

November 20, 1998

Mr. William R. McCollum, Jr.  
Vice President, Oconee Nuclear Site  
Duke Energy Corporation  
P. O. Box 1439  
Seneca, SC 29679

SUBJECT: REQUEST FOR ADDITIONAL INFORMATION FOR THE REVIEW OF THE  
OCONEE NUCLEAR STATION, UNITS 1, 2, AND 3, LICENSE RENEWAL  
APPLICATION

Dear Mr. McCollum:

By letter dated July 6, 1998, Duke Energy Corporation (Duke) submitted for the Nuclear Regulatory Commission's (NRC's) review an application pursuant to 10 CFR Part 54, to renew the operating licenses for the Oconee Nuclear Station (Oconee), Units 1, 2, and 3. Exhibit A to the application is the Oconee Nuclear Station License Renewal Technical Information Report (OLRP-1001), which contains the technical information required by 10 CFR Part 54. The NRC staff is reviewing the information contained in OLRP-1001 and has identified, in the enclosure, areas where additional information is needed to complete its review. Specifically, the enclosed questions are from the Mechanical Engineering Branch regarding Sections 3.4.4 and 3.5.5 of OLRP-1001.

Please provide a schedule by letter, electronic mail, or telephonically for the submittal of your responses within 30 days of the receipt of this letter. Additionally, the staff would be willing to meet with Duke prior to the submittal of the responses to provide clarifications of the staff's requests for additional information.

Sincerely,

original signed by:

Joseph M. Sebrosky, Project Manager  
License Renewal Project Directorate  
Division of Reactor Program Management  
Office of Nuclear Reactor Regulation

Docket Nos. 50-269, 50-270,  
and 50-287

Enclosure: Request for Additional Information  
cc w/encl: See next page  
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Oconee Nuclear Station (License Renewal)

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REQUEST FOR ADDITIONAL INFORMATION  
OCONEE NUCLEAR STATION, UNITS 1, 2, AND 3  
LICENSE RENEWAL APPLICATION, EXHIBIT A

OLRP-1001 Section No.

3.4.4 Pressurizer

- 3.4.4-1 Section 3.4.3 of the license renewal application references report BAW-2243A. Section 3.4.4 references BAW-2244A. These reports do not address specific time limited aging analyses for the reactor coolant system piping or for the pressurizer. It is left up to the individual plant to address this issue. Therefore, for Oconee Units 1, 2, and 3, we request that you provide a demonstration that the ASME Code Section III cumulative usage factor for all Reactor Coolant System Piping and Pressurizer Class 1 components will be less than or equal to 1.0 for 60 years of plant operation.

3.5.5 Emergency Core Cooling Systems

- 3.5.5-1 Show that the ASME Code Section III cumulative usage factor for all High Pressure Injection (HPI) System Class 1 piping and components in Oconee Units 1, 2, and 3, will be less than or equal to 1.0 for 60 years of plant operation, considering the thermal cycling effects of the following events:
- a. Unanticipated outleakage in HPI/EI (emergency injection) lines, described in NRC Bulletin 88-08.
  - b. "Warming" make-up flow in HPI/NMU (normal makeup) lines, described in NRC Information Notice 97-46.

Enclosure

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Docket File

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