

RS-15-124

10 CFR 50.46

May 1, 2015

U.S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, DC 20555-0001

Quad Cities Nuclear Power Station, Units 1 and 2
Renewed Facility Operating License Nos. DPR-29 and DPR-30
NRC Docket Nos. 50-254 and 50-265

Subject: 10 CFR 50.46 Annual Report

Reference: Letter from P. R. Simpson (Exelon Generation Company, LLC) to U.S. NRC,
"10 CFR 50.46 Annual Report," dated May 2, 2014

This letter provides the annual report required by 10 CFR 50.46, "Acceptance criteria for emergency core cooling systems for light-water nuclear power reactors," for Quad Cities Nuclear Power Station (QCNPS), Units 1 and 2. The attachments describe the changes in accumulated peak cladding temperature (PCT) since the previous annual report submitted in the referenced letter.

There are no regulatory commitments contained in this letter. Should you have any questions concerning this letter, please contact Mr. Kenneth M. Nicely at (630) 657-2803.

Respectfully,



Patrick R. Simpson
Manager – Licensing

Attachments:

1. Quad Cities Nuclear Power Station Unit 1, 10 CFR 50.46 Report (Westinghouse Fuel)
2. Quad Cities Nuclear Power Station Unit 2, 10 CFR 50.46 Report (Westinghouse Fuel)
3. Quad Cities Nuclear Power Station Units 1 and 2, 10 CFR 50.46 Report Assessment Notes

cc: NRC Regional Administrator, Region III
NRC Senior Resident Inspector, Quad Cities Nuclear Power Station

ATTACHMENT 1
Quad Cities Nuclear Power Station Unit 1,
10 CFR 50.46 Report (Westinghouse Fuel)

PLANT NAME: Quad Cities Unit 1
 ECCS EVALUATION MODEL: USA5
 REPORT REVISION DATE: 05/01/2015
 CURRENT OPERATING CYCLE: 24

ANALYSIS OF RECORD

Evaluation Model: "Westinghouse BWR ECCS Evaluation Model: Supplement 3 to Code Description, Qualification and Application to SVEA-96 Optima2 Fuel," WCAP-16078-P-A, November 2004

Calculations: "Quad Cities 1 & 2 LOCA Analysis for SVEA-96 Optima2 Fuel," OPTIMA2-TR021QC-LOCA, Revision 5, Westinghouse Electric Company, LLC, September 2009

Fuel Analyzed in Calculation: SVEA-96 Optima2

Limiting Fuel Type: SVEA-96 Optima2

Limiting Single Failure: Low Pressure Coolant Injection System injection valve

Limiting Break Size and Location: 1.0 double-ended guillotine break in the recirculation pump suction line

Reference Peak Cladding Temperature (PCT): PCT = 2150°F

MARGIN ALLOCATION

A. PRIOR LOCA MODEL ASSESSMENTS

10 CFR 50.46 Report dated May 7, 2010 (See Note 1)	$\Delta PCT = 11^{\circ}F$
10 CFR 50.46 Report dated May 6, 2011 (See Note 2)	$\Delta PCT = 0^{\circ}F$
10 CFR 50.46 Report dated May 4, 2012 (See Note 3)	$\Delta PCT = 18^{\circ}F$
10 CFR 50.46 Report dated May 3, 2013 (See Note 4)	$\Delta PCT = 0^{\circ}F$
10 CFR 50.46 Report dated May 2, 2014 (See Note 5)	$\Delta PCT = 0^{\circ}F$
Net PCT	2179°F

B. CURRENT LOCA MODEL ASSESSMENTS

None (See Note 6)	$\Delta PCT = 0^{\circ}F$
Total PCT change from current assessments	$\Sigma \Delta PCT = 0^{\circ}F$
Cumulative PCT change from current assessments	$\Sigma \Delta PCT = 0^{\circ}F$
Net PCT	2179°F

ATTACHMENT 2
Quad Cities Nuclear Power Station Unit 2,
10 CFR 50.46 Report (Westinghouse Fuel)

PLANT NAME: Quad Cities Unit 2
 ECCS EVALUATION MODEL: USA5
 REPORT REVISION DATE: 05/01/2015
 CURRENT OPERATING CYCLE: 23

ANALYSIS OF RECORD

Evaluation Model: "Westinghouse BWR ECCS Evaluation Model: Supplement 3 to Code Description, Qualification and Application to SVEA-96 Optima2 Fuel," WCAP-16078-P-A, November 2004

Calculations: "Quad Cities 1 & 2 LOCA Analysis for SVEA-96 Optima2 Fuel," OPTIMA2-TR021QC-LOCA, Revision 5, Westinghouse Electric Company, LLC, September 2009

Fuel Analyzed in Calculation: SVEA-96 Optima2

Limiting Fuel Type: SVEA-96 Optima2

Limiting Single Failure: Low Pressure Coolant Injection System injection valve

Limiting Break Size and Location: 1.0 double-ended guillotine break in the recirculation pump suction line

Reference Peak Cladding Temperature (PCT): PCT = 2150°F

MARGIN ALLOCATION

A. PRIOR LOCA MODEL ASSESSMENTS

10 CFR 50.46 Report dated May 7, 2010 (See Note 1)	$\Delta PCT = 11^{\circ}F$
10 CFR 50.46 Report dated May 6, 2011 (See Note 2)	$\Delta PCT = 0^{\circ}F$
10 CFR 50.46 Report dated May 4, 2012 (See Note 3)	$\Delta PCT = 18^{\circ}F$
10 CFR 50.46 Report dated May 3, 2013 (See Note 4)	$\Delta PCT = 0^{\circ}F$
10 CFR 50.46 Report dated May 2, 2014 (See Note 5)	$\Delta PCT = 0^{\circ}F$
Net PCT	2179°F

B. CURRENT LOCA MODEL ASSESSMENTS

None (See Note 6)	$\Delta PCT = 0^{\circ}F$
Total PCT change from current assessments	$\Sigma \Delta PCT = 0^{\circ}F$
Cumulative PCT change from current assessments	$\Sigma \Delta PCT = 0^{\circ}F$
Net PCT	2179°F

ATTACHMENT 3
Quad Cities Nuclear Power Station Units 1 and 2,
10 CFR 50.46 Report Assessment Notes

1. Prior Loss-of-Coolant Accident (LOCA) Assessment

The referenced letter reported a new Westinghouse LOCA analysis of record (AOR), which incorporated modifications for the newly added recirculation pump adjustable speed drives (ASD). The new AOR (Revision 5) updated the LOCA analysis to apply approved updated Westinghouse methods, incorporate previous corrections, and update plant specific inputs. The new Westinghouse LOCA analysis demonstrated that the limiting peak cladding temperature (PCT) was 2150°F. The referenced letter also reported the impact of subsequent corrections for an incorrect bypass hole flow coefficient, and for updated vessel leakage values. The PCT impact of these two corrections on the limiting PCT was 9°F and 2°F, respectively.

[Reference: Letter from J. L. Hansen (Exelon Generation Company, LLC) to U.S. NRC, "10 CFR 50.46, 'Acceptance criteria for emergency core cooling systems for light-water nuclear power reactors,' Annual Report," dated May 7, 2010]

2. Prior LOCA Assessment

The referenced letter reported no new PCT assessment for the Westinghouse LOCA analysis. Also, no emergency core cooling system (ECCS)-related changes or modifications occurred at Quad Cities Nuclear Power Station (QCNPS) that affected the assumptions of the ECCS analyses.

[Reference: Letter from J. L. Hansen (Exelon Generation Company, LLC) to U.S. NRC, "10 CFR 50.46, 'Acceptance criteria for emergency core cooling systems for light-water nuclear power reactors,' Annual Report," dated May 6, 2011]

3. Prior LOCA Assessment

The referenced letter provided the annual 10 CFR 50.46 report for Units 1 and 2. The letter reported errors in the current Westinghouse QCNPS LOCA analysis associated with the use of incorrect R-factors. The impact due to this change was determined to be an 18°F increase in PCT. For 10 CFR 50.46 reporting purposes, the PCT update was conservatively applied to all bundle types including the fresh bundles. This PCT update will remain in effect only until the maximum average planar linear heat generation rate (MAPLHGR) limits for all bundles in future QCNPS Units 1 and 2 cores are evaluated with the correct R-factors.

[Reference: Letter from D. M. Gullott (Exelon Generation Company, LLC) to U.S. NRC, "10 CFR 50.46, 'Acceptance criteria for emergency core cooling systems for light-water nuclear power reactors,' Annual Report," dated May 4, 2012]

ATTACHMENT 3
Quad Cities Nuclear Power Station Units 1 and 2,
10 CFR 50.46 Report Assessment Notes

4. Prior LOCA Assessment

The referenced letter reported no new PCT assessment for the Westinghouse LOCA analysis. Also, no ECCS-related changes or modifications occurred at QCNPS that affected the assumptions of the ECCS analyses.

[Reference: Letter from D. M. Gullott (Exelon Generation Company, LLC) to U.S. NRC, "10 CFR 50.46 Annual Report," dated May 3, 2013]

5. Prior LOCA Assessment

The referenced letter reported no new PCT assessment for the Westinghouse LOCA analysis. Also, no ECCS-related changes or modifications occurred at QCNPS that affected the assumptions of the ECCS analyses.

[Reference: Letter from P. R. Simpson (Exelon Generation Company, LLC) to U.S. NRC, "10 CFR 50.46 Annual Report," dated May 2, 2014]

6. Current Assessment

There are no new changes, error corrections, or enhancements in the current QCNPS LOCA analysis. Also, no ECCS-related changes or modifications occurred at QCNPS that affected the assumptions in the LOCA AOR.