

OPERATING DATA REPORT

DOCKET NO 50-269

DATE January 15, 1999

COMPLETED BY R.A. Williams

TELEPHONE 704-382-5346

OPERATING STATUS

1. Unit Name: Oconee 1
2. Reporting Period: December 1, 1998-December 31, 1998
3. Licensed Thermal Power (Mwt): 2568
4. Nameplate Rating (Gross MWe): 934
5. Design Electrical Rating (Net MWe): 886
6. Maximum Dependable Capacity (Gross MWe): 886
7. Maximum Dependable Capacity (Net MWe): 846
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report. Give Reasons: _____

Notes Year-to date and cumulative capacity factors are calculated using a weighted average for maximum dependable capacity.

9. Power Level To Which Restricted, If Any (Net MWe): _____

10. Reason For Restrictions, If any: _____

9901200397 990112
PDR ADOCK 05000269
R PDR

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	744.0	8760.0	223201.0
12. Number Of Hours Reactor Was Critical	744.0	7303.9	172307.2
13. Reactor Reserve Shutdown Hours	--0--	--0--	--0--
14. Hours Generator On-Line	744.0	7255.6	169230.1
15. Unit Reserve Shutdown Hours	--0--	--0--	--0--
16. Gross Thermal Energy Generated (MWH)	1909359	18248619	416506604
17. Gross Electrical Energy Generated (MWH)	658050	6290605	143923760
18. Net Electrical Energy Generated (MWH)	629277	5989197	136784261
19. Unit Service Factor	100.0	82.8	75.8
20. Unit Availability Factor	100.0	82.8	75.8
21. Unit Capacity Factor (Using MDC Net)	100.0	80.8	71.7
22. Unit Capacity Factor (Using DER Net)	95.5	77.2	69.1
23. Unit Forced Outage Rate	0.0	17.2	10.2
24. Shutdown Scheduled Over Next 6 Months (Type, Date, and Duration of Each): Refueling - June 04, 1999 - 40 days			

25. If Shut Down At End Of Report Period. Estimated Date of Startup: None

26. Units In Test Status (Prior to Commercial Operation):

	Forecast	Achieved
INITIAL CRITICALITY	_____	_____
INITIAL ELECTRICITY	_____	_____
COMMERCIAL OPERATION	_____	_____

NRC Calculated from Generator Nameplate Data:
1 037 937 KVA x 0.90 Pf=934 MW

OPERATING DATA REPORT

DOCKET NO 50-269
 UNIT Oconee 1
 DATE January 15, 1998
 COMPLETED BY R.A. Williams
 TELEPHONE 704-382-5346

MONTH December, 1998

<u>DAY</u>	<u>AVERAGE DAILY POWER LEVEL (MWe-Net)</u>	<u>DAY</u>	<u>AVERAGE DAILY POWER LEVEL (MWe-Net)</u>
1	<u>842</u>	17	<u>839</u>
2	<u>842</u>	18	<u>838</u>
3	<u>842</u>	19	<u>845</u>
4	<u>839</u>	20	<u>849</u>
5	<u>835</u>	21	<u>851</u>
6	<u>837</u>	22	<u>856</u>
7	<u>836</u>	23	<u>856</u>
8	<u>836</u>	24	<u>856</u>
9	<u>835</u>	25	<u>857</u>
10	<u>837</u>	26	<u>857</u>
11	<u>839</u>	27	<u>857</u>
12	<u>839</u>	28	<u>857</u>
13	<u>843</u>	29	<u>858</u>
14	<u>843</u>	30	<u>858</u>
15	<u>843</u>	31	<u>858</u>
16	<u>841</u>		

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-269

UNIT NAME OCONEE 1

DATE 01/15/99

COMPLETED BY R. A. Williams

TELEPHONE (704)-382-5346

REPORT MONTH December 1998

NO.	DATE	(1) TYPE	DURATION HOURS	(2) REASON	(3) METHOD OF SHUT DOWN R/X	LICENSE EVENT REPORT NO.	(4) SYS- TEM CODE	(5) COMPONENT CODE	CAUSE AND CORRECTIVE ACTION TO PREVENT RECURRENCE
		NO	SHUTDOWNS	OR		REDUCTION	S		

- (1)
F Forced
S Scheduled

- (2)
Reason:
A-Equipment Failure (Explain)
B-Maintenance or test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operator Error (Explain)
H-Other (Explain)

- (3)
Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Other (Explain)

- (4)
Exhibit G - Instructions
for Preparation of Data
Entry Sheets For License
Event Report (LER)
File (NUREG-0161)

- (5)
Exhibit I - Same Source

MONTHLY REFUELING INFORMATION REQUEST

1. Facility name: Oconee Unit 1
2. Scheduled next refueling shutdown: June 1999
3. Scheduled restart following refueling: July 1999

THE PROJECT MANAGER HAS BEEN ADVISED BY SEPARATE COMMUNICATION OF ANY T.S. CHANGE OR LICENSE AMENDMENT. THEREFORE, QUESTIONS 4 THROUGH 6 WILL NO LONGER BE MAINTAINED IN THIS REPORT.

4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment?

If yes, what will these be?

If no, has reload design and core configuration been reviewed by Safety Review Committee regarding unreviewed safety questions?

5. Scheduled date(s) for submitting proposed licensing action and supporting information.
6. Important licensing considerations (new or different design or supplier, unreviewed design or performance analysis methods, significant changes in design or new operating procedures).
7. Number of Fuel assemblies
(a) in the core: 177
(b) in the spent fuel pool: 1094*
(c) in the ISFSI: 960****
8. Present licensed fuel pool capacity: 1312
Size of requested or planned increase: **
9. Projected date of last refueling which can be accommodated by present license capacity: February 2013***

DUKE POWER COMPANY

DATE: January 15, 1999

Name of Contact: R. A. Williams

Phone: (704) - 382-5346

* Represents the combined total for Units 1 and 2

** On January 29, 1990, received a license for ISFSI which will store 2112 assemblies

*** This date is based on 88 Dry Storage Modules. We currently have 48 modules (1152 spaces). Additional modules will be built on an as-needed basis.

**** Represents the combined total for Units 1, 2, and 3

DOCKET: 50 - 269

UNIT: Oconee 1

DATE: 01/15/99

NARRATIVE SUMMARY

MONTH: December, 1998

Oconee Unit 1 began the month of December operating at 100% full power. The unit operated at or near 100% full power for the entire month.

Prepared by: R. A. Williams
Telephone: (704) - 382-5346

OPERATING DATA REPORT

DOCKET NO 50-270

DATE January 15, 1999

COMPLETED BY R.A. Williams

TELEPHONE 704-382-5346

OPERATING STATUS

1. Unit Name: Oconee 2
2. Reporting Period: December 1, 1998-December 31, 1998
3. Licensed Thermal Power (Mwt): 2568
4. Nameplate Rating (Gross MWe): 934
5. Design Electrical Rating (Net MWe): 886
6. Maximum Dependable Capacity (Gross MWe): 886
7. Maximum Dependable Capacity (Net MWe): 846
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report. Give Reasons: _____

Notes Year-to date and cumulative capacity factors are calculated using a weighted average for maximum dependable capacity.

9. Power Level To Which Restricted, If Any (Net MWe): _____
10. Reason For Restrictions, If any: _____

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	744.0	8760.0	213121.0
12. Number Of Hours Reactor Was Critical	744.0	6969.2	169041.8
13. Reactor Reserve Shutdown Hours	--0--	--0--	--0--
14. Hours Generator On-Line	744.0	6778.0	166756.2
15. Unit Reserve Shutdown Hours	--0--	--0--	--0--
16. Gross Thermal Energy Generated (MWH)	1909976	17066517	409400544
17. Gross Electrical Energy Generated (MWH)	668303	5935693	140366931
18. Net Electrical Energy Generated (MWH)	640350	5652233	133674298
19. Unit Service Factor	100.0	77.4	78.2
20. Unit Availability Factor	100.0	77.4	78.2
21. Unit Capacity Factor (Using MDC Net)	101.7	76.3	73.4
22. Unit Capacity Factor (Using DER Net)	97.1	72.8	70.8
23. Unit Forced Outage Rate	0.0	4.5	9.9
24. Shutdown Scheduled Over Next 6 Months (Type, Date, and Duration of Each): None			

25. If Shut Down At End Of Report Period: Estimated Date of Startup: None
26. Units In Test Status (Prior to Commercial Operation):

	Forecast	Achieved
INITIAL CRITICALITY	_____	_____
INITIAL ELECTRICITY	_____	_____
COMMERCIAL OPERATION	_____	_____

NRC Calculated from Generator Nameplate Data:
1 937 937 KVA x 0.90 Pf=934 MW

OPERATING DATA REPORT

DOCKET NO 50-270
 UNIT Oconee 2
 DATE January 15, 1998
 COMPLETED BY R.A. Williams
 TELEPHONE 704-382-5346

MONTH December, 1998

<u>DAY</u>	<u>AVERAGE DAILY POWER LEVEL (MWe-Net)</u>	<u>DAY</u>	<u>AVERAGE DAILY POWER LEVEL (MWe-Net)</u>
1	<u>862</u>	17	<u>862</u>
2	<u>862</u>	18	<u>862</u>
3	<u>862</u>	19	<u>863</u>
4	<u>862</u>	20	<u>863</u>
5	<u>859</u>	21	<u>863</u>
6	<u>862</u>	22	<u>864</u>
7	<u>862</u>	23	<u>861</u>
8	<u>862</u>	24	<u>861</u>
9	<u>862</u>	25	<u>862</u>
10	<u>861</u>	26	<u>862</u>
11	<u>862</u>	27	<u>862</u>
12	<u>862</u>	28	<u>862</u>
13	<u>862</u>	29	<u>863</u>
14	<u>862</u>	30	<u>863</u>
15	<u>862</u>	31	<u>823</u>
16	<u>862</u>		

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-270
 UNIT NAME OCONEE 2
 DATE 01/15/99
 COMPLETED BY R. A. Williams
 TELEPHONE (704)-382-5346

REPORT MONTH December 1998

NO.	DATE	(1) TYPE	DURATION HOURS	(2) REASON	(3) METHOD OF SHUT DOWN R/X	LICENSE EVENT REPORT NO.	(4) SYS- TEM CODE	(5) COMPONENT CODE	CAUSE AND CORRECTIVE ACTION TO PREVENT RECURRENCE
		NO	SHUTDOWNS	OR		REDUCTION	S		

- (1)
 F Forced
 S Scheduled

- (2)
 Reason:
 A-Equipment Failure (Explain)
 B-Maintenance or test
 C-Refueling
 D-Regulatory Restriction
 E-Operator Training & License Examination
 F-Administrative
 G-Operator Error (Explain)
 H-Other (Explain)

- (3)
 Method:
 1-Manual
 2-Manual Scram
 3-Automatic Scram
 4-Other (Explain)

- (4)
 Exhibit G - Instructions
 for Preparation of Data
 Entry Sheets For License
 Event Report (LER)
 File (NUREG-0161)
- (5)
 Exhibit I - Same Source

MONTHLY REFUELING INFORMATION REQUEST

1. Facility name: Oconee Unit 2
2. Scheduled next refueling shutdown: November 1999
3. Scheduled restart following refueling: December 1999

THE PROJECT MANAGER HAS BEEN ADVISED BY SEPARATE COMMUNICATION OF ANY T.S. CHANGE OR LICENSE AMENDMENT. THEREFORE, QUESTIONS 4 THROUGH 6 WILL NO LONGER BE MAINTAINED IN THIS REPORT.

4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment?

If yes, what will these be?

If no, has reload design and core configuration been reviewed by Safety Review Committee regarding unreviewed safety questions?

5. Scheduled date(s) for submitting proposed licensing action and supporting information.
6. Important licensing considerations (new or different design or supplier, unreviewed design or performance analysis methods, significant changes in design or new operating procedures).
7. Number of Fuel assemblies
(a) in the core: 177
(b) in the spent fuel pool: 1094*
(c) in the ISFSI: See unit 1 ****
8. Present licensed fuel pool capacity: 1312
Size of requested or planned increase: **
9. Projected date of last refueling which can be accommodated by present license capacity: October 2013***

DUKE POWER COMPANY

DATE: January 15, 1999

Name of Contact: R. A. Williams

Phone: (704) - 382-5346

* Represents the combined total for Units 1 and 2

** See footnote on Unit 1

*** This date is based on 88 Dry Storage Modules. We currently have 48 modules (1152 spaces). Additional modules will be built on an as needed basis.

**** See footnote on Unit 1

DOCKET: 50 - 270

UNIT: Oconee 2

Date: 01/15/99

NARRATIVE SUMMARY

MONTH: December, 1998

Oconee Unit 2 began the month of December operating at 100% full power. The unit operated at or near 100% full power for the entire month.

Prepared by: R. A. Williams
Telephone: (704) - 382-5346

OPERATING DATA REPORT

DOCKET NO 50-287

DATE January 15, 1999

COMPLETED BY R.A. Williams

TELEPHONE 704-382-5346

OPERATING STATUS

1. Unit Name: Oconee 3
2. Reporting Period: December 1, 1998-December 31, 1998
3. Licensed Thermal Power (Mwt): 2568
4. Nameplate Rating (Gross MWe): 934
5. Design Electrical Rating (Net MWe): 886
6. Maximum Dependable Capacity (Gross MWe): 886
7. Maximum Dependable Capacity (Net MWe): 846
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report. Give Reasons: _____

Notes Year-to date and cumulative capacity factors are calculated using a weighted average for maximum dependable capacity.

9. Power Level To Which Restricted, If Any (Net MWe): _____

10. Reason For Restrictions, If any: _____

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	744.0	8760.0	210768.0
12. Number Of Hours Reactor Was Critical	345.8	7082.8	163361.2
13. Reactor Reserve Shutdown Hours	--0--	--0--	--0--
14. Hours Generator On-Line	293.3	7026.5	160954.8
15. Unit Reserve Shutdown Hours	--0--	--0--	--0--
16. Gross Thermal Energy Generated (MWH)	705686	17398096	401028228
17. Gross Electrical Energy Generated (MWH)	241499	6067785	138517041
18. Net Electrical Energy Generated (MWH)	217537	5782938	132092651
19. Unit Service Factor	39.4	80.2	76.4
20. Unit Availability Factor	39.4	80.2	76.4
21. Unit Capacity Factor (Using MDC Net)	34.6	78.0	73.3
22. Unit Capacity Factor (Using DER Net)	33.0	74.5	70.7
23. Unit Forced Outage Rate	60.4	7.9	10.4
24. Shutdown Scheduled Over Next 6 Months (Type, Date, and Duration of Each): None			

25. If Shut Down At End Of Report Period. Estimated Date of Startup: January 02, 1999

26. Units In Test Status (Prior to Commercial Operation): _____ Forecast: _____ Achieved: _____

INITIAL CRITICALITY _____
 INITIAL ELECTRICITY _____
 COMMERCIAL OPERATION _____

NRC Calculated from Generator Nameplate Data:
 1 037 937 KVA x 0.90 Pf=934 MW

OPERATING DATA REPORT

DOCKET NO 50-287
 UNIT Oconee 3
 DATE January 15, 1998
 COMPLETED BY R.A. Williams
 TELEPHONE 704-382-5346

MONTH December, 1998

<u>DAY</u>	<u>AVERAGE DAILY POWER LEVEL (MWe-Net)</u>	<u>DAY</u>	<u>AVERAGE DAILY POWER LEVEL (MWe-Net)</u>
1	<u>0</u>	17	<u>0</u>
2	<u>0</u>	18	<u>0</u>
3	<u>0</u>	19	<u>65</u>
4	<u>0</u>	20	<u>508</u>
5	<u>0</u>	21	<u>764</u>
6	<u>0</u>	22	<u>858</u>
7	<u>0</u>	23	<u>859</u>
8	<u>0</u>	24	<u>858</u>
9	<u>0</u>	25	<u>857</u>
10	<u>0</u>	26	<u>857</u>
11	<u>0</u>	27	<u>858</u>
12	<u>0</u>	28	<u>858</u>
13	<u>0</u>	29	<u>860</u>
14	<u>0</u>	30	<u>861</u>
15	<u>0</u>	31	<u>490</u>
16	<u>0</u>		

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-287
 UNIT NAME OCONEE 3
 DATE 01/15/99
 COMPLETED BY R. A. Williams
 TELEPHONE (704)-382-5346

PAGE 1 OF 4

REPORT MONTH December 1998

N O	DATE	(1) T Y P E	DURATION HOURS	(2) R E A S O N	(3) M E T - H O D O F S H U T D O W N R/ X	LICENSE EVENT REPORT NO.	(4) S Y S - T E M C O D E	(5) C O M P O N E N T C O D E	CAUSE AND CORRECTIVE ACTION TO PREVENT RECURRENCE
7	98-12- 1	F	1.12	A	--		RB	CRDRVE	0.05 DAY OUTAGE DELAY DUE TO CONTROL ROD DRIVE MECHANISM PROBLEM
8	98-12- 1	F	15.00	B	--		CB	XXXXXX	0.63 DAY OUTAGE DELAY DUE TO REACTOR COOLANT SYSTEM HIGH POINT VENT TEST
9	98-12- 1	F	29.00	A	--		HH	VALVEX	1.21 DAY OUTAGE DELAY DUE TO FEEDWATER VALVE PACKING LEAK
10	98-12- 2	F	14.00	B	--		RC	ZZZZZZ	0.58 DAY OUTAGE DELAY DUE TO ZERO POWER PHYSICS TESTING
11	98-12- 3	F	10.00	A	--		HG	XXXXXX	0.42 DAY OUTAGE DELAY DUE TO FEEDWATER CHEMISTRY OUT OF SPEC
12	98-12- 3	F	47.00	B	--		SF	VALVEX	1.96 DAY OUTAGE DELAY DUE TO "3CF13" VALVE FAILED INTER-SYSTEM LOCA LEAK RATE TEST

(1)
 F Forced
 S Scheduled

(2)
 Reason:
 A-Equipment Failure (Explain)
 B-Maintenance or test
 C-Refueling
 D-Regulatory Restriction
 E-Operator Training & License Examination
 F-Administrative
 G-Operator Error (Explain)
 H-Other (Explain)

(3)
 Method:
 1-Manual
 2-Manual Scram
 3-Automatic Scram
 4-Other (Explain)

(4)
 Exhibit G - Instructions
 for Preparation of Data
 Entry Sheets For Licensee
 Event Report (LER)
 File (NUREG-0161)

(5)
 Exhibit I - Same Source

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-287
 UNIT NAME OCONEE 3
 DATE 01/15/99
 COMPLETED BY R. A. Williams
 TELEPHONE (704)-382-5346

REPORT MONTH December 1998

N O	DATE	(1) T Y P E	DURATION HOURS	(2) R E A S O N	(3) M E T - H O D O F S H U T D O W N R/ X	LICENSE EVENT REPORT NO.	(4) S Y S - T E M C O D E	(5) C O M P O N E N T C O D E	CAUSE AND CORRECTIVE ACTION TO PREVENT RECURRENCE
13	98-12- 5	F	21.00	A	--		CF	VALVEX	0.88 DAY OUTAGE DELAY DUE TO LOW PRESSURE INJECTION VALVE LEAK
14	98-12- 6	F	35.00	A	--		RB	CRDRVE	1.46 DAY OUTAGE DELAY DUE TO CONTROL ROD DRIVE MECHANISM #9 VENT FAILED TO OPEN
15	98-12- 8	F	28.00	B	--		CH	HTEXCH	1.17 DAY OUTAGE DELAY DUE TO STEAM GENERATOR BUBBLE TEST
16	98-12- 9	F	19.00	B	--		CH	HTEXCH	0.79 DAY OUTAGE DELAY DUE TO STEAM GENERATOR DRIP TEST
17	98-12-10	F	16.00	A	--		RB	FUELXX	0.67 DAY OUTAGE DELAY DUE TO BOWED FUEL ASSEMBLIES
18	98-12-10	F	1.95	A	--		CH	HTEXCH	0.08 DAY OUTAGE DELAY DUE TO STEAM GENERATOR INSPECTION ROBOT FAILURE

- (1)
 F Forced
 S Scheduled

- (2)
 Reason:
 A-Equipment Failure (Explain)
 B-Maintenance or test
 C-Refueling
 D-Regulatory Restriction
 E-Operator Training & License Examination
 F-Administrative
 G-Operator Error (Explain)
 H-Other (Explain)

- (3)
 Method:
 1-Manual
 2-Manual Scram
 3-Automatic Scram
 4-Other (Explain)

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 Exhibit I - Same Source

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-287
 UNIT NAME OCONEE 3
 DATE 01/15/99
 COMPLETED BY R. A. Williams
 TELEPHONE (704)-382-5346

REPORT MONTH December 1998

N O	DATE	(1) T Y P E	DURATION HOURS	(2) R E A S O N	(3) M E T- H O D O F S H U T D O W N R/X	LICENSE EVENT REPORT NO.	(4) S Y S- T E M C O D E	(5) C O M P O N E N T C O D E	CAUSE AND CORRECTIVE ACTION TO PREVENT RECURRENCE
9-P	98-12-10	S	--	B	--		HA	TURBIN	HOLDING AT 17% POWER FOR TURBINE OVERSPEED TRIP TEST
19	98-12-11	S	2.97	B	--		HA	TURBIN	TURBINE OVERSPEED TRIP TEST
20	98-12-11	F	201.20	A	1		HG	XXXXXX	SECONDARY SYSTEM HIGH SODIUM CONCENTRATION
10-P	98-12-19	F	--	A	--		HG	XXXXXX	SECONDARY SYSTEM HIGH SODIUM CONCENTRATION PROBLEMS
11-P	98-12-19	F	--	B	--		IA	INSTRU	NUCLEAR INSTRUMENTATION CALIBRATION
12-P	98-12-19	F	--	A	--		HG	DEMINX	HOLDING AT 34% POWER TO PLACE MORE POWDEX CELLS IN-SERVICE
13-P	98-12-19	S	--	B	--		IE	INSTRU	POWER ESCALATION TESTING

(1)
 F Forced
 S Scheduled

(2)
 Reason:
 A-Equipment Failure (Explain)
 B-Maintenance or test
 C-Refueling
 D-Regulatory Restriction
 E-Operator Training & License Examination
 F-Administrative
 G-Operator Error (Explain)
 H-Other (Explain)

(3)
 Method:
 1-Manual
 2-Manual Scram
 3-Automatic Scram
 4-Other (Explain)

(4)
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(5)
 Exhibit I - Same Source

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-287
 UNIT NAME OCONEE 3
 DATE 01/15/99
 COMPLETED BY R. A. Williams
 TELEPHONE (704)-382-5346

REPORT MONTH December 1998

NO.	DATE	(1) TYPE	DURATION HOURS	(2) REASON	(3) METHOD OF SHUT DOWN R/X	LICENSE EVENT REPORT NO.	(4) SYS- TEM CODE	(5) COMPONENT CODE	CAUSE AND CORRECTIVE ACTION TO PREVENT RECURRENCE
14-P	98-12-20	F	--	A	--		HG	XXXXXX	SECONDARY CHEMISTRY
15-P	98-12-20	F	--	B	--		IA	INSTRU	NUCLEAR INSTRUMENTATION CALIBRATION
16-P	98-12-20	S	--	B	--		IE	INSTRU	POWER ESCALATION TESTING
21	98-12-31	F	9.42	A	3		RB	CKTBKR	(REACTOR/TURBINE TRIP) REACTOR PROTECTION SYSTEM CONTROL ROD DRIVE BREAKER TRIPPED

(1)
 F Forced
 S Scheduled

(2)
 Reason:
 A-Equipment Failure (Explain)
 B-Maintenance or test
 C-Refueling
 D-Regulatory Restriction
 E-Operator Training & License Examination
 F-Administrative
 G-Operator Error (Explain)
 H-Other (Explain)

(3)
 Method:
 1-Manual
 2-Manual Scram
 3-Automatic Scram
 4-Other (Explain)

(4)
 Exhibit G - Instructions
 for Preparation of Data
 Entry Sheets For Licensee
 Event Report (LER)
 File (NUREG-0161)

(5)
 Exhibit I - Same Source

DOCKET: 50 - 287

UNIT: Oconee 3

Date: 01/15/99

NARRATIVE SUMMARY

MONTH: December, 1998

Oconee Unit 3 began the month of December in end-of-cycle 17 refueling outage. The refueling outage spanned a total of 63.33 days. The refueling outage was delayed for the following reasons; 0.05 days due to control rod drive mechanism problems continued from November, 1998, 0.63 days due to reactor coolant system high point vent test, 1.21 days due to feedwater valve packing leak, 0.58 days due to zero power physics testing, 0.42 days due to feedwater chemistry out of spec, 1.96 days due to "3CF13" valve failing inter-system LOCA rate test, 0.88 days due to low pressure injection valve leak, 1.46 days due to control rod drive mechanism #9 vent failed to open, 1.17 days due to steam generator bubble test, 0.79 days due to steam generator drip test, 0.67 days due to bowed fuel assemblies, 0.08 days due to steam generator inspection robot failure. The unit was placed on-line 12/10/98 at 2104. The unit increased power and held at 17% power to perform the turbine overspeed trip test. The unit was taken off-line on 12/11/98 at 0051 due to the turbine overspeed trip test. On 12/11/98 at 0349 the unit remained off-line due to secondary system high sodium concentration. The unit returned to service on 12/19/98 at 1301 and held at 15% due to secondary system high sodium concentration problems until 1649. The unit held at 30% power from 1819 to 1858 due to nuclear instrumentation calibration. The unit held at 34% power from 2030 to 2107 to place more powdex cells in-service. The unit held at 41% power from 12/19/98 at 2220 to 12/20/98 at 0006 due to power escalation testing. The unit held at 42% power from 0045 to 0201 due to secondary chemistry. On 12/20/98 the unit held at 65% power from 0953 to 1755 due to nuclear instrumentation calibration. The unit held at 72% power from 2039 to 12/21/98 at 0507 due to power escalation testing. The unit returned to 100% full power on 12/21/98 at 1521 and operated at or near 100% full power until 12/31/98 at 1435, when the unit experience an reactor/turbine trip when the reactor protection system control rod drive breaker tripped. The unit was in the outage the remainder of the month.

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