



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

May 20, 2015

Mr. David A. Heacock
President and Chief Nuclear Officer
Dominion Nuclear Connecticut, Inc.
Innsbrook Technical Center
5000 Dominion Boulevard
Glen Allen, VA 23060-6711

SUBJECT: MILLSTONE POWER STATION, UNIT NO. 3 - ISSUANCE OF AMENDMENT
RE: ADMINISTRATIVE CHANGES AND CORRECTIONS TO THE TECHNICAL
SPECIFICATIONS (TAC NO. MF3872)

Dear Mr. Heacock:

The Commission has issued the enclosed Amendment No. 261 to Renewed Facility Operating License No. NPF-49 for the Millstone Power Station, Unit No. 3. This amendment is in response to your application dated March 28, 2014.

The amendment deletes the Technical Specification (TS) index and makes other editorial, corrective and minor changes to the TSs.

A copy of the related Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly *Federal Register* notice.

Sincerely,

A handwritten signature in black ink, appearing to read "R. Guzman", with a long horizontal flourish extending to the right.

Richard V. Guzman, Senior Project Manager
Plant Licensing Branch I-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket No. 50-423

Enclosures:

1. Amendment No. 261 to NPF-49
2. Safety Evaluation

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

DOMINION NUCLEAR CONNECTICUT, INC.

DOCKET NO. 50-423

MILLSTONE POWER STATION, UNIT NO. 3

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 261
Renewed License No. NPF-49

1. The U.S. Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by the applicant dated March 28, 2014, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Renewed Facility Operating License No. NPF-49 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 261 and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated in the renewed license. DNC shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of issuance, and shall be implemented within 30 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Michael I. Dudek, Acting Chief
Plant Licensing Branch I-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment:
Changes to the License
and Technical Specifications

Date of Issuance: May 20, 2015

ATTACHMENT TO LICENSE AMENDMENT NO. 261

RENEWED FACILITY OPERATING LICENSE NO. NPF-49

DOCKET NO. 50-423

Replace the following page of the Renewed Facility Operating License with the attached revised page. The revised page is identified by amendment number and contains marginal lines indicating the areas of change.

Remove

4

Insert

4

Replace the following pages of the Appendix A Technical Specifications, with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Remove

Index, Pages i-xix

3/4 3-37

6-5

6-24

Insert

i

3/4 3-37

6-5

6-24

(2) Technical Specifications

The Technical Specifications contained in Appendix A, revised through Amendment No. 261 and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto are hereby incorporated into the license. DNC shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

- (3) DNC shall not take any action that would cause Dominion Resources, Inc. (DRI) or its parent companies to void, cancel, or diminish DNC's Commitment to have sufficient funds available to fund an extended plant shutdown as represented in the application for approval of the transfer of the licenses for MPS Unit No. 3.
- (4) Immediately after the transfer of interests in MPS Unit No. 3 to DNC, the amount in the decommissioning trust fund for MPS Unit No. 3 must, with respect to the interest in MPS Unit No. 3, that DNC would then hold, be at a level no less than the formula amount under 10 CFR 50.75.
- (5) The decommissioning trust agreement for MPS Unit No. 3 at the time the transfer of the unit to DNC is effected and thereafter is subject to the following:
- (a) The decommissioning trust agreement must be in a form acceptable to the NRC.
 - (b) With respect to the decommissioning trust fund, investments in the securities or other obligations of Dominion Resources, Inc. or its affiliates or subsidiaries, successors, or assigns are prohibited. Except for investments tied to market indexes or other non-nuclear-sector mutual funds, investments in any entity owning one or more nuclear power plants are prohibited.
 - (c) The decommissioning trust agreement for MPS Unit No. 3 must provide that no disbursements or payments from the trust, other than for ordinary administrative expenses, shall be made by the trustee until the trustee has first given the Director of the Office of Nuclear Reactor Regulation 30 days prior written notice of payment. The decommissioning trust agreement shall further contain a provision that no disbursements or payments from the trust shall be made if the trustee receives prior written notice of objection from the NRC.
 - (d) The decommissioning trust agreement must provide that the agreement can not be amended in any material respect without 30 days prior written notification to the Director of the Office of Nuclear Reactor Regulation.

PAGES i THROUGH xix HAVE BEEN INTENTIONALLY DELETED

TABLE 4.3-2 (Continued)
ENGINEERED SAFETY FEATURES ACTUATION SYSTEM INSTRUMENTATION
SURVEILLANCE REQUIREMENTS

FUNCTIONAL UNIT	CHANNEL CHECK	CHANNEL CALIBRATION	ANALOG CHANNEL OPERATIONAL TEST	TRIP ACTUATING DEVICE OPERATIONAL TEST	MASTERSLAVE RELAY LOGIC TEST	MASTERSLAVE RELAY TEST	MODES FOR WHICH SURVEILLANCE IS REQUIRED
3. Containment Isolation							
a. Phase "A" Isolation							
1. Manual Initiation	N.A.	N.A.	N.A.	SFCP	N.A.	N.A. N.A.	1, 2, 3, 4
2. Automatic Actuation Logic and Actuation Relays	N.A.	N.A.	N.A.	N.A.	SFCP(1)	SFCP(1) SFCP(4)	1, 2, 3, 4
3. Safety Injection	See Item 1. above for all Safety Injection Surveillance Requirements.						
b. Phase "B" Isolation							
1. Manual Initiation	N.A.	N.A.	N.A.	SFCP	N.A.	N.A. N.A.	1, 2, 3, 4
2. Automatic Actuation Logic and Actuation Relays	N.A.	N.A.	N.A.	N.A.	SFCP(1)	SFCP(1) SFCP(4)	1, 2, 3, 4
3. Containment Pressure-High-3	SFCP	SFCP	SFCP	N.A.	N.A.	N.A. N.A.	1, 2, 3, 4
c. DELETED							
4. Steam Line Isolation							
a. Manual Initiation							
1. Individual	N.A.	N.A.	N.A.	SFCP	N.A.	N.A. N.A.	1, 2, 3, 4
2. System	N.A.	N.A.	N.A.	SFCP	N.A.	N.A. N.A.	1, 2, 3, 4

MILLSTONE - UNIT 3

3/4 3-37

Amendment No. 46, 70, 79, 100, 129, 198, 219, 258, 261

ADMINISTRATIVE CONTROLS

6.3 FACILITY STAFF QUALIFICATIONS

6.3.1 Each member of the facility staff shall meet or exceed the minimum qualifications of ANSI N18.1-1971* for comparable positions. Exceptions to this requirement are specified in the Quality Assurance Program.

6.3.2 The operations manager or at least one operations middle manager shall hold a senior reactor operator license for Millstone Unit No. 3.

6.4 TRAINING

6.4.1 A retraining and replacement training program for the facility staff that meets or exceeds the requirements as specified in the Quality Assurance Program and 10 CFR Part 55.59 shall be maintained.

6.4.2 Deleted.

6.5 DELETED.

* As of November 1, 2001, applicants for reactor operator and senior reactor operator qualification shall meet or exceed the education and experience guidelines of Regulatory Guide 1.8, Revision 3, May 2000.

ADMINISTRATIVE CONTROLS

6.13 RADIOLOGICAL EFFLUENT MONITORING AND OFFSITE DOSE CALCULATION MANUAL (REMODCM)

- a. The REMODCM shall contain the methodology and parameters used in the calculation of offsite doses resulting from radioactive gaseous and liquid effluents, in the calculation of gaseous and liquid effluent monitoring alarm and trip setpoints, and in the conduct of the radiological environmental program; and
- b. The REMODCM shall also contain the radioactive effluent controls and radiological environmental monitoring activities and descriptions of the information that should be included in the Annual Radiological Environmental Operating, and Radioactive Effluent Release, reports required by Specification 6.9.1.3 and Specification 6.9.1.4.

Licensee initiated changes to the REMODCM:

- a. Shall be documented and records of reviews performed shall be retained. This documentation shall contain:
 - 1) sufficient information to support the change(s) together with the appropriate analyses or evaluations justifying the change(s), and
 - 2) a determination that the change(s) will maintain the level of radioactive effluent control required by 10 CFR 20.1302, 40 CFR Part 190, 10 CFR 50.36a, and Appendix I to 10 CFR 50, and not adversely impact the accuracy or reliability of effluent, dose, or setpoint calculations;
- b. Shall become effective after review and acceptance by FSRC and the approval of the designated officer; and
- c. Shall be submitted to the Commission in the form of a complete, legible copy of the entire REMODCM as a part of or concurrent with the Radioactive Effluent Release Report for the period of the report in which any change in the REMODCM was made. Each change shall be identified by markings in the margin of the affected pages, clearly indicating the area of the page that was changed, and shall indicated the date (i.e., month and year) the change was implemented.

6.14 RADIOACTIVE WASTE TREATMENT

Procedures for liquid and gaseous radioactive effluent discharges from the Unit shall be prepared, approved, maintained and adhered to for all operations involving offsite releases of radioactive effluents. These procedures shall specify the use of appropriate waste treatment systems utilizing the guidance provided in the REMODCM.

The Solid Radioactive Waste Treatment System shall be operated in accordance with the Process Control Program to process wet radioactive wastes to meet shipping and burial ground requirements.



UNITED STATES
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SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 261

TO RENEWED FACILITY OPERATING LICENSE NO. NPF-49

DOMINION NUCLEAR CONNECTICUT, INC.

DOCKET NO. 50-423

MILLSTONE POWER STATION, UNIT NO. 3

1.0 INTRODUCTION

By letter dated March 28, 2014 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML14093A026), the Dominion Nuclear Connecticut, Inc. (DNC, the licensee), submitted a request for changes to the Millstone Power Station, Unit No. 3 (MPS3) Technical Specifications (TSs). The proposed changes would delete the TS index and make other editorial, corrective, and minor changes to the TSs.

2.0 REGULATORY EVALUATION

Section 182a of the Atomic Energy Act requires applicants for nuclear power plant operating licenses to include TSs as part of the license. The U.S. Nuclear Regulatory Commission's (NRC's) regulatory requirements related to the content of the TSs are contained in Title 10 of the *Code of Federal Regulations* (10 CFR) Part 50, Section 50.36, "Technical specifications." The TS requirements in 10 CFR 50.36 include the following categories: (1) safety limits, limiting safety systems settings and control settings, (2) limiting conditions for operation, (3) surveillance requirements (SRs), (4) design features, (5) administrative controls, (6) decommissioning, (7) initial notification, and (8) written reports.

3.0 TECHNICAL EVALUATION

3.1 Deletion of TS Index

The licensee proposed to delete the entire MPS3 TS index, all pages from I through XIX, to eliminate the need to include TS index pages as part of license amendment request (LAR) submittals for future TS changes and to issue revised index pages with an amendment. The TS index identifies the contents of the TSs and where the specific TS sections can be found throughout the TSs but does not contain any technical information required by 10 CFR 50.36. Therefore, this is an editorial change and does not involve any physical changes to structures,

systems, or components (SSCs) in the plant, or the way SSCs are operated or controlled, and is, therefore, acceptable.

3.2 Correction to TS Table 4.3-2, "Engineered Safety Features Actuation System Instrumentation Surveillance Requirements"

Proposed change:

Replace Item 4, sub-letter 'd' with sub-letter 'a' on TS page 3/4 3-37 to correct a typographical error.

A typographical error is on page 3/4 3-37 in TS Table 4.3-2, "Engineered Safety Features Actuation System Instrumentation Surveillance Requirements." Specifically, the item under 4, "Steam Line Isolation" is mislabeled as "d. Manual Initiation" and should be "a. Manual Initiation." This proposed change is corrective in nature and does not involve any physical changes to SSCs in the plant, or the way SSCs are operated or controlled, and is, therefore, acceptable.

3.3 Revision of TS 6.3.2, "Facility Staff Qualifications"

TS 6.3.2 currently states:

If the operations manager does not hold a senior reactor operator license for Millstone Unit No. 3, then the operations manager shall have held a senior reactor operator license at a pressurized water reactor, and the assistant operations manager shall hold a senior reactor operator license for Millstone Unit No. 3.

The proposed change to TS 6.3.2 is as follows:

The operations manager or at least one operations middle manager shall hold a senior reactor operator license for Millstone Unit No. 3.

TS 6.3.1 currently requires that each member of the facility staff shall meet or exceed the minimum qualifications of American National Standards Institute (ANSI) N18.1-1971, "Selection and Training of Nuclear Power Plant Personnel," for comparable positions (note that as of November 1, 2001, applicants for reactor operator and senior reactor operator (SRO) qualification shall meet or exceed the education and experience guidelines of NRC Regulatory Guide 1.8, Revision 3, "Qualification and Training of Personnel for Nuclear Power Plants," May 2000 (available at ADAMS Accession No. ML003706932). ANSI N18.1-1971 specifies that the operations manager hold an SRO license for the unit. In 1987, this guidance was revised in ANSI/American Nuclear Society ANSI/ANS 3.1-1987 such that if the operations manager does not hold an SRO license, then the operations middle manager shall hold an SRO license.

The existing language used in TS 6.3.2, "Facility Staff Qualifications," reflects a limited exception to ANSI N18.1-1971. Specifically, the exception allows the assistant operations manager (i.e., the operations middle manager referenced in ANSI/ANS 3.1-1987), if one is designated, to hold an SRO license if the operations manager does not hold an SRO license for

the unit. This exception was approved for MPS3 under Amendment No. 132 issued January 7, 1997 (ADAMS Accession No. ML011790443). As stated in the NRC safety evaluation report related to Amendment No. 132, the staff concluded, in part, that:

The specification that either the Operations Manager or the Assistant Operations Manager hold a Millstone Unit 3 SRO license is consistent with the requirements of 10 CFR 50.54(l) and ensures that a licensed off-shift senior operator is directing the licensed activities of the licensed operators provided that the crews report directly to and receive overall day-to-day guidance on plant operations from the individual holding the current Senior Reactor Operator license. Requiring an ANSI/ANS-3.1-1987 qualification and licensed Assistant Operations Manager when the Operations Manager does not hold a valid Millstone Unit 3 SRO license is consistent with the requirements of ANSI N18.1-1971 and ensures there is site-specific detailed relevant technical and systems knowledge in a senior operations management position.

The proposed change is consistent with the NRC-approved limited exception to ANSI N18.1-1971. The licensee proposed to revise TS 6.3.2 such that the operations manager or at least one operations middle manager (e.g., an assistant operations manager or the supervisor in charge of the operations shift crews) shall hold an SRO license for MPS3. Modifying the title in TS 6.3.2 to allow at least one operations middle manager (i.e., any position between the operations manager and the shift managers) to hold the SRO license on the unit meets the intent of ANSI N18.1-1971. This change will allow the operations manager and other operations middle managers to perform higher level duties such as management, planning, and coordinating of operations activities with more general senior operator knowledge.

The proposed change to TS 6.3.2 is administrative in nature, pertains to manager qualifications and does not substantially change staffing requirements because the staff finds that a middle manager and assistant operations manager are equivalent positions. This proposed change continues to be consistent with the requirements of 10 CFR 50.54(l) to ensure that a licensed off-shift senior operator is directing the licensed activities of the licensed operators and with the standards set forth in ANSI N18.1-1971 to ensure there is relevant operational experience and knowledge in senior operations management position(s). The TSs would continue to require the requisite management qualifications for safe operations at MPS3, and is, therefore, acceptable.

3.4 Correction to TS 6.13, "Radiological Effluent Monitoring and Offsite Dose Calculation Manual (REMODCM)"

In TS 6.13, paragraph b under, "Licensee initiated changes to the REMODCM," currently states:

Shall become effective after review and acceptance by SORC [Site Operations Review Committee] and the approval of the designated officer; and

The proposed change to revise TS 6.13, paragraph b under, "Licensee initiated changes to the REMODCM," is as follows:

Shall become effective after review and acceptance by FSRC and the approval of the designated officer; and

The term SORC is no longer used at MPS3 as those functions are now performed by the Facility Safety Review Committee (FSRC). Accordingly, the acronym SORC has been replaced with FSRC. This proposed change updates the TS to include the current name of the committee, is corrective in nature, and does not involve any physical changes to SSCs in the plant, or the way SSCs are operated or controlled, and is, therefore, acceptable.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Connecticut State official was notified on April 22, 2015, of the proposed issuance of the amendment. The State official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding published in the *Federal Register* (FR) on November 25, 2014 (79 FR 70213). The amendment also involves changes to administrative procedures or requirements and makes editorial, corrective or other minor revisions. Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9) and (c)(10). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) there is reasonable assurance that such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: K. West

Date: May 20, 2015

May 20, 2015

Mr. David A. Heacock
President and Chief Nuclear Officer
Dominion Nuclear Connecticut, Inc.
Innsbrook Technical Center
5000 Dominion Boulevard
Glen Allen, VA 23060-6711

SUBJECT: MILLSTONE POWER STATION, UNIT NO. 3 - ISSUANCE OF AMENDMENT
RE: ADMINISTRATIVE CHANGES AND CORRECTIONS TO THE TECHNICAL
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Sincerely,

/RA/

Richard V. Guzman, Senior Project Manager
Plant Licensing Branch I-1
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Office of Nuclear Reactor Regulation

Docket No. 50-423

Enclosures:

1. Amendment No. 261 to NPF-49
2. Safety Evaluation

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DATE	4/27/2015	5/20/2015	5/20/2015	

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