

NOTICE OF VIOLATION

Duke Power Company
Oconee 1, 2, and 3

Docket Nos. 50-269, 270, and 287
License Nos. DPR-38, 47, and 55

During an NRC inspection conducted on May 4 through June 18, 1997, violations of NRC requirements were identified. In accordance with the "General Statement of Policy and Procedures for NRC Enforcement Actions," NUREG 1600, the violations are listed below:

- A. 10 CFR 50, Appendix B, Criterion V, Instructions, Procedures and Drawings, requires activities affecting quality shall be prescribed by documented instructions, procedures, or drawings, of a type appropriate to the circumstances and shall be accomplished in accordance with these instructions, procedures, or drawings.

Contrary to the above, on January 7, 1997, the bolts for the emergency sump cover plates had not been reinstalled prior to plant operation as required by drawings O-67-A and O-67-J. This condition had existed on all three units since approximately 1991.

This is a Severity Level IV violation (Supplement I).

- B. 10 CFR 50, Appendix B, Criterion XVI, Corrective Actions, states that measures shall be established to assure that conditions adverse to quality such as failures, malfunctions, deficiencies, deviations, defective material, and equipment, and non-conformances are promptly identified and corrected. In the case of significant conditions adverse to quality, the measures shall assure that the cause of the condition is determined and corrective action taken to preclude repetition.

Contrary to the above:

1. Corrective actions taken in 1975 with respect to severe pressure boundary wall erosion problems on High Pressure Injection (HPI) pump mini-flow (recirculation) orifice assemblies were inadequate to preclude repetition of the problems. Specifically, in May 1997, the mini-flow orifice assemblies associated with HPI pumps A and B in all three units were discovered to be severely damaged internally by flow induced erosion, requiring replacement or repair, similar to the problems identified in 1975.
2. Corrective actions taken for Violation 50-269,270,287/96-17-09 (October 1996) with respect to returning systems back to service prior to meeting applicable code case requirements were inadequate to preclude repetition. Specifically, on June 6, 1997, the Authorized Nuclear Inservice inspector found that the radiographs of weld 3HP-262-45 on one of the repaired Unit 3 "A" HPI pump mini-flow orifice assemblies did not reflect 100 percent coverage as required under ASME Section XI, Code Case N-416-1, for this Class 2, ANSI B31.7 weld. Similar to the situation in Violation 96-17-09 where Code Case N-416-1 was also invoked for Low Pressure Service Water welds, the affected systems in both situations had

been returned to service prior to meeting the non-destructive examination requirements of the Code Case.

This is a Severity Level IV violation (Supplement I).

- C. Technical Specification 6.4.1 requires that the station shall be operated and maintained in accordance with approved procedures.

PT/2/A/0203/04, Low Pressure System Leakage Test, Enclosure 13.3, Leakage Test of Low Pressure Injection (LPI) Discharge Piping and High Pressure Suction Piping During Unit Startup, is used to identify any leaks in the Unit 2 LPI system. Enclosure 13.3 step 2.3 states "When Steam Generators are controlling Reactor Coolant System (RCS) temperature and the LPI system is not needed to maintain RCS temperature, secure all operating LPI pumps." Step 2.4 verifies RCS pressure at approximately 310 psig and then step 2.5 opens 2LP-10 to leak check the 2B piping.

Contrary to the above, on May 18, 1997, the station was not maintained and operated in accordance with approved procedures in that: during the performance of PT/2/A/0203/04, Enclosure 13.3, Step 2.5 was performed out of sequence, in that, valve 2LP-10 was opened with the 2C LPI pump still in operation (to have been shutdown by previous step 2.3). This resulted in overpressurizing the Unit 2 "B" train of the LPI system.

This is a Severity Level IV violation (Supplement I).

- D. 10 CFR 50, Appendix B, Criterion XVI, Corrective Actions, states that measures shall be established to assure that conditions adverse to quality such as failures, malfunctions, deficiencies, deviations, defective material, and equipment, and non-conformances are promptly identified and corrected. In the case of significant conditions adverse to quality, the measures shall assure that the cause of the condition is determined and corrective action taken to preclude repetition.

Contrary to the above, Calculation OSC-6548 (dated July 22, 1996), a measure taken in response to a previously identified valve operability concern, was inadequate to assure valve inoperability would be promptly identified and corrected. Specifically, Calculation OSC-6548 was established for determining the operability of specified containment isolation valves, but operability evaluations contained in the calculation and subsequent evaluations were inadequate due to the failure of the calculation to consider instrument error and other uncertainties. This inadequacy was identified on April 23, 1997.

This is a Severity Level IV violation (Supplement I).

Pursuant to the provisions of 10 CFR 2.201, Duke Power Company, is hereby required to submit a written statement or explanation to the U.S. Nuclear Regulatory Commission, ATTN: Document Control Desk, Washington, DC 20555, with a copy to the Regional Administrator, Region II, and a copy to the NRC Resident Inspector, Oconee Nuclear Plant, within 30 days of the date of the letter transmitting this Notice of Violation (Notice). This reply should be

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clearly marked as a "Reply to a Notice of Violation" and should include for each violation: (1) the reason for the violation, or, if contested, the basis for disputing the violation; (2) the corrective steps that will be taken and the results achieved; (3) the corrective steps that will be taken to avoid further violations; and (4) the date when full compliance will be achieved. Your response may reference or include previous docketed correspondence, if the correspondence adequately addresses the required response. If an adequate reply is not received within the time specified in this Notice, an Order or a Demand for Information may be issued as to why the license should not be modified, suspended, or revoked, or why such other action as may be proper should not be taken. Where good cause is shown, consideration will be given to extending the response time.

Because your response will be placed in the NRC Public Document Room (PDR), to the extent possible, it should not include any personal privacy, proprietary, or safeguards information so that it can be placed in the PDR without redaction. If personal privacy or proprietary information is necessary to provide an acceptable response, then please provide a bracketed copy of your redacted copy of your response that deletes such information. If you request withholding of such material, you must specifically identify the portions of your response that you seek to have withheld and provide in detail the bases for your claim of withholding (e.g., explain why the disclosure of information will create an unwarranted invasion of personal privacy or provide the information required by 10 CFR 2.790(b) to support a request for withholding confidential commercial or financial information). If safeguards information is necessary to provide an acceptable response, please provide the level of protection described in 10 CFR 73.21.

Dated at Atlanta, Georgia
this 18 day of July 1997