Mr. W. R. McCollum, Jr.

Vice President, Oconee Site

Duke Energy Corporation P. O. Box 1439

Seneca, SC 29679

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SUBJECT: OCONEE NUCLEAR STATION UNITS 1, 2 AND 3 RE: TOPICAL REPORT

NUMBER DPC-NE-2005P USE OF BWU-Z CRITICAL HEAT FLUX

CORRELATION FOR MARK-B11 FUEL (TAC NOS. M98660, M98661, AND

M98662)

Dear Mr. McCollum:

By letter dated April 22, 1997, and supplemented by letters dated September 21, 1998, and May 13, 1999, Duke Energy Corporation requested NRC review and approval of the use of the BWU-Z Critical Heat Flux Correlation for Mark-B11 Fuel, which is described in Appendix D to Topical Report DPC-NE-2005P, "Duke Power Company Thermal - Hydraulic Statistical Core Design Methodology." The submittal contains analyses of the Mark-B11 fuel assemblies (which were analyzed using the BWU-Z critical heat flux correlation) and justifications for the specific uncertainties and distributions used in the application, and for the selected statepoints used to generate the statistical design limit.

The NRC staff was assisted in this review by its consultant, Pacific Northwest National Laboratory (PNNL). Based on the information provided and the analysis and recommendations provided by PNNL, we find the proposed Appendix D to DPC-NE-2005P to be acceptable. However, this approval is subject to the conditions described in the Safety Evaluation, which are also the commitments made in your letter dated May 13, 1999.

Sincerely,

original signed by: David E. LaBarge, Senior Project Manager, Section 1 Project Directorate II Division of Licensing Project Management Office of Nuclear Reactor Regulation

was file sente

Docket Nos. 50-269, 50-270 and 50-287

Enclosure: Safety Evaluation

cc w/encl: See next page

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UNITED STATES NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

June 8, 1999

Mr. W. R. McCollum, Jr. Vice President, Oconee Site Duke Energy Corporation P. O. Box 1439 Seneca, SC 29679

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David E. LaBarge, Senior Project Manager, Section 1

Project Directorate II

Division of Licensing Project Management

Office of Nuclear Reactor Regulation

Docket Nos. 50-269, 50-270 and 50-287

Enclosure: Safety Evaluation

cc w/encl: See next page

Oconee Nuclear Station

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