27 RATED THERMAL POWER (RTP) shall be a total reactor core heat transfer rate to the reactor coolant of 3455 MWt.

REACTOR TRIP SYSTEM (RTS) RESPONSE TIME

1.28 The REACTOR TRIP SYSTEM RESPONSE TIME shall be the time interval from when the monitored parameter exceeds its (RTS) trip setpoint at the channel sensor until loss of stationary gripper coil voltage. The response time may be measured by means of any series of sequential, overlapping, or total steps so that the entire response time is measured. In lieu of measurement, response time may be verified for selected components provided that the components and the methodology for verification have been previously reviewed and approved by NRC.

REPORTABLE EVENT

1.29 DELETED

ENCLOSURE 3

Corrected Page 12b that includes Amendment No. 193 to DPR-77

D. Exemptions from certain requirements of Appendices G and J to 10 CFR Part 50 are described in the Office of Nuclear Reactor Regulation's Safety Evaluation Report, Supplements No. 1. These exemptions are authorized by law and will not endanger life or property or the common defense and security and are otherwise in the public interest. The exemptions are, therefore, hereby granted. The granting of these exemptions are authorized with the issuance of the License for Fuel Loading and Low Power Testing, dated February 29, 1980. The facility will operate, to the extent authorized herein, Act, and the regulations of the Commission.

E. Physical Protection

- (1) The licensee shall fully implement and maintain in effect all provisions of the Commission- approved physical security, training and qualification, and safeguards contingency plans including amendments made pursuant to provisions of the Miscellaneous Amendments and Search Requirements revisions to 10 CFR 73.55 (51 FR 27817 and 27822) and to the authority of 10 CFR 50.90 and 10 CFR 50.54(p). The combined set of plans, which contain Safeguards Information protected under 10 CFR 73.21, is entitled: "Sequoyah Nuclear Plant Security Plan, Training And Qualification Plan, And Safeguards Contingency Plan" submitted by letter dated May 8, 2006.
- (2) The licensee shall fully implement and maintain in effect all provisions of the Commission-approved cyber security plan (CSP), including changes made pursuant to the authority of 10 CFR 50.90 and 10 CFR 50.54(p). The licensee CSP was approved by License Amendment No. 329, as amended by changes approved by License Amendment No. 333.

ENCLOSURE 4

Corrected Page for Amendment No. 37 to DPR-77

ELECTRICAL POWER SYSTEMS

D.C. DISTRIBUTION - OPERATING

LIMITING CONDITION FOR OPERATION

3.8.2.3 The following D.C. vital battery channels shall be energized and OPERABLE:

- CHANNEL I Consisting of 125 - volt D.C. board No. I, 125 - volt D.C. battery bank No. I* and a full capacity charger.
- CHANNEL II Consisting of 125 - volt D.C. board No. II, 125 - volt D.C. battery bank No. II*, and a full capacity charger.
- CHANNEL III Consisting of 125 - volt D.C. board No. III, 125 - volt D.C. battery bank No. III*, and a full capacity charger.
- CHANNEL IV Consisting of 125 - volt D.C. board No. IV, 125 - volt D.C. battery bank No. IV*, and a full capacity charger.

APPLICABILITY: MODES 1, 2, 3 and 4.

ACTION:

- a. With one 125-volt D.C. board inoperable, restore the inoperable board to OPERABLE status within 2 hours or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.
- b. With one 125-volt D.C. battery bank and/or its charger inoperable, restore the inoperable battery bank and/or charger to OPERABLE status within 2 hours or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

*D.C. Battery Bank V may be substituted for any other Battery Bank as needed.

ENCLOSURE 5

Corrected Page for Amendment No. 290 to DPR-79

TABLE 3.3-3 (Continued)

ENGINEERED SAFETY FEATURE ACTUATION SYSTEM INSTRUMENTATION

<u>FUNCTIONAL UNIT</u>	<u>TOTAL NO. OF CHANNELS</u>	<u>CHANNELS TO TRIP</u>	<u>MINIMUM CHANNELS OPERABLE</u>	<u>APPLICABLE MODES</u>	<u>ACTION</u>
d. Containment Pressure (EAM)	4	2	3	1, 2, 3	38
ii. Start Turbine Driven Pump					
a. Steam Gen. Water Level--Low-Low (Adverse)	3/Strm. Gen.	2/Strm. Gen. in any 2 operating Strm. Gen.	2/Strm. Gen. in each operating Strm. Gen.	1, 2, 3	36
b. Steam Gen. Water Level--Low-Low (EAM)	3/Strm. Gen.	2/Strm. Gen. in any 2 operating Strm. Gen.	2/Strm. Gen. in each operating Strm. Gen.	1, 2, 3	36
c. RCS Loop ΔT	4(1/loop)	2	3	1, 2, 3	37
d. Containment Pressure (EAM)	4	2	3	1, 2, 3	38
d. S. I. Start Motor-Driven Pumps and Turbine Driven Pump	See 1 above (all S.I. initiating functions and requirements)				

ENCLOSURE 6

Corrected Page 12a that includes Amendment No. 170 to DPR-79

A temporary exemption from General Design Criterion 57 found in Appendix A to 10 CFR part 50 is described in the Office of Nuclear Reactor Regulation's Safety Evaluation Report, Supplement No. 5, Section 6.2.4. This exemption is authorized by law and will not endanger life or property or the common defense and security and is otherwise in the public interest. The exemption, therefore, is hereby granted and shall remain in effect through the first refueling outage as discussed in Section 6.2.4 of Supplement 5 to the Safety Evaluation Report. The granting of the exemption is authorized with the issuance of the Facility Operating License. The facility will operate, to the extent authorized herein, in conformity with the application as amended, the provisions of the Act, and the regulations of the Commission.

E. Physical Protection

- (1) The licensee shall fully implement and maintain in effect all provisions of the Commission-approved physical security, training and qualification, and safeguards contingency plans including amendments made pursuant to provisions of the Miscellaneous Amendments and Search Requirements revisions to 10 CFR 73.55 (51 FR 27817 and 27822) and to the authority of 10 CFR 50.90 and 10 CFR 50.54(p). The combined set of plans, which contain Safeguards Information protected under 10 CFR 73.21, is entitled: "Sequoyah Nuclear Plant Security Plan, Training And Qualification Plan, And Safeguards Contingency Plan" submitted by letter dated May 8, 2006.
- (2) The licensee shall fully implement and maintain in effect all provisions of the Commission-approved cyber security plan (CSP), including changes made pursuant to the authority of 10 CFR 50.90 and 10 CFR 50.54(p). The licensee CSP was approved by License Amendment No. 322, as amended by changes approved by License Amendment No. 326.

F. Reactor Safety Methodology Applications Programs (Section 24.0)

TVA will provide a report prepared by the Kaman Sciences Corporation (KSC) on a full scale nuclear safety and availability analysis within six months from the date of the KSC report.

- G. This amended license is subject to the following additional condition for the protection of the environment:

Before engaging in additional construction or operational activities which may result in an environmental impact that was not evaluated by the Commission, Tennessee Valley Authority will prepare and record an environmental evaluation of such activity. When the evaluation indicates that such activity may result in a significant adverse environmental impact that was not evaluated, or that is significantly greater than that evaluated in the Final Environmental Statement prepared by the Tennessee Valley Authority and the Environmental Impact Appraisal prepared by the Commission in May 1979, the Tennessee Valley Authority shall provide a written evaluation of such activities and obtain prior approval from the Director, Office of Nuclear Reactor Regulation.

H. Deleted

- I. TVA shall immediately notify the Commission of any accident at this facility which could result in an unplanned release of quantities of fission products in excess of allowable limits for normal operation established by the Commission.

J. Shea

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We regret any inconvenience these errors may have caused. If you have any questions, please contact me at (301) 415-8480 or andrew.hon@nrc.gov.

Sincerely,

/RA/

Andrew Hon, Project Manager
Plant Licensing Branch II-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

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***w/comment**

OFFICE	NRR/DORL/LPL2-2/PM	NRR/DORL/LPL2-2/PM	NRR/DORL/LPL2-2/LAit	NRR/DORL/LPL2-2/LA
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DATE	6/5/15	5/12/15	5/12/15	5/12/15
OFFICE	NRR/DSS/STSB/BC*	NRR/DORL/LPL2-2/BC	NRR/DORL/LPL2-2/PM	
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