

November 18, 1998

Mr. William R. McCollum, Jr.
Vice President, Oconee Nuclear Site
Duke Energy Corporation
P. O. Box 1439
Seneca, SC 29679

SUBJECT: REQUEST FOR ADDITIONAL INFORMATION FOR THE REVIEW OF THE
OCONEE NUCLEAR STATION, UNITS 1, 2, AND 3, LICENSE RENEWAL
APPLICATION

Dear Mr. McCollum:

By letter dated July 6, 1998, Duke Energy Corporation (Duke) submitted for the Nuclear Regulatory Commission's (NRC's) review an application pursuant to 10 CFR Part 54, to renew the operating licenses for the Oconee Nuclear Station (Oconee), Units 1, 2, and 3. Exhibit A to the application is the Oconee Nuclear Station License Renewal Technical Information Report (OLRP-1001), which contains the technical information required by 10 CFR Part 54. The NRC staff is reviewing the information contained in OLRP-1001 and has identified, in the enclosure, areas where additional information is needed to complete its review. Specifically, the enclosed questions are from the Mechanical Engineering Branch regarding the following Sections of OLRP-1001: 3.5.6, 3.5.7, and 3.5.13.

Please provide a schedule by letter, electronic mail, or telephonically for the submittal of your responses within 30 days of the receipt of this letter. Additionally, the staff would be willing to meet with Duke prior to the submittal of the responses to provide clarifications of the staff's requests for additional information.

Sincerely,

Original Signed By

Joseph M. Sebrosky, Project Manager
License Renewal Project Directorate
Division of Reactor Program Management
Office of Nuclear Reactor Regulation

Docket Nos. 50-269, 50-270, and 50-287
Enclosure: Request for Additional Information
cc w/encl: See next page
Distribution: See next page

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Oconee Nuclear Station (License Renewal)

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REQUEST FOR ADDITIONAL INFORMATION
OCONEE NUCLEAR STATION, UNITS 1, 2, AND 3
LICENSE RENEWAL APPLICATION, EXHIBIT A

OLRP-1001 Section No.

3.5.6 Auxiliary Systems

- 3.5.6-1 Based on the staff's experience, degradation of piping systems (e.g., cracking of weld) may potentially be caused by vibration (mechanical or hydrodynamic) loading. Clarify whether this loading effect has been considered in the aging review for the auxiliary systems discussed in Section 3.5.6, and, if this effect is excluded, provide the basis for its exclusion.
- 3.5.6-2 Section 2.5.6 indicates that some portions of the auxiliary systems within the scope of license renewal are not designed to withstand the effects of a design basis earthquake. Clarify which components and piping segments within the category of "Seismic II over I" (a nonseismic Category I system, structure, or component whose failure could cause loss of safety function of a seismic Category I system, structure, or component) would be subject to aging management review. Additionally, clarify which aging management program will address these components and piping segments and specifically discuss implementation of the program to manage the applicable aging effects during the period of extended operation.

3.5.7 Process Auxiliaries

- 3.5.7-1 Based on the staff's experience, degradation of piping systems (e.g., cracking of weld) may potentially be caused by vibration (mechanical or hydrodynamic) loading. Clarify whether this loading effect has been considered in the aging review for the process auxiliaries discussed in Section 3.5.7, and, if this effect is excluded, provide the basis for its exclusion.
- 3.5.7-2 Section 2.5.7 indicates that some portions of the process auxiliaries within the scope of license renewal are not designed to withstand the effects of a design basis earthquake. Clarify which components and piping segments within the category of "Seismic II over I" (a nonseismic Category I system, structure, or component whose failure could cause loss of safety function of a seismic Category I system, structure, or component) would be subject to aging management review. Additionally, clarify which aging management program will address these components and piping segments and specifically discuss implementation of the program to manage the applicable aging effects during the period of extended operation.

3.5.13 Keowee Hydroelectric Station

- 3.5.13-1 Based on the staff's experience, degradation of piping systems (e.g., cracking of weld) may potentially be caused by vibration (mechanical or hydrodynamic) loading. Clarify whether this loading effect has been considered in the aging review for the

Enclosure

Keowee Hydroelectric Station discussed in Section 3.5.13, and, if this effect is excluded, provide the basis for its exclusion.

- 3.5.13-2 Section 2.5.13 indicates that some portions of the Keowee Hydroelectric Station piping systems within the scope of license renewal are not designed to withstand the effects of a design basis earthquake. Clarify which components and piping segments within the category of "Seismic II over I" (a nonseismic Category I system, structure, or component whose failure could cause loss of safety function of a seismic Category I system, structure, or component) would be subject to aging management review. Additionally, clarify which aging management program will address these components and piping segments and specifically discuss implementation of the program to manage the applicable aging effects during the period of extended operation.