



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

May 4, 2015

Vice President, Operations
Entergy Operations, Inc.
River Bend Station
5485 U.S. Highway 61N
St. Francisville, LA 70775

SUBJECT: RIVER BEND STATION, UNIT 1 - CORRECTION TO RELIEF REQUEST
NO. VRR-RBS-2014-1 REGARDING THE THIRD 10-YEAR INSERVICE
TESTING INTERVAL (TAC NO. MF4125)

Dear Sir or Madam:

By letter dated March 23, 2015 (Agencywide Documents Access and Management System Accession No. ML15071A141), the U.S. Nuclear Regulatory Commission (NRC) authorized the use of Relief Request VRR-RBS-2014-1 to Facility Operating License No. NPF-47 for the River Bend Station, Unit 1 (RBS), in response to your application dated May 14, 2014, as supplemented by letter dated October 8, 2014. The relief request provides an alternative to certain inservice test (IST) requirements of the 2001 Edition through 2003 Addenda of the American Society of Mechanical Engineers, Code for Operation and Maintenance of Nuclear Power Plants, for the IST program at RBS, for the remainder of the third 10-year IST program interval.

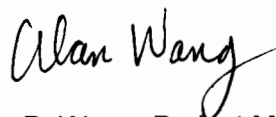
On April 15, 2015, the RBS staff notified the NRC staff that the safety evaluation for RBS Relief Request VRR-RBS-2014-1 was missing page 4. The missing page is enclosed.

Please insert page 4 of the safety evaluation with the enclosed page. The NRC staff regrets any inconvenience this may have caused.

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If you have any questions, please contact me at (301) 415-1445 or by e-mail at Alan.Wang@nrc.gov.

Sincerely,

A handwritten signature in black ink that reads "Alan Wang". The signature is written in a cursive style with a large, looping 'A' and a long, sweeping 'W'.

Alan B. Wang, Project Manager
Plant Licensing Branch IV-2 and Decommissioning
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket No. 50-458

Enclosure:
Safety Evaluation page 4

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ENCLOSURE

SAFETY EVALUATION PAGE 4 FOR
RELIEF REQUEST NO. VRR-RBS-2014-1
DATED MARCH 23, 2015

ENTERGY OPERATIONS, INC.

RIVER BEND STATION, UNIT 1

DOCKET NO. 50-458

outage, which includes the removal and replacement of eight SRVs, are conducted under equivalent radiological conditions and with the same personnel requirements.

Based on this data, Entergy has concluded that the expected cumulative radiation exposure to remove and replace a single SRV would be approximately 0.594 person-rem [roentgen equivalent man]. ... Therefore, absent the requested relief, replacement of eight incremental SRVs would result in approximately 4.752 additional person-rem over three refueling outages.

Proposed Alternate and Basis for Use

As an alternative to the Code required 5-year test interval per Appendix I, paragraph I-1320(a), RBS proposes that the Class 1 pressure relief valves be tested at least once every three refueling cycles (approximately 6 years/72 months) with a minimum of 20% of the valves tested within any 24-month interval. This 20% would consist of valves that have not been tested during the current 72-month interval, if they exist. The test interval for any individual valve would not exceed 72 months except that a 6-month grace period is allowed to coincide with refueling outages to accommodate extended shutdown periods.

After as-found set pressure testing, the valves shall be disassembled and inspected to verify that parts are free of defects resulting from time-related degradation or service induced wear. As-left set pressure testing shall be performed following maintenance and prior to returning the valve to service. Each valve shall have been disassembled and inspected prior to the start of the 72 month interval. Disassembly and inspection performed prior to the implementation of Code Case OMN-17 may be used.

The relief valve testing and maintenance cycle at RBS consists of removal of the SRV complement requiring testing and transportation to an off-site test facility. Upon receipt at the off-site facility the valves are subject to an as-found inspection, seat leakage and set pressure testing. Prior to the return of a complement of SRVs for installation in the plant, the valves are disassembled and inspected to verify that internal surfaces and parts are free from defects or service induced wear prior to the start of the next test interval. During this process, anomalies or damage are identified and resolved. Damaged or worn parts, springs, gaskets and seals are replaced as necessary. The valve seats are lapped, if necessary. Following reassembly, the valve's set pressure is recertified with an acceptance criterion of $\pm 1\%$. This existing process is in accordance with ASME OM Code Case OMN-17, Paragraphs (d) and (e).

RBS has reviewed the as-found set pressure test results for all of the SRVs tested since 2008.... RBS has had only one as-found test failure since 2008 that exceeded the as-found acceptance criteria (+3%, -5%). The one as-found failure was in the negative (or conservative) direction.

RBS submits that the proposed alternative of increasing the test interval for the Class 1 pressure relief valves from 5 years to 3 fuel cycles (approximately 6

If you have any questions, please contact me at (301) 415-1445 or by e-mail at Alan.Wang@nrc.gov.

Sincerely,

/RA/

Alan B. Wang, Project Manager
Plant Licensing Branch IV-2 and Decommissioning
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

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Safety Evaluation page 4

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