

April 14, 2015

MEMORANDUM TO: Nathan Sanfilippo, Chief
Performance Assessment Branch
Division of Inspection and Regional Support
Office of Nuclear Reactor Regulation

FROM: Andrew Waugh, Reactor Operations Engineer */RA/*
Performance Assessment Branch
Division of Inspection and Regional Support
Office of Nuclear Reactor Regulation

SUBJECT: SUMMARY OF THE REACTOR OVERSIGHT PROCESS WORKING
GROUP PUBLIC MEETING HELD ON MARCH 18, 2015

On March 18, 2015, the U.S. Nuclear Regulatory Commission (NRC) staff hosted the Reactor Oversight Process (ROP) Working Group (WG) public meeting with the Nuclear Energy Institute (NEI) ROP Task Force and other industry representatives. Enclosure 1 contains the meeting attendance list; and Enclosure 2 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML15103A245) contains the white papers and handouts discussed during the meeting; Enclosure 3 (ADAMS Accession No. ML15103A246) contains the Frequently Asked Questions (FAQs) Log and the FAQs discussed during the meeting. Meeting attendees discussed topics related to probabilistic risk assessment (PRA), assessment, and performance indicators (PIs).

In response to industry and licensee interest on how to request plant specific Standardized Plant Analysis Risk (SPAR) models for review, the PRA Operations and Human Factors Branch (APHB) staff is in the process of drafting a letter that will present the proper protocol and procedures. The Office of Nuclear Regulatory Research has also tentatively planned to hold a public workshop in July 2015 to discuss quality assurance aspects of the SPAR models as well as the proper method for making SPAR model updates.

On February 26, 2015, the NRC hosted a Category 2 public meeting to discuss the technical basis for the Significance Determination Process (SDP), the appropriate process to model the safety significance of inspection findings that cause initiating event occurrence, and approaches to modify applicable guidance documents. The summary for this meeting is in ADAMS under the Accession No. ML15070A050. Based on the dialogue and feedback from the public meetings over the past year, the staff plans to revise Inspection Manual Chapter (IMC) 0308, Attachment 3 "Significance Determination Process Basis Document" and Risk Assessment Standardization Project, Volume 1, Section 8, sometime in the summer of 2015.

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The Security Training and Support Branch (STSB) staff from the Office of Nuclear Security and Incident Response discussed efforts to modify IMC 0609, Appendix E, to make the current security SDP a more graded approach that is repeatable and predictable. These efforts should be completed around the June timeframe.

The Reactor Inspection Branch (IRIB) staff discussed the regional differences in screening findings of very low safety significance (Green). The staff is looking to enhance procedures to help improve objectivity and predictability in this area. The IRIB staff also discussed that it is looking at ways to make a more standardized process for those licensees transitioning into decommissioning.

The IRIB staff discussed the revised Occupational Safety and Health (OSHA) reporting criteria under 29 CFR 1904.39(a), "Reporting fatalities, hospitalizations, amputations, and losses of an eye as a result of work-related incidents to OSHA." The staff discussed that the change in OSHA reporting criteria did not affect the changes in NRC reporting criteria of 10 CFR 50.72, "Immediate notification requirements for operating nuclear power reactors".

The Performance Assessment Branch (IPAB) staff shared the potential new layout of the ROP public webpage with the WG and solicited their feedback. Slides showing the proposed website layout can be found in ADAMS under the Accession No. ML15079A008. The staff will share updates during a future meeting.

The IPAB staff also mentioned that they are beginning development regarding the treatment of new reactors in the ROP. A more detailed discussion will be had on this topic during the May ROP WG public meeting.

Regarding the PI program, staff and industry addressed the following items:

- (1) The staff discussed the status of the Watts Bar Nuclear Plant 2 transition into the ROP with regards to PI validity. The staff indicated that it is currently discussing, both internally and with the licensee, the approach for making the Mitigating System Performance Index (MSPI), Unplanned Scrams per 7,000 Critical Hours, and Unplanned Power Changes per 7,000 Critical Hours PIs valid.
- (2) The staff discussed the "Exclusion of Low Risk Trains/ Segments from Unavailability Monitoring" white paper (Enclosure 2). The staff requested that industry submit an FAQ with any proposed changes to the MSPI indicator for the staff to review.

Staff and industry discussed the following PI FAQs (Enclosure 3):

- FAQ 14-08: This is a site-specific FAQ for Prairie Island. This FAQ addresses a diesel generator failure counted under the Emergency AC MSPI. The licensee proposes to retract a diesel generator failure because it resulted from a reverse power condition that would not occur while the diesel generator is performing its monitored function and it is not indicative of the reliability of the equipment. The staff agreed with this position and presented their response with no comments from industry or the licensee. This FAQ was made final.

- FAQ 14-10: This is a site-specific FAQ for Indian Point Units 2 and 3. This FAQ requests guidance on the data reporting for the Protected Area Equipment PI. The licensee proposes to maintain two separate indicators for Unit 2 and Unit 3, but to use the same value for both Units starting early 2015. The NRC staff reviewed the information provided by the licensee and provided comments on the licensee's approach. The staff is aligned with the licensee's proposed treatment for future Protected Area Equipment PI data at this site. The staff's response was made final during this meeting.
- FAQ 15-01: This is a site-specific FAQ for Perry. During an unplanned scram, the Full Core Display indicated dual rod indication, both red and green lights for Rods Out and In, respectively. Operators utilized a second indication on the Rod Action Control System to verify rods were indeed inserted before proceeding with the normal scram response procedure. A question about the NEI guidance existed whether this should then count as a complicated scram (IE04). The staff agreed with the licensee that in this particular case, the complicated scram PI is not applicable because the operators were able to verify the condition of the reactor without exiting the normal scram response procedure. NEI committed to provide an update to the guidance in NEI 99-02, and the NRC would draft a closure response.

A public tele-conference will be held on April 16, 2015, to discuss FAQs. The next ROP WG public meeting is scheduled to be held on May 20, 2015.

Enclosures:

Attendance List – March
18, 2015

Handouts Discussed in the March
18, 2015, ROP WG Public Meeting

Reactor Oversight Process Task Force
FAQ Log – March 18, 2015

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- Reactor Oversight Process Task Force FAQ Log – March 18, 2015

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ADAMS ACCESSION NO: ML15103A127

ADAMS Package No. ML15103A124

***concurring via email**

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DATE	4/07/15	4/07/15	4/07/15	4/14/15

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**REACTOR OVERSIGHT PROCESS PUBLIC MEETING
ATTENDANCE LIST
March 18, 2015**

Nathan Sanfilippo	NRC
Dan Merzke	NRC
Andrew Waugh	NRC
Mary Anderson	NRC
Eric Powell	NRC
Aron Lewin	NRC
Ron Frahm	NRC
Zack Hollcraft	NRC
See-Meng Wong	NRC
Chris Regan	NRC
Rani Franovich	NRC
Alonzo Richardson	NRC
Bridget Curran	NRC
Robert Krsek	NRC
Mike Balazik*	NRC
Steve Vaughn*	NRC
Luis Cruz*	NRC
Marlone Davis*	NRC
Jim Slider	NEI
Larry Parker	STARS Alliance
Tony Zimmerman	Duke Energy
Suzanne Leblang	Entergy
Lenny Sueper	Xcel Energy
Ron Gaston	Exelon
Pete Wilson	TVA
Eugene Cobey	TVA
Doug McKinney	Southern Company
William Webster	Dominion
Robin Ritzman	First Energy
Doug Rapp	First Energy
Shirelle Allen	Duke Energy
Sarah Zafar	Erin Engineering
Deann Raleigh**	Sciencetech
Ken Heffner	Certrec
Roy Linthicum*	Exelon
Steve Catron*	NextEra Energy
Lance Sterling*	South Texas
Carlos Cisco*	Winston
David Lockwood	First Energy
Li Zhiping	

*participated via teleconference and/or online meeting