CATEGORY 1

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50-270 Oconee Nuclear Station, Unit 2, Duke Power Co. 05000270
50-287 Oconee Nuclear Station, Unit 3, Duke Power Co. 05000287
AUTH.NAME AUTHOR AFFILIATION
MCCOLLUM,W.R. Duke Power Co.
RECIP.NAME RECIPIENT AFFILIATION
Records Management Branch (Document Control Desk)

SUBJECT: Forwards Suppl 6 to amend request,converting to Improved Tech Spec.Encl revises Attachment 7 of licensee 971028 cover ltr to be consistent with revs to discussion of changes made in Suppls 1-5 & bases for 3.6.5.

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Duke Energy

W. R. McCollum, Jr.
Vice President

Duke Energy Corporation
Oconee Nuclear Station
P.O. Box 1439
Seneca, SC 29679
(864) 885-3107 OFFICE
(864) 885-3564 FAX

December 3, 1998

U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Attention: Document Control Desk

Subject: Oconee Nuclear Station
Docket Numbers 50-269, 270, and 287
Supplement 6 to Improved Technical Specifications (ITS) Submittal

Reference: W. R. McCollum, Jr.(Duke Power Company) letter to NRC Document Control Desk, dated October 28, 1997.

By letter dated October 28, 1997, Duke Energy submitted an amendment request to convert to Improved Technical Specifications. By letters dated March 26, 1998, May 20, 1998, July 29, 1998, October 1, 1998, October 28, 1998, and November 23, 1998, Duke Energy submitted Supplements 1, 1A, 2, 3, 4, and 5 to the ITS submittal. This Supplement revises Attachment 7 of the October 28, 1997 cover letter to make the attachment consistent with revisions to the Discussion of Changes (DOCs) made in Supplements 1 through 5 and the Bases for ITS 3.6.5 to more completely capture relocated requirements. These changes are provided in Enclosure 1 with revision numbers in the left hand margin to facilitate review. Enclosure 2 provides a Table of Contents, with Amendment 300 in the footer, for the Oconee final proposed ITS and Bases that were provided in Supplements 4 and 5. This enclosure also includes the revised page for B 3.6.5 with the Amendment Number in the footer. This Supplement also revises the proposed License Conditions provided in Supplement 5 based on discussions between Duke Energy and the Staff. These changes are provided in Enclosure 3.

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Duke Energy plans to implement the ITS at Oconee no later than April 30, 1999.

The proposed changes are administrative in nature and have been determined to be within the scope of the original PORC and NSRB reviews.

Pursuant to 10 CFR 50.91(b)(1), a copy of this amendment has been provided to the appropriate State of South Carolina officials.

If any additional information is needed, please contact Noel Clarkson at (864) 885-3077.

Very truly yours,

W. R. McCollum, Jr., Vice President
Oconee Nuclear Site

cc: Mr. D. E. LaBarge, Project Manager
Office of Nuclear Reactor Regulation
U. S. Nuclear Regulatory Commission
Mail Stop O-14 H25
Washington, D. C. 20555

Mr. L. A. Reyes, Regional Administrator
U. S. Nuclear Regulatory Commission - Region II
Atlanta Federal Center
61 Forsyth St., SW, Suite 23T85
Atlanta, Georgia 30303

M. A. Scott
Senior Resident Inspector
Oconee Nuclear Station

Virgil R. Autry, Director
Division of Radioactive Waste Management
Bureau of Land and Waste Management
Department of Health & Environmental Control
2600 Bull Street
Columbia, SC 29201

W. R. McCollum, Jr., being duly sworn, states that he is Vice President, Oconee Nuclear Site, Duke Energy Corporation, that he is authorized on the part of said Company to sign and file with the U. S. Nuclear Regulatory Commission this revision to the Facility Operating License Nos. DPR-38, DPR-47, DPR-55; and that all the statements and matters set forth herein are true and correct to the best of his knowledge.

W. R. McCollum, Jr., Vice President
Oconee Nuclear Site

Subscribed and sworn to before me this 3rd day of December, 1998

Connie M Breazeale
Notary Public

My Commission Expires:

2-12-2003

ENCLOSURE 1

ATTACHMENT 7
CTS REQUIREMENTS RELOCATED TO LICENSEE CONTROLLED DOCUMENTS

DOC	ISSUE RELOCATED	DOCUMENT	CONTROL
ITS SECTION: 1.0 USE AND APPLICATION			
LA1	Requirements related to the Process Control Program and definition for the Gaseous Radwaste Treatment System and Ventilation Exhaust Treatment System	SLC	10 CFR 50.59
LA2	Descriptive information regarding instrumentation, Heat Balance Check and Heat Balance Calibration	Bases	Bases Control Program
ITS SECTION: 2.0 SAFETY LIMITS			
LA1	Not used		
LA2	Actions when a Safety Limit is violated	QA Topical Report	10 CFR 50.54
ITS SECTION: 3.1 REACTIVITY CONTROL SYSTEM			
LA1	Details regarding positioning remaining rods in the affected group	Bases	Bases Control Program
LA2	Verification regarding loss of power to APSRs and calibration requirements for rod position indications	SLC	10 CFR 50.59
LA3	Requirement regarding OPERABILITY when a control rod cannot be located by position indication instrumentation	Bases	Bases Control Program
LA4	Values for SDM	COLR	5.6.5
LA5	Notification to NRC regarding cause of reactivity anomaly	SLC	10 CFR 50.59
LA6	Requirements for verifying control rod insertion time after maintenance or modification	QA Topical Report	10 CFR 50.54

ATTACHMENT 7
CTS REQUIREMENTS RELOCATED TO LICENSEE CONTROLLED DOCUMENTS

DOC	ISSUE RELOCATED	DOCUMENT	CONTROL
LA7	Requirement that safety rod groups be fully withdrawn prior to any other reduction in shutdown margin	Bases	Bases Control Program
R1	Control rod program verification	SLC	10 CFR 50.59
ITS SECTION: 3.2 POWER DISTRIBUTION LIMITS			
LA1	Regulating rod group overlap value	COLR	ITS 5.6.5
LA2	Requirements for generation of a power distribution map using the incore instrumentation system	SLC	10 CFR 50.59
LA3	Value for SDM	COLR	ITS 5.6.5
R1	Requirements associated with the control rod drive patch panels	SLC	10 CFR 50.59
R2	Requirements associated with the incore instrumentation	SLC	10 CFR 50.59
ITS SECTION: 3.3 INSTRUMENTATION			
LA1	Information regarding how the shutdown bypass setpoints are controlled or set	Bases	Bases Control Program
LA2	Details regarding functional test	Bases	Bases Control Program
LA3	Information regarding total channels available for each function and the number of channels necessary to trip each function	Bases	Bases Control Program
LA4	Details regarding the number of RCP monitor channels required to be OPERABLE for the RCP monitor logic to be considered OPERABLE	SLC	10 CFR 50.59
LA5	Information regarding total number of channels available and equipment identification nomenclature.	Base	Bases Control Program

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DOC	ISSUE RELOCATED	DOCUMENT	CONTROL
LA6	Information regarding equipment actuated by an ESPS signal	Bases	Bases Control Program
LA7	Description of the method of sensing loss of main feedwater pumps	Bases	Bases Control Program
LA8	Information regarding how channels are bypassed and controls on the bypass key	Bases	Bases Control Program
LA9	Information regarding how the minimum of three operable channels may be maintained	Bases	Bases Control Program
LA10	SCR (ETA) control relay letter designators included in the description of the function	Bases	Bases Control Program
LA11	Information regarding combinations of direct indications necessary for an OPERABLE subcooling margin monitor	Bases	Bases Control Program
LA12	Method of performance of surveillance	Bases	Bases Control Program
LA13	Method of performance of surveillance	Bases	Bases Control Program
LA14	Information regarding equipment nomenclature	Bases	Bases Control Program
R1	Surveillance requirements for instrumentation not retained in ITS	SLC	10 CFR 50.59
ITS SECTION 3.4: REACTOR COOLANT SYSTEM			
LA1	Surveillance of PORV	IST Program	10 CFR 50.55a
LA2	Requirements regarding establishment of a steam bubble prior to reducing shutdown margin < 1% $\Delta k/k$ and specifies a minimum level of 80 inches in the pressurizer	SLC	10 CFR 50.59

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DOC	ISSUE RELOCATED	DOCUMENT	CONTROL
LA3	Not used.		
LA4	Not used.		
LA5	Requirements regarding inspection of the PORV	SLC	10 CFR 50.59
LA6	Administrative requirements associated with evaluation of the safety implications of RCS LEAKAGE	SLC	10 CFR 50.59
LA7	Details regarding testing methods associated with testing the PIVs	Bases	Bases Control Program
LA8	Details regarding administrative controls which constitute the second LTOP subsystem.	Bases	Bases Control Program
LA9	Details regarding verification that required RCS vent paths are open.	SLC	10 CFR 50.59
LA10	Subcriticallity requirements when the reactor coolant temperature is < 525°F	SLC	10 CFR 50.59
LA11	Detail regarding the comparison of RCS total activity	Bases	Bases Control Program
LA12	Requirements for leak testing the RCS following opening	SLC	10 CFR 50.59
LA13	Leak testing for specified PIVs after maintenance, repair or replacement	QA Topical Report	10 CFR 50.54
LA14	Requirement that setpoint of the pressurizer code safety valves be in accordance with ASME, Boiler and Pressure Vessel Code.	Bases	Bases Control Program
R1	Restrictions regarding steam generator secondary side pressure	SLC	10 CFR 50.59

ATTACHMENT 7
CTS REQUIREMENTS RELOCATED TO LICENSEE CONTROLLED DOCUMENTS

DOC	ISSUE RELOCATED	DOCUMENT	CONTROL
R2	Restrictions upon the pressurizer heatup and cooldown rates and use of the pressurizer spray	SLC	10 CFR 50.59
R3	Not used.		
R4	Requirements associated with RCS LEAKAGE which is returnable leakage	SLC	10 CFR 50.59
ITS SECTION: 3.5 EMERGENCY CORE COOLING SYSTEMS			
LA1	Details on what constitutes an OPERABLE train and valve numbers associated with HPI crossover valves	Bases	Bases Control Program
LA2	Requirements regarding locking and tagging CFT electrically operated discharge valves and locking manual valve on discharge line from BWST.	SLC	10 CFR 50.59
LA3	Details of the methods of performing HPI and LPI tests and what constitutes acceptable test	Bases	Bases Control Program
LA4	Core Flooding System Test	SLC	10 CFR 50.59
LA5	Verification of manual OPERABILITY of power operated valves	SLC	10 CFR 50.59
LA6	LPI valve identification numbers	Bases	Bases Control Program
6 LA7	Requirements for minimum level in the CFT	Bases	Bases Control Program
R1	Requirements for the High Pressure Injection and Chemical Addition Systems	SLC	10 CFR 50.59
ITS SECTION: 3.6 CONTAINMENT SYSTEMS			
LA1	Details of methods of performing a test and what constitutes an acceptable test	Bases	Bases Control Program

ATTACHMENT 7
CTS REQUIREMENTS RELOCATED TO LICENSEE CONTROLLED DOCUMENTS

DOC	ISSUE RELOCATED	DOCUMENT	CONTROL
LA2	Leakage integrity testing for purge isolation valves	SLC	10 CFR 50.59
LA3	Requirements regarding Containment Hydrogen Control Systems	SLC	10 CFR 50.59
LA4	Details of what constitutes an OPERABLE system	Bases	Bases Control Program
LA5	Determination of fouling rate for reactor building cooling units	SLC	10 CFR 50.59
LA6	Requirements regarding abnormal degradation of reactor building structural integrity	Pre-Stressed Concrete Tendon Surveil-lance Program	ITS 5.5.7
LA7	Requirements associated with inservice testing of ASME Code valves in accordance with Section XI.	IST Program	10 CFR 50.55a
LA8	Requirement that the valve seals of the containment purge isolation valves be visually inspected and adjusted or replaced as appropriate.	SLC	10 CFR 50.59
ITS SECTION: 3.7 PLANT SYSTEMS			
LA1	Not used.		
LA2	Requirements associated with Section XI requirements for relief valve testing	IST Program	10 CFR 50.55a
LA3	Requirements for testing movement capability of TSVs	IST Program	10 CFR 50.55a
LA4	Methodology for testing EFW system	SLC	10 CFR 50.59
LA5	Details regarding the steam supply requirements for the turbine driven EFW pump	Bases	Bases Control Program

ATTACHMENT 7
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DOC	ISSUE RELOCATED	DOCUMENT	CONTROL
LA6	Not used.		
LA7	Details regarding testing for specified LPSW valves	SLC	10 CFR 50.59
LA8	Details regarding filter testing	Bases	Bases Control Program
LA9	Not used		
LA10	Details regarding requirements for OPERABLE ECCW siphon header and testing of Emergency Condenser Circulating Water System	Bases	Bases Control Program
LA11	Requirements for testing louvers in the Control Room Booster Fan Trains	Bases	Bases Control Program
LA12	Requirements for snubbers	SLC	10 CFR 50.59
LA13	Testing requirements for automatic valves in the EFW System	IST Program	10 CFR 50.55a
LA14	Equipment functional testing requirements for equipment actuated by the MSLB Feedwater isolation feature	SLC	10 CFR 50.59
6 6 LA15	Details regarding the testing of SFPVS	Bases	Bases Control Program
6 6 6 LA16	Requirements associated with verifying the SFPVS flow capabilities	SLC	10 CFR 50.59
6 6 LA17	Details regarding testing of SFPVS	Bases	Bases Control Program
R1	Independence of controls of emergency feedwater system from integrated control system	SLC	10 CFR 50.59
R2	Testing spent fuel pool (SFP) cooling system	SLC	10 CFR 50.59
R3	Testing High Pressure Service Water Pumps and Power Supplies	SLC	10 CFR 50.59

ATTACHMENT 7
CTS REQUIREMENTS RELOCATED TO LICENSEE CONTROLLED DOCUMENTS

	DOC	ISSUE RELOCATED	DOCUMENT	CONTROL
6	R4	Not used		
	R5	Requirements for Reactor Building Polar Crane and Auxiliary Hoist	SLC	10 CFR 50.59
	R6	Not used.		
	R7	Testing requirements for Radioactive Material Sources	SLC	10 CFR 50.59
		ITS SECTION: 3.8 ELECTRICAL POWER SYSTEMS		
	LA1	Requirements regarding Keowee lake level and power restrictions for Keowee Hydro Units (KHU) during periods of commercial power generation and requirements regarding the location of these restrictions	SLC	10 CFR 50.59
	LA2	Requirements for 100 kV transmission circuit when energizing the standby buses from an OPERABLE Lee combustion turbine	Bases	Bases Control Program
	LA3	Verification of peak inverse voltage capability for each I&C auctioneering diodes	SLC	10 CFR 50.59
	LA4	Requirements regarding alternate power source availability	Bases	Bases Control Program
	LA5	Requirement for the main feeder buses and ES power strings to be energized to be considered OPERABLE	Bases	Bases Control Program
		CTS SECTION: 3.9 REFUELING OPERATIONS		
	LA1	Not used.		
	LA2	Decay time prior to the movement of irradiated fuel assemblies from reactor	SLC	10 CFR 50.59
	LA3	Indication available for source range neutron instrumentation	Bases	Bases Control Program

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CTS REQUIREMENTS RELOCATED TO LICENSEE CONTROLLED DOCUMENTS

DOC	ISSUE RELOCATED	DOCUMENT	CONTROL
LA4	Restriction value of K_{eff}.	Bases	Bases Control Program
R1	Area radiation monitoring in the reactor building and spent fuel pool	SLC	10 CFR 50.59
R2	Communications between the control room and refueling personnel	SLC	10 CFR 50.59
R3	Minimum distance between fuel assemblies being handled simultaneously and handling of fuel assemblies with the Auxiliary Hoist	SLC	10 CFR 50.59
R4	Restrictions on maximum weight of suspended loads which can be transported over spent fuel in spent fuel pools	SLC	10 CFR 50.59
ITS SECTION: 3.10 STANDBY SHUTDOWN FACILITY			
LA1	Information regarding what constitutes an OPERABLE SSF System or Instrumentation	Bases	Bases Control Program
LA2	Requirements for instrumentation associated with the Reactor Coolant Makeup Pumps, the Auxiliary Service Water Pump, the Underground Fuel Oil Storage Tank Inventory, the DG Service Water Pump and DG Air Start System pressure	SLC	10 CFR 50.59
LA3	Annual inspection of the SSF DG	SLC	10 CFR 50.59
LA4	Verification that DG starts from standby condition and runs according to procedures and requirements recommended by manufacturer	Bases	Bases Control Program
LA5	DG testing after maintenance or modification	QA Topical Report	10 CFR 50.54

ATTACHMENT 7
CTS REQUIREMENTS RELOCATED TO LICENSEE CONTROLLED DOCUMENTS

DOC	ISSUE RELOCATED	DOCUMENT	CONTROL
LA6	Duration of battery discharge test	Bases	Bases Control Program
LA7	Specific gravity to be corrected to 77°F	Bases	Bases Control Program
LA8	Not used		
ITS SECTION: 4.0 DESIGN FEATURES			
LA1	Not used.		
LA2	Descriptive details related to the Reactor Containment, Penetrations, and Containment Systems	UFSAR	10 CFR 50.59
LA3	Descriptive details related to the Reactor Core and the Reactor Coolant System (RCS)	UFSAR	10 CFR 50.59
LA4	Equivalency of Restricted area to the exclusion area for purposes of 10 CFR 20 and gaseous releases	UFSAR	10 CFR 50.59
ITS SECTION: 5.0 ADMINISTRATIVE CONTROLS			
LA1	Methods for implementing requirements for Liquid Holdup Tanks, Waste Gas Holdup Tanks, and Explosive Gas Mixtures	SLC	10 CFR 50.59
LA2	Inspection and maintenance of structural integrity of reactor internals	SLC	10 CFR 50.59
LA3	Method of ventilation system filter testing	SLC	10 CFR 50.59
LA4	Additional requirements for licensed and non-licensed personnel.	SLC	10 CFR 50.59
LA5	Requirements regarding training for station personnel and the fire brigade	SLC	10 CFR 50.59
LA6	Not used		

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DOC	ISSUE RELOCATED	DOCUMENT	CONTROL
LA7	Requirements regarding PORC and Technical Review and Control; temporary procedure changes; review and control for procedures, modifications, tests, experiments, reportable events, special reviews and investigations, and unplanned onsite releases; records of the above activities	QA Topical Report	10 CFR 50.54
LA8	Requirements for the offsite review committee	QA Topical Report	10 CFR 50.54
LA9	Details associated with internal actions required for reportable events	QA Topical Report	10 CFR 50.54
LA10	Requirements for procedures for a station survey following an earthquake	SLC	10 CFR 50.59
LA11	Requirements for procedures for implementing Process Control Program	QA Topical Report	10 CFR 50.54
LA12	Content of procedures for controlling ph in recirculated coolant after a LOCA and procedures for remote or local operation of components necessary to establish high and low pressure injection within 15 minutes of a line break	SLC	10 CFR 50.59
LA13	Requirements for a respiratory protective program	SLC	10 CFR 50.59
LA14	Requirements regarding Radiological Environmental Monitoring Program	SLC	10 CFR 50.59
LA15	Requirements for records retention	QA Topical Report	10 CFR 50.54

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DOC	ISSUE RELOCATED	DOCUMENT	CONTROL
LA16	Requirements for submitting a report following receipt of an operating license; installation of fuel that has a different design or has been manufactured by a different fuel supplier; modifications that may have altered the nuclear, thermal, or hydraulic performance of the unit; and amendments to the license involving planned increase in power operation	SLC	10 CFR 50.59
LA17	Requirement for Shift Manager (current title is Shift Work Manager) to be an experienced SRO	SLC	10 CFR 50.59
LA18	Requirements of the LPI System leakage rate testing	SLC	10 CFR 50.59
LA19	Not used		
LA20	Details regarding testing associated with containment structural integrity.	Pre-Stressed Concrete Contain-ment Tendon Testing Program	ITS 5.5.7
LA21	Method of ventilation system filter testing	SLC	10 CFR 50.59
LA22	Requirements regarding establishment of values and documentation in COLR for volume and boron concentration in Concentrated Boric Acid Tank	SLC	10 CFR 50.59
6 6 6 LA23	Requirements for operating the PRVS fans at design flow every 18 months	SLC	10 CFR 50.59
6 6 LA24	Details of the methods of ventilation filter testing	SLC	10 CFR 50.59