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SUBJECT: Provides clarification of rev level of TR DPC-NE-2005, "DPC Thermal-Hydraulic Statistical Core Design Methodology," submitted in Sept 1992. Util accepts listed NRC specified conditions re use of BWU-Z critical heat flux correlation.

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May 13, 1999

U. S. Nuclear Regulatory Commission
Washington D. C. 20555-0001

ATTENTION: Document Control Desk

Subject: Oconee Nuclear Station
Docket Numbers 50-269, -270, and -287
Duke Commitment to Conditions of SER and
Clarification of Topical Report DPC-NE-2005
Revision Level

Duke Energy Corporation Topical Report DPC-NE-2005, "Duke Power Company Thermal-Hydraulic Statistical Core Design Methodology," was submitted to NRC in September 1992; approval was granted in February 1995. This initial revision included Appendices A and B, which contained Oconee and McGuire/Catawba plant specific data. Subsequent to Rev 0, Duke submitted Appendix C on April 26, 1996 requesting approval for applying the BWU-Z CHF correlation for analyses of the McGuire and Catawba reactor cores with MkbW fuel. Appendix C contained McGuire/Catawba plant specific data for MkbW fuel using the new CHF correlation, BWU-Z. Appendix C was approved on November 7, 1996. Duke placed the November 7, 1996 NRC Safety Evaluation and Appendix C in the back of DPC-NE-2005 and entitled this report DPC-NE-2005P-A, Rev 1. Rev 1 contains no unreviewed technical information. It simply places previously NRC approved documents DPC-NE-2005, Rev. 0 and Appendix C into the same report.

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Within this letter, Duke makes the following commitment:

Following NRC's approval of Appendix D (which was submitted in a Duke letter to the NRC dated April 22, 1997) Duke will incorporate the NRC's Safety Evaluation and Appendix D into DPC-NE-2005P-A, Rev. 1 and at this time change the revision level to DPC-NE-2005P-A, Rev 2. No other technical changes will be made.

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Further, Duke Energy Corporation accepts the following NRC specified conditions applicable to use of the BWU-Z critical heat flux correlation for Mark B11 fuel in the Oconee reactors:

- (1) Omission of the parameter "Fq" from the SCD analysis of the Oconee plant with a new fuel design must be justified for each particular case. Acceptance of its omission in the case of Mark-B11 fuel does not constitute a general approval of its removal from the parameters to be considered in this methodology.
- (2) The applicability of a CHF correlation to mixed core geometries is an issue that must be examined for each transition to new fuel to determine if the mixed core non-uniformities take the local hot channel conditions outside the range of applicability of CHF correlation.
- (3) The SCD analysis should be revised as needed to reflect the modification if Mark-B11 CHF correlation range of applicability is changed.

If there are any questions, or additional information required, please call R. M. Gribble at (704) 382-6160 or K. R. Epperson at (704) 382-6785.

M. S. Tuckman

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