Mr. William R. McCollum, Jr. Vice President, Oconee Nuclear Site Duke Energy Corporation P. O. Box 1439 Seneca, SC 29679

SUBJECT: REQUEST FOR ADDITIONAL INFORMATION FOR THE REVIEW OF THE

OCONEE NUCLEAR STATION, UNITS 1, 2, AND 3, LICENSE RENEWAL

APPLICATION

Dear Mr. McCollum:

By letter dated July 6, 1998, Duke Energy Corporation (Duke) submitted for the Nuclear Regulatory Commission's (NRC's) review an application pursuant to 10 CFR Part 54, to renew the operating licenses for the Oconee Nuclear Station (Oconee), Units 1, 2, and 3. Exhibit A to the application is the Oconee Nuclear Station License Renewal Technical Information Report (OLRP-1001), which contains the technical information required by 10 CFR Part 54. The NRC staff is reviewing the information contained in OLRP-1001 and has identified, in the enclosure, areas where additional information is needed to complete its review. Specifically, the enclosed questions are from the Mechanical Engineering Branch regarding Sections 3.4.4 and 3.5.5 of OLRP-1001.

Please provide a schedule by letter, electronic mail, or telephonically for the submittal of your responses within 30 days of the receipt of this letter. Additionally, the staff would be willing to meet with Duke prior to the submittal of the responses to provide clarifications of the staff's requests for additional information.

Sincerely,

original signed by:

Joseph M. Sebrosky, Project Manager License Renewal Project Directorate Division of Reactor Program Management Office of Nuclear Reactor Regulation

Docket Nos. 50-269, 50-270, and 50-287

Enclosure: Request for Additional Information

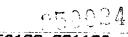
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NAME	LBerry (JSebrosky Jeff	CIGrimes (5)
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CC:

Paul R. Newton, Esquire
Duke Energy Corporation
422 South Church Street
Mail Stop PB-05E
Charlotte, North Carolina 28201-1006

J. Michael McGarry, III, Esquire Anne W. Cottingham, Esquire Winston and Strawn 1400 L Street, NW. Washington, DC 20005

Mr. Rick N. Edwards Framatome Technologies Suite 525 1700 Rockville Pike Rockville, Maryland 20852-1631

Manager, LIS NUS Corporation 2650 McCormick Drive, 3rd Floor Clearwater, Florida 34619-1035

Senior Resident Inspector U. S. Nuclear Regulatory Commission 7812B Rochester Highway Seneca, South Carolina 29672

Regional Administrator, Region II U. S. Nuclear Regulatory Commission Atlanta Federal Center 61 Forsyth Street, SW, Suite 23T85 Atlanta, Georgia 30303

Virgil R. Autry, Director
Division of Radioactive Waste Management
Bureau of Land and Waste Management
Department of Health and
Environmental Control
2600 Bull Street
Columbia, South Carolina 29201-1708

County Supervisor of Oconee County Walhalla, South Carolina 29621

Chattooga River Watershed Coalition P. O. Box 2006 Clayton, GA 30525 Mr. J. E. Burchfield Compliance Manager Duke Energy Corporation Oconee Nuclear Site P. O. Box 1439 Seneca, South Carolina 29679

Ms. Karen E. Long Assistant Attorney General North Carolina Department of Justice P. O. Box 629 Raleigh, North Carolina 27602

L. A. Keller
Manager - Nuclear Regulatory Licensing
Duke Energy Corporation
526 South Church Street
Charlotte, North Carolina 28201-1006

Mr. Richard M. Fry, Director
Division of Radiation Protection
North Carolina Department of
Environment, Health, and
Natural Resources
3825 Barrett Drive
Raleigh, North Carolina 27609-7721

Gregory D. Robison
Duke Energy Corporation
Mail Stop EC-12R
P. O. Box 1006
Charlotte, North Carolina 28201-1006

Robert L. Gill, Jr.
Duke Energy Corporation
Mail Stop EC-12R
P. O. Box 1006
Charlotte, North Carolina 28201-1006
RLGILL@DUKE-ENERGY.COM

Douglas J. Walters
Nuclear Energy Institute
1776 I Street, NW
Suite 400
Washington, DC 20006-3708
DJW@NELORG

REQUEST FOR ADDITIONAL INFORMATION OCONEE NUCLEAR STATION, UNITS 1, 2, AND 3 LICENSE RENEWAL APPLICATION, EXHIBIT A

OLRP-1001 Section No.

3.4.4 Pressurizer

3.4.4-1 Section 3.4.3 of the license renewal application references report BAW-2243A. Section 3.4.4 references BAW-2244A. These reports do not address specific time limited aging analyses for the reactor coolant system piping or for the pressurizer. It is left up to the individual plant to address this issue. Therefore, for Oconee Units 1, 2, and 3, we request that you provide a demonstration that the ASME Code Section III cumulative usage factor for all Reactor Coolant System Piping and Pressurizer Class 1 components will be less than or equal to 1.0 for 60 years of plant operation.

3.5.5 Emergency Core Cooling Systems

- 3.5.5-1 Show that the ASME Code Section III cumulative usage factor for all High Pressure Injection (HPI) System Class 1 piping and components in Oconee Units 1, 2, and 3, will be less than or equal to 1.0 for 60 years of plant operation, considering the thermal cycling effects of the following events:
 - a. Unanticipated outleakage in HPI/EI (emergency injection) lines, described in NRC Bulletin 88-08.
 - b. "Warming" make-up flow in HPI/NMU (normal makeup) lines, described in NRC Information Notice 97-46.

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PDLR RF

M. El-Zeftawy

ACRS T2E26

E-mail:

F. Miraglia

J. Roe

D. Matthews

C. Grimes

T. Essig

G. Lainas

J. Strosnider

G. Bagchi

H. Brammer

T. Hiltz

G. Holahan

S. Newberry

C. Gratton

L. Spessard

R. Correia

R. Latta

J. Peralta

J. Moore

R. Weisman

M. Zobler

E. Hackett

A. Murphy

T. Martin

D. Martin

W. McDowell ·

S. Droggitis

PDLR Staff

M. Banic

G. Hornseth

-----H. Berkow

D. LaBarge

L. Plisco

C. Ogle

R. Trojanowski

M. Scott

C. Julian

R. Architzel

J. Wilson

R. Wessman

E. Sullivan

R. Gill, Duke

D. Walters, NEI

M. Hartzman