

PTN L-15-1 NRC EXAM

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ES-401

Site-Specific SRO Written Examination
Cover Sheet

Form ES-401-8

U.S. Nuclear Regulatory Commission

Site-Specific SRO Written Examination

Applicant Information

Name:	
Date:	Facility/Unit: Turkey Point Units 3 and 4
Region: II	Reactor Type: W
Start Time:	Finish Time:

Instructions

Use the answer sheets provided to document your answers. Staple this cover sheet on top of the answer sheets. To pass the examination, you must achieve a final grade of at least 80.00 percent overall, with 70.00 percent or better on the SRO-only items if given in conjunction with the RO exam; SRO-only exams given alone require a final grade of 80.00 percent to pass. You have 8 hours to complete the combined examination.

Applicant Certification

All work done on this examination is my own. I have neither given nor received aid.

Applicant's Signature

Results

RO/SRO-Only/Total Examination Values	___ / ___ / ___ Points
Applicant's Score	___ / ___ / ___ Points
Applicant's Grade	___ / ___ / ___ Percent

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QUESTION 1

Given the following conditions:

- Unit 3 is operating at 100% power with all systems in automatic.

Which ONE of the following completes the statement below?

Raising _____ will result in a lower final steady state RCP #1 Seal leakoff flow.

- A. Reactor Coolant Drain Tank pressure
- B. Volume Control Tank pressure
- C. Reactor Coolant Pump Standpipe level
- D. Letdown flow

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QUESTION 2

Given the following conditions:

- Unit 4 is in MODE 5.
- The Pressurizer is solid with OMS in low pressure operation.
- 4B RHR loop is in operation.
- RCS pressure control is in manual.

Subsequently:

- Operator manually lowers demand on TC-4-144A, Letdown Temperature Controller.

Which ONE of the following identifies the plant response with no additional operator action?

- A. NRHX CCW flow decreases.
- B. Shutdown Margin increases.
- C. RCS pressure decreases.
- D. RCS temperature increases.

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QUESTION 3

Given the following conditions:

- Letdown flow is 105 gpm.

Which ONE of the following completes the statement below?

In accordance with 3-OP-047, CVCS – Charging and Letdown, before removing a letdown orifice from service, PCV-3-145, Low Pressure Letdown Controller demand is (1) to maintain pressure at (2) on PI-3-145.

- A. (1) raised
(2) 150 psig
- B. (1) raised
(2) 300 psig
- C. (1) lowered
(2) 150 psig
- D. (1) lowered
(2) 300 psig

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QUESTION 4

Given the following conditions:

- Unit 3 is in MODE 4.
- RCS pressure is 370 psig.
- Crew prepares to place RHR in service.
- MOV-3-750, Loop 3C RHR Pump Suction Stop Valve, is open.
- MOV-3-862B, RHR Suction from RWST, is open.

Subsequently:

- RCS pressure rises to 475 psig.
- MOV-3-751, Loop 3C RHR Pump Suction Stop Valve, will not open.

Which ONE of the following completes the statements below?

MOV-3-751 (1) prevented from opening by RCS pressure.

MOV-3-751 (2) prevented from opening by MOV-3-862B.

- A. (1) is
(2) is not
- B. (1) is not
(2) is not
- C. (1) is
(2) is
- D. (1) is not
(2) is

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QUESTION 5

Given the following conditions:

- A large break LOCA has occurred on Unit 3.
- The 3B 4kV Bus is faulted and locked-out.
- The crew is performing actions of 3-EOP-E-1, Loss of Reactor or Secondary Coolant.
- Conditions are met to isolate the SI Accumulators.

Which ONE of the following completes the statement below and identifies the Accumulator Isolation Valves capable of being closed?

MOV-3-865A and (1) , Accumulator Discharge Isolation valves, can be closed from (2).

- A. (1) MOV-3-865B
(2) VPB
- B. (1) MOV-3-865C
(2) VPB
- C. (1) MOV-3-865B
(2) MCC using pushbutton
- D. (1) MOV-3-865C
(2) MCC using pushbutton

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QUESTION 6

Which ONE of the following completes the statements below regarding the operation of the Pressurizer Relief Tank?

PCV-3-473, Nitrogen Regulator, is set to maintain PRT pressure at (1) psig.

PCV-3-473 (2) automatically isolate on a Containment Isolation Phase A.

- A. (1) 6 - 8
(2) will
- B. (1) 10 - 12
(2) will
- C. (1) 6 - 8
(2) will NOT
- D. (1) 10 - 12
(2) will NOT

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QUESTION 7

Given the following conditions:

- Unit 3 is in MODE 4.
- TCV-3-143, L/D Demineralizer Divert Valve, is aligned to AUTO.
- RHR is in service.

Subsequently:

- Annunciator A3/5, LTDN DEMIN HI TEMP/FLOW DIVERTED, alarms.
- FI-3-620, local NRHX CCW flow is reading 90 gpm.

Which ONE of the following completes the following statements below?

The LTDN DEMIN HI TEMP alarm setpoint is (1) .

In accordance with 3-ARP-097.CR.A, A3/5, the RCO must manually adjust (2) to restore Letdown Temperature.

- A. (1) 135°F
(2) TC-3-144A, L/D Temp Controller
- B. (1) 135°F
(2) HCV-3-142, RHR Ltdn to CVCS
- C. (1) 125°F
(2) TC-3-144A, L/D Temp Controller
- D. (1) 125°F
(2) HCV-3-142, RHR Ltdn to CVCS

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QUESTION 8

Which ONE of the following identifies the power supply to the Spray Valve Controller for PCV-3-455B, Pressurizer Spray Valve, Loop B?

REFERENCE PROVIDED

- A. 3D01
- B. 3D23
- C. 3P06
- D. 3P08

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QUESTION 9

Given the following conditions:

- Unit 4 is raising power with Main Generator output at 200 MW.
- I&C is performing 4-SMI-086.01, Turbine Emergency Trip Header Pressure Channel Calibration of PS-4-3629.
- After discovering PS-4-3629 is calibrated to 45 psig, I&C suspends 4-SMI-086.01 for lunch and leaves work as is.

Subsequently:

- Prior to restart I&C verifies PS-4-3630, Turbine Emergency Trip Header Pressure Switch, is tripped prior to recommencing work.

Which of the following completes the sentence below?

The Turbine Emergency Trip Header Pressure Switches are expected to be set at (1).

Continue with (2).

- A. (1) 45 psig
(2) 4-EOP-E-0, Reactor Trip or Safety Injection
- B. (1) 45 psig
(2) 4-GOP-301, Hot Standby to Power Operation
- C. (1) 1000 psig
(2) 4-GOP-301, Hot Standby to Power Operation
- D. (1) 1000 psig
(2) 4-EOP-E-0, Reactor Trip or Safety Injection

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QUESTION 10

Given the following conditions:

- Unit 4 is at 7% power.
- Pressurizer Pressure Channel PT-4-457 fails high.
- PT-4-457 is removed from service in accordance with 4-ONOP-049.1, Deviation or Failure of Safety Related or Reactor Protection Channels.

Which ONE of the choices below completes the follow statements?

At Power Trips Blocked light on VPA is (1) for current plant conditions.

With PT-4-457 removed from service, a Safety Injection (2) occur with a subsequent loss of 4P06.

- A. (1) NOT lit
(2) will NOT
- B. (1) NOT lit
(2) will
- C. (1) lit
(2) will NOT
- D. (1) lit
(2) will

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QUESTION 11

Given the following conditions:

- Unit 4 is at 50% power.
- PT-4-494, C S/G Pressure Transmitter Channel II, fails high.
- The bi-stables for the failed channel are tripped in accordance with 4-ONOP-049.1, Deviation or Failure of Safety Related or Reactor Protection Channels.

Subsequently:

- PT-4-495, C S/G Pressure Transmitter Channel III, fails low.

Which ONE of the following describes how the plant will respond and why?

An automatic Safety Injection will (1) Loop C Lo Stm Pressure inputs to the actuation logic are indicating a header pressure 100 psig (2) S/G pressure.

- A. (1) occur because 2/3
(2) greater than
- B. (1) occur because 2/3
(2) less than
- C. (1) NOT occur because only 1/3
(2) greater than
- D. (1) NOT occur because only 1/3
(2) less than

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QUESTION 12

Given the following plant conditions:

- Unit 3 is operating at 100% power.
- The breaker to 3A Vital MCC has tripped OPEN.

Which ONE of the following components has lost power?

- A. 3B Auxiliary Building Exhaust Fan
- B. 3A Main Steam Penetration Cooling Fan
- C. 3A Normal Containment Cooler Fan
- D. 3B CRDM Cooler Fan

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QUESTION 13

Given the following conditions:

- Unit 4 is at 100% power.

Subsequently:

- A fault occurs on the 4C SG inside containment.
- A Reactor Trip and Safety injection occur.
- 4C SG completely depressurizes.
- The Pressurizer is empty.
- Containment temperature is 197°F and lowering.
- Containment pressure is 15 psig and lowering.
- The plant conditions are stabilized.

Which ONE of the following identifies the instrumentation which is **NOT** designed to operate under the given conditions?

- A. Particulate/Gas Monitors, R-11/R-12
- B. 4C SG Pressure Transmitters, PT-4-494/495/496
- C. Pressurizer Level Transmitters, LT-4-459/460/461
- D. 4C SG Level Transmitters – Narrow range , LT-4-494/495/496

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QUESTION 14

Given the following conditions:

- A Unit 3 Reactor Trip and Safety Injection occurs.
- RCS pressure is 1000 psig and stable.
- RCS subcooling is 45°F.
- Containment pressure rises to 16 psig and continues to slowly rise.
- Annunciator H5/2, CNTMT ISOLATION ACTIVATED, is in alarm.
- The crew is performing 3-EOP-E-0, Reactor Trip or Safety Injection.
- The RCO is checking Containment Spray requirements IAW Attachment 3.

Which ONE of the following describes the NEXT required operator action(s) in accordance with Attachment 3?

- A. Manually initiate Containment Spray using ONLY the Containment Spray pushbuttons.
- B. Manually start the Containment Spray Pumps and open Containment Spray Isolation Valves.
- C. Verify SI is reset. Verify SI Amber Lights on VPB are ALL BRIGHT.
- D. Verify SI is reset and stop ALL RCPs.

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QUESTION 15

Given the following conditions:

- Unit 4 is at 100% power.

Subsequently:

- A Reactor trip occurs.
- The crew is performing Immediate Operator Actions of 4-EOP-E-0, Reactor Trip or Safety Injection.
- The BOP attempts to manually close MSR Main Steam Stop Supply Valves, but the position indications are NOT lit.

Which ONE of the following is required NEXT in accordance with 4-EOP-E-0?

- A. Manually trip the turbine.
- B. Close Main Steamline Isolation and Bypass valves.
- C. Continue verification of MSR Purge valve closure.
- D. Continue verification of Reheater Timing valve closure.

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QUESTION 16

Given the following conditions:

- Unit 3 Reactor power is stable at 8%.
- The RCO is manually controlling SG levels at 50%.

Subsequently:

- The Unit Supervisor directs raising power to 30%.
- During Turbine load increase, the Turbine Control valves opened rapidly.
- Reactor power increases to 14%.
- All SG level deviations are in alarm.

Which ONE of the following completes the following statements?

The initial SG level deviation alarms occur because SG narrow range levels indicate 5% (1) than programmed level. The RCO (2) required to throttle the Main Feedwater Control valves to maintain S/G levels.

- A. (1) less
(2) is
- B. (1) greater
(2) is
- C. (1) less
(2) is NOT
- D. (1) greater
(2) is NOT

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QUESTION 17

Given the following:

- Unit 4 is at 60% power.

Subsequently:

- A Condensate header rupture occurs.
- Both Main Feedwater Pumps trip.
- After 10 seconds, SG levels are 30% narrow range and lowering.

Which ONE of the following completes the following statements?

An automatic Reactor Trip setpoint (1) currently exceeded. AFW Pumps (2) running.

- A. (1) is
(2) are
- B. (1) is
(2) are NOT
- C. (1) is NOT
(2) are.
- D. (1) is NOT
(2) are NOT.

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QUESTION 18

Given the following conditions:

- Unit 3 is increasing power following a refueling outage.
- Unit 3 experiences a Reactor trip from 25% power.
- The crew completes 3-EOP-E-0, Reactor Trip and Safety Injection, and enters 3-EOP-ES-0.1, Reactor Trip Response.
- Pressurizer level is 14% and slowly decreasing.
- All Steam Generator Narrow Range levels are between 12% and 15% and slowly rising.
- Steam Generators pressures are approximately 990 psig and slowly decreasing.
- Tavg is 544°F and slowly decreasing.
- RCS pressure is 2125 psig and slowly decreasing.

Which ONE of the following identifies the crew's initial required response in accordance with 3-EOP-ES-0.1 to address the conditions above?

- A. Establish Emergency Boration.
- B. Reduce Auxiliary Feedwater Flow.
- C. Close MSIVs and bypass valves.
- D. Initiate a Safety Injection and return to 3-EOP-E-0.

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QUESTION 19

Given the following conditions:

- The Auxiliary Feedwater (AFW) system receives an auto-start signal.
- A AFW Pump governor malfunctions and AFW Turbine speed rises.

Which ONE of the following describes the effect of this event?

The A AFW Pump trips on over-speed at (1).

The Train 1 AFW Flow Control Valves will (2).

- A. (1) 5900 rpm
(2) remain throttled
- B. (1) 5900 rpm
(2) fully open
- C. (1) 6500 rpm
(2) remain throttled
- D. (1) 6500 rpm
(2) fully open

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QUESTION 20

Given the following plant conditions:

- Unit 3 is in MODE 1.
- Unit 4 is in MODE 1.
- A lightning strike damages the Unit 3 Startup Transformer.

Which ONE of the following statements correctly describes the required response in accordance with TS 3.8.1.1?

Within one hour, the Unit 3 crew (1) required to demonstrate the OPERABILITY of the Unit 4 Startup Transformer and (2) required to demonstrate the OPERABILITY of the Unit 3 EDGs.

- A. (1) is
(2) is
- B. (1) is not
(2) is
- C. (1) is
(2) is not
- D. (1) is not
(2) is not

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QUESTION 21

Which ONE of the following describes the effect of placing the yellow NORMAL/ISOLATE switch to ISOLATE on the 4B HHSI Pump breaker cubicle?

- A. Enables the pump's control switch on the 4KV Breaker cubicle door.
- B. Aligns backup fuses into the 4B HHSI Pump control circuit.
- C. Allows local starts at the SI Pump Room.
- D. Disables all 4B HHSI Pump Breaker protective trip signals.

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QUESTION 22

Given the following conditions:

- The 3A RHR Pump is started.
- After the pump starts, DC control power is lost.

Which ONE of the following completes the following sentence?

The 3A RHR Pump Breaker 3AA15 _____.

- A. can NOT be tripped remotely.
The breaker's blue light is extinguished.
- B. can be tripped remotely.
The breaker's blue light is extinguished.
- C. can NOT be tripped remotely.
The breaker's blue light is on.
- D. can be tripped remotely.
The breaker's blue light is on.

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QUESTION 23

Given the following conditions:

- The 4A EDG is running in parallel with 4A 4KV bus.
- The 4A EDG is running at 2800 KW
- The Varmeter on the 4A EDG Generator Control Panel 4C12A is reading 500 KVAR out.

Subsequently:

- The operator locally positions the Voltage Regulator Switch at the 4A EDG Generator Control Panel 4C12A to LOWER.

Which ONE of the following predicts the initial behavior of EDG parameters?

- A. (1) VARs increase
(2) current and temperature lower
- B. (1) VARs decrease
(2) current and temperature lower
- C. (1) VARs increase
(2) current and temperature rise
- D. (1) VARs decrease
(2) current and temperature rise

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QUESTION 24

Given the following conditions:

- A bus fault has caused 125 VDC Electrical Distribution Bus 3D01 to de-energize.

Which ONE of the following plant components will be directly affected by this loss of DC power?

- A. Loss of Train 2 Feedwater Isolation capability
- B. Loss of all 3D Switchgear control power
- C. Loss C AFW Pump control and protection
- D. Loss of 3A EDG DC power

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QUESTION 25

Given the following conditions:

- Unit 3 and 4 are at 100% power
- Liquid release is in progress IAW 0-NOP-061.11C, Controlled Liquid Release From Monitor Tank A.

Subsequently:

- Annunciator H1/4, PRMS HI RADIATION, alarms.
- Annunciator H1/6, PRMS CHANNEL FAILURE, alarms.
- Unit 3 RCO reports no indications are available for Process Radiation Monitor R-18 due to no power.

Which ONE of the following describes the initial required response?

- A. Enter the applicable TS Action Statement.
- B. Enter 3-ONOP-067, Radioactive Effluent Release.
- C. Direct Chemistry to sample effluent in accordance with ODCM.
- D. Direct Chemistry to sample effluent in accordance with 3-ARP-097.CR.H, H1/6.

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QUESTION 26

Given the following conditions:

- Units 3 and 4 are at 100% power with a normal electrical alignment.
- 3A and 3B ICW Pumps are running.

Subsequently:

- Annunciator I3/4, TRAVELING SCREEN GENERAL TROUBLE, alarms
- Annunciator I4/2, ICWP A/B/C TRIP, alarms
- Annunciator I4/4, ICW HEADER A/B LO PRESS, alarms
- Ultimate Heat Sink Temperature on TI-3/4-3605 is 96°F.
- 3A1 and 3A2 Traveling Screen Wash Pumps are ON.
- 3A ICW Pump RED light is lit.

Which ONE of the following completes the statement below?

In accordance with 3-ARP-097.CR.I, the RCO (1) start the 3C ICW pump and refer to (2) to address the given conditions?

- A. (1) will
(2) 3-ONOP-011.1, Intake Canal Low Level or High Temperature
- B. (1) will NOT
(2) 3-ONOP-011.1, Intake Canal Low Level or High Temperature
- C. (1) will
(2) 3-ONOP-019, Intake Cooling Water Malfunction
- D. (1) will NOT
(2) 3-ONOP-019, Intake Cooling Water Malfunction

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QUESTION 27

Given the following conditions:

- Units 3 and 4 are at 100% power.
- The 4CM Instrument Air compressor is out for maintenance.
- 3CM is running in LEAD, 3CD is in LAG, 4CD is in STANDBY-LAG.
- The Instrument Air systems are cross-tied.

Subsequently:

- Unit 4 Annunciators I6/1, INSTR AIR SYSTEM HI TEMP/LO PRESS, alarms.

Which ONE of the following identifies (1) the setpoint at which CV-4-1605, UNIT 4 Instrument Air Crosstie Isolation Control Valve, closes to protect Unit 4 and (2) the minimum pressure on PI-4-1444, INST AIR PRESS, if it cannot be maintained, when a Unit 4 Reactor Trip is required per 0-ONOP-013, Loss of Instrument Air?

- A. (1) 80 psig
(2) 60 psig
- B. (1) 80 psig
(2) 65 psig
- C. (1) 90 psig
(2) 60 psig
- D. (1) 90 psig
(2) 65 psig

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QUESTION 28

Given the following conditions:

- Unit 3 is in MODE 1.
- An emergent Containment Entry is in progress to investigate RCS leakage.
- 0-ADM-009, Containment Entries When Containment Integrity Is Established, is in effect.

NOTES

- TS LCO 3.6.1.3 is Containment Air Locks.

Which ONE of the following completes the following statements?

TS LCO 3.6.1.3 (1) satisfied with a single Personnel Hatch door open for transit.

In accordance with 0-ADM-009, the RCO (2) required to log the status of Personnel Hatch Inner and Outer Door indication from VPB.

- A. (1) is not
(2) is not
- B. (1) is
(2) is not
- C. (1) is
(2) is
- D. (1) is not
(2) is

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QUESTION 29

Given the following conditions:

- Unit 3 is in a refueling outage.
- The Upper Internals Assembly is being lifted out of the Reactor Vessel to set into the lower cavity.

Subsequently:

- B4/1, SOURCE RANGE HI FLUX AT SHUTDOWN, alarms.

The Source Range high flux alarm (1) initiate a containment evacuation alarm.

If flux continues to increase, B4/1 (2) require the operators to initiate Emergency Boration.

- A. (1) will
(2) does
- B. (1) will
(2) does NOT
- C. (1) will NOT
(2) does
- D. (1) will NOT
(2) does NOT

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PTN L-15-1 NRC EXAM

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QUESTION 30

Given the following conditions:

- Unit 4 is in MODE 5.
- Letdown is in service.
- The RCS is solid.
- A plant cooldown and depressurization is in progress.

Which ONE of the following describes the action to lower RCS pressure?

- A. Raise the setpoint of HCV-4-142, RHR L/D to CVCS.
- B. Raise the setpoint of PCV-4-145, Low Pressure LTDN Controller.
- C. Lower the setpoint of HCV-4-142, RHR L/D to CVCS.
- D. Lower the setpoint of PCV-4-145, Low Pressure LTDN Controller.

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QUESTION 31

Given the following conditions:

- Unit 3 trips due to Loss of Offsite Power.
- 3A and 3B EDGs energize their respective 4KV buses.
- 3-EOP-ES-0.2, Natural Circulation Cooldown, is being implemented.
- Automatic SI is blocked in accordance with 3-EOP-ES-0.2.

Subsequently during the cooldown:

- Average CET temperature is 586°F and rising.
- RCS pressure is 1335 psig and rapidly lowering.
- Pressurizer level is 11% and lowering.

Which ONE of the following actions is required in accordance with 3-EOP-ES-0.2?

- A. Verify PORVs are closed.
- B. Initiate SI and Phase A.
- C. Start all available Charging Pumps.
- D. Energize all Pressurizer heaters.

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PTN L-15-1 NRC EXAM

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QUESTION 32

Given the following conditions:

- A Unit 3 Reactor Startup is in progress.
- Power level is 1×10^{-8} amps.

Subsequently:

- 120V Vital Instrument Panel 3P07 is lost.

Which ONE of the following completes the following statement?

A Reactor Trip occurs due to the loss of (1), and as power lowers below $10E-10$ amps (2) will energize.

- A. (1) IR NI-35
(2) SR NI-31
- B. (1) IR NI-35
(2) SR NI-32
- C. (1) IR NI-36
(2) SR NI-31
- D. (1) IR NI-36
(2) SR NI-32

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PTN L-15-1 NRC EXAM

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QUESTION 33

Given the following conditions:

- Unit 4 is in MODE 6.
- The crew has commenced rod unlatching.

Subsequently:

- Annunciator I4/6, CNTMT SUMP HI LEVEL, alarms.
- Reactor Cavity water level is 56 feet and lowering.
- Bubbles are rising around the outside of the Reactor Vessel.
- R-11, Containment Air Particulate Monitor, is in alarm

Which ONE of the following completes the following statements based on the given conditions?

Containment Purge Isolation valves (1) required to be closed in accordance with 4-ONOP-033.2, Refueling Cavity Seal Failure.

Normal Containment Coolers are (2) secured.

- A. (1) are
(2) automatically
- B. (1) are NOT
(2) manually
- C. (1) are
(2) manually
- D. (1) are NOT
(2) automatically

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PTN L-15-1 NRC EXAM

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QUESTION 34

Given the following conditions:

- A Unit 3 startup is in progress.
- The crew prepares to synch the Main Generator to the grid.
- Rod Control is in Manual.
- Steam Dumps to Condenser Mode Selector Switch is in Manual.
- Tavg is 550°F.
- Tref is 548°F.
- Power Level is 7%.

Subsequently:

- The Turbine trips.

Which ONE of the following completes the following sentence for the initial plant response to the Turbine trip?

The Condenser Steam Dumps will _____.

- A. modulate open on the Turbine trip program
- B. quick open on the Turbine trip program
- C. quick open on the Steam Header pressure
- D. modulate open on the Steam Header pressure

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PTN L-15-1 NRC EXAM

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QUESTION 35

Given the following conditions:

- Unit 3 is at 100% power

Subsequently:

- BOP manually trips the Turbine due to high vibration.

Which ONE of the following identifies how secondary system parameters respond?

PT-3-447, Turbine Inlet Pressure, (1) and Condenser Steam Dumps will reduce RCS temperature (2).

- A. (1) lowers
(2) to no load Tavg
- B. (1) rises
(2) to no load Tavg
- C. (1) lowers
(2) to 5°F of program
- D. (1) rises
(2) to 5°F of program

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PTN L-15-1 NRC EXAM

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QUESTION 36

Given the following conditions:

- Unit 4 is at 100% power.
- Main Condenser vacuum is lowering.
- The crew enters 4-ONOP-014, Main Condenser Loss of Vacuum.

Which ONE of the following describes (1) the initial required action and (2) radiation monitoring capabilities?

- A. (1) Place the Steam Jet Air Ejector Hogging Jet in service.
(2) SJAЕ SPING is not OPERABLE.
- B. (1) Place the Steam Jet Air Ejector Hogging Jet in service.
(2) SJAЕ SPING is OPERABLE.
- C. (1) Place the Standby Steam Jet Air Ejector in service.
(2) SJAЕ SPING is not OPERABLE.
- D. (1) Place the Standby Steam Jet Air Ejector in service.
(2) SJAЕ SPING is OPERABLE.

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PTN L-15-1 NRC EXAM

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QUESTION 37

Which ONE of the following is a condition which causes X6/2, RADWASTE BLDG PANEL C46 TROUBLE, to alarm in the Control Room?

- A. Spent Resin Storage Tank High Level
- B. Waste Monitor Tank A High Level
- C. Reactor Coolant Drain Tank Unit 3 High Level
- D. Waste Liquid High Radiation

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PTN L-15-1 NRC EXAM

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QUESTION 38

Given the following conditions:

- Unit 3 startup is in progress per 3-GOP-301, Hot Standby to Power Operation.
- Unit 3 is at 25% power and stable.
- RCS Boron Concentration is 1274 ppm.
- Turbine is in MW Control with the following Control Valve positions:
 - CV-UL (#1) 3% CV-UR (#2) 3%
 - CV-LL (#3) 14.5% CV-LR (#4) 14.5%

Subsequently:

- CV-3-1606, A S/G Atmospheric Steam Dump fails open.

Which ONE of the following identifies the INITIAL response of the plant as result of this failure, with no operator action?

- A. Main Generator electrical output will remain constant at 25% with the Turbine in MW control.
- B. Reactor Coolant Hot Leg temperature, as read on TR-3-413, rises and results in a decrease in subcooling.
- C. The Turbine Control valves, CV-LL and CV-LR, move to maintain constant Turbine Inlet Pressure.
- D. Reactor Coolant Cold Leg temperature, as read on TR-3-410, lowers and results in a Reactor power rise.

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PTN L-15-1 NRC EXAM

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QUESTION 39

Given the following plant conditions:

- Unit 3 trips from 100% power on a spurious SI actuation.
- Crew enters 3-EOP-E-0, Reactor Trip or Safety Injection.
- RCS pressure is 2150 psig and recovering.
- RCS Tavg stabilizes at 535°F.

Which ONE of the following correctly completes the following statement?

To ensure adequate Shutdown Margin, a minimum boration rate of (1) is required until Shutdown Margin is verified.

From the time of the trip, with no operator action, Shutdown Margin will be (2) after eighteen (18) hours.

- A. (1) 20 gpm
(2) higher
- B. (1) 45 gpm
(2) higher
- C. (1) 20 gpm
(2) lower
- D. (1) 45 gpm
(2) lower

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PTN L-15-1 NRC EXAM

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QUESTION 40

Given the following conditions:

- A loss of all AC power occurs on Unit 3.
- The crew enters 3-EOP-ECA-0.0, Loss of All AC Power.
- Pressurizer level is 5% and lowering.
- Containment Sump level is rising.
- The crew prepares to depressurize the SGs.

Which ONE of the following actions (1) ensures natural circulation or reflux boiling cooling is sufficient and (2) lists the applicable recovery procedure after power is restored?

- A. (1) Verify total AFW flow between 400 and 450 gpm.
(2) 3-EOP-ECA-0.1, Loss of All Power Recovery Without SI Required
- B. (1) Maintain >7% Narrow Range Level in at least one SG.
(2) 3-EOP-ECA-0.1, Loss of All Power Recovery Without SI Required
- C. (1) Verify total AFW flow between 400 and 450 gpm.
(2) 3-EOP-ECA-0.2, Loss of All Power Recovery With SI Required
- D. (1) Maintain >7% Narrow Range Level in at least one SG.
(2) 3-EOP-ECA-0.2, Loss of All Power Recovery With SI Required

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PTN L-15-1 NRC EXAM

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QUESTION 41

Given the following conditions:

- Unit 3 Reactor trips and Safety Injection actuates.
- 3A 4KV Bus is de-energized due to an overcurrent condition.
- All RCPs are tripped in accordance with the Fold-Out page.
- The crew is performing Attachment 3 of 3-EOP-E-0, Reactor Trip or Safety Injection.
- All other safeguards equipment functions normally.

Which ONE of the following completes the following statement?

In accordance with Attachment 3 of 3-EOP-E-0, the RCO (1) manually start the Normal Containment Coolers.

The 3B ECC (2) automatically start.

- A. (1) will
(2) will
- B. (1) will
(2) will NOT
- C. (1) will NOT
(2) will NOT
- D. (1) will NOT
(2) will

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PTN L-15-1 NRC EXAM

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QUESTION 42

Given the following conditions:

- Unit 4 is at 38% power and stable.
- 4C Loop RCS Flow Meters are: 86%, 89%, and 91%.
- 4C RCP motor frame vibration is 3 mils.
- 4C RCP Oil Reservoir level $\frac{1}{2}$ " below normal.

Which ONE of the following identifies the crew response in accordance with 4-ONOP-041.1, Reactor Coolant Pump Off-Normal, to the given conditions?

- A. No actions required to trip 4C RCP or Reactor or reduce load.
- B. A Reactor trip and 4C RCP trip are required.
- C. A Reactor trip is required and 4C RCP trip is NOT required.
- D. A Fast Load Reduction is required.

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PTN L-15-1 NRC EXAM

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QUESTION 43

Given the following conditions:

- Unit 3 is at 100% power with 3A Charging Pump running.
- HCV-3-121, Charging Flow To Regen Heat Exchanger, is throttled.

Subsequently:

- Annunciator A6/5, RCP LABYRINTH SEAL LO ΔP , alarms.
- Annunciator A5/1, CHARGING PUMP A TRIP, alarms.
- 3C Charging Pump is started in accordance with 3-ONOP-047.1, Loss of Charging Flow.
- The RCO balances Charging and Letdown flows, and establishes a stable VCT level.
- RCP Seal Injection flows are 4 to 5 gpm.

Which ONE of the following completes the following sentence?

IAW 3-ONOP-041.01, Reactor Coolant Pump Off-Normal, the first action to address the given conditions is _____.

- A. locally throttle open 3-297A/B/C RCP Seal Injection Valves
- B. manually raise 3C Charging pump speed
- C. manually throttle closed HCV-3-121
- D. manually throttle open HCV-3-121

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PTN L-15-1 NRC EXAM

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QUESTION 44

Given the following conditions:

- Unit 3 is in MODE 4 in 3-GOP-305, Hot Standby to Cold Shutdown.
- Unit 3 is cooling down with the 3B RHR loop in accordance with 3-OP-050, Residual Heat Removal System.
- 3A CCW Heat Exchanger is out of service for maintenance with required pumps in pull-to-lock.

Subsequently:

- The 3B CCW pump breaker trips on motor overload, causing an electrical transient resulting in a momentary loss of the 3B 4KV Bus.
- 3B 4KV Bus power is restored on the 3B EDG.

Which ONE of the following completes the statement below?

Shutdown cooling is restored when 3B RHR pump (1) and 3C CCW Pump (2) .

- A. (1) auto starts from the sequencer
(2) auto starts from the sequencer
- B. (1) is manually started
(2) auto starts from the sequencer
- C. (1) auto starts from the sequencer
(2) is manually started
- D. (1) is manually started
(2) is manually started

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PTN L-15-1 NRC EXAM

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QUESTION 45

Given the following conditions:

- Unit 4 is at 80% power.
- 4B Steam Generator Feed Pump's breaker trips.

Subsequently

- PC-4-444J, Pressurizer Pressure Controller, does not respond in automatic.

Which ONE of the following completes the sentences below?

PC-4-444J's output is required to be initially (1) in accordance with 4-ONOP-041.5 Pressurizer Pressure Control Malfunction. PORV (2) may be operated by controller PC-4-444J.

- A. (1) raised
(2) PCV-4-455C
- B. (1) raised
(2) PCV-4-456
- C. (1) lowered
(2) PCV-4-456
- D. (1) lowered
(2) PCV-4-455C

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PTN L-15-1 NRC EXAM

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QUESTION 46

Given the following conditions:

- Unit 3 experiences an Anticipated Transient without Scram (ATWS).
- 3-EOP-FR-S.1, Response To Nuclear Power Generation/ATWS is in progress.
- The RCO is initiating Emergency Boration.
- Neither 3A nor 3B Boric Acid Pumps starts.

Which ONE of the following describes the required response to initiate Emergency Boration?

- A. Open MOV-3-350, Emergency Boration Valve.
- B. Close LCV-3-115C, VCT Outlet to Charging Pump Suction.
- C. Close FCV-3-113B, Blender to Charging Pump.
- D. Open 3-356, Manual Emergency Boration Valve.

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PTN L-15-1 NRC EXAM

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QUESTION 47

Given the following conditions:

- A SGTR occurs on 3B S/G.
- 3-EOP-E-3, Steam Generator Tube Rupture is in progress.
- A cooldown is commenced to target temperature.

Subsequently:

- RCS Subcooling lowers to 14°F with no other accidents in progress.

Which ONE of the following identifies whether or not the RCPs should be secured and why?

- A. Trip the Reactor Coolant Pumps to minimize the potential for RCP damage when an RCS depressurization is initiated.
- B. Trip the Reactor Coolant Pump on the affected loop to minimize RCS inventory loss.
- C. Keep the Reactor Coolant Pumps running to prevent the automatic opening of the SDTA.
- D. Keep the Reactor Coolant Pumps running because a controlled RCS cooldown is in progress.

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PTN L-15-1 NRC EXAM

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QUESTION 48

Given the following conditions:

- Unit 3 is at 100% power.
- 3A EDG is out of service for maintenance.
- A fault on the Unit 3 Startup Transformer generates a Transformer Lockout and a 3B 4KV Bus Lockout which do not reset.
- 3-EOP-ECA-0.0, Loss of All AC Power, is in progress.

Which ONE of the following statements identifies (1) the preferred source of power restoration in accordance with 3-EOP-ECA-0.0 and (2) the design basis battery duration?

- A. (1) 3C 4KV Bus
(2) 30 minutes
- B. (1) 3C 4KV Bus
(2) 2 hours
- C. (1) Unit 4 Startup Transformer
(2) 30 minutes
- D. (1) Unit 4 Startup Transformer
(2) 2 hours

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PTN L-15-1 NRC EXAM

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QUESTION 49

Given the following conditions:

Unit 3 is at 3% power while performing a reactor startup.

Vital Instrument Panel 3P06 loses power.

3-ONOP-003.6, Loss of 120V Vital Instrument Panel 3P06, is in progress.

Which ONE of the following completes the statements below?

The Pressurizer Control Heaters ___(1)___ . 3-ONOP-003.6 directs the reduction of Charging flow to reduce the PRZ fill rate to prevent ___(2)___.

- A. (1) remain on
(2) lifting a PRZ PORV
- B. (1) de-energize
(2) lifting a PRZ PORV
- C. (1) remain on
(2) a High PRZ level trip
- D. (1) de-energize
(2) a High PRZ level trip

PTN L-15-1 NRC EXAM

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QUESTION 50

Given the following conditions:

- Unit 3 is in MODE 3.
- Vital 480V MCC 3B is out of service.

Subsequently:

- Vital DC Bus 3D23 loses power due to a fault on the bus.
- The crew is restoring power to DC Bus 3D23 in accordance with 3-ONOP-003.5, Loss of DC Bus 3D23 and 3D23A (3B).
- The fault has been isolated.

Which ONE of the following identifies (1) the battery charger that is still OPERABLE and (2) the expected battery charger voltage?

- A. (1) 3B1
(2) 120 volts
- B. (1) 3B1
(2) 135 volts
- C. (1) 3B2
(2) 135 volts
- D. (1) 3B2
(2) 120 volts

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PTN L-15-1 NRC EXAM

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QUESTION 51

Given the following conditions:

- Unit 3 is operating at 95% power.
- ICW Header pressure is 8 psig on PI-3-1619 and PI-3-1620.
- 3A1 and 3A2 Traveling Screen Differential pressures are 12.7 inches water on R-3-2300 TWS DP Recorder.
- TPCW Discharge pressure is 98 psig PI-3-1468.
- CCW Header Supply pressure is 105 psig on PI-3-640.

Which ONE of the following procedures is required to be entered?

- A. 3-ONOP-019, Intake Cooling Water Malfunction
- B. 3-ONOP-030, Component Cooling Water Malfunction
- C. 3-ONOP-008, Turbine Plant Cooling Water Malfunction
- D. 3-ONOP-011, Screen Wash System/Intake Malfunction

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PTN L-15-1 NRC EXAM

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QUESTION 52

Given the following conditions:

- Unit 3 is at 95% power.
- The TCS MVARs are oscillating.
- The Switchyard voltage is stable.
- Voltage Regulator Selector Switch is ON.

Subsequently:

- The crew enters 3-ONOP-090, Abnormal Generator MW/MVAR Oscillation.
- The U3 Turbine Operator reports the Minimum Excitation Module #5 light is lit.

Which ONE of the following describes the operator's response to stabilize the Main Generator?

- A. Raise voltage using the Main Generator AC Voltage Regulator.
- B. Raise voltage using the Main Generator DC Voltage Regulator.
- C. Lower voltage using the Main Generator AC Voltage Regulator.
- D. Lower voltage using the Main Generator DC Voltage Regulator.

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PTN L-15-1 NRC EXAM

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QUESTION 53

Given the following conditions:

- 3-EOP-ECA-1.2, LOCA Outside Containment, has been entered.
- The crew closed MOV-3-744A & B, RHR Discharge to Cold Leg Isolation Valves.
- The leak is between the 3B RHR Heat Exchanger and 3-HCV-758, RHR HX Outlet Flow Control Valve.

Which ONE of the following completes the following statement?

In accordance with 3-EOP-ECA-1.2, isolation of the LOCA outside containment can be verified based on (1).

Local operator actions (2) for Alternate RHR to be available for plant cooldown.

- A. (1) increasing RCS pressure
(2) are required
- B. (1) increasing RCS pressure
(2) are NOT required
- C. (1) decreasing Auxiliary Building radiation
(2) are required
- D. (1) decreasing Auxiliary Building radiation
(2) are NOT required

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PTN L-15-1 NRC EXAM

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QUESTION 54

Given the following conditions:

- The Reactor trips due to a loss of offsite power.
- A loss of all feedwater occurs.
- 3-EOP-FR-H.1, Loss of Secondary Heat Sink, is in progress.
- A source of feedwater is NOT restored.
- The crew is establishing RCS bleed and feed when only one Pressurizer PORV opens.

Which ONE of the following correctly completes the statement below?

Based on these plant conditions, the RCS bleed path is (1) and the crew should (2), while continuing efforts to re-establish a source of feedwater to the SGs.

- A. (1) adequate
(2) depressurize SGs to less than 360 psig
- B. (1) adequate
(2) open all RCS Vents
- C. (1) inadequate
(2) depressurize SGs to less than 360 psig
- D. (1) inadequate
(2) open all RCS Vents

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PTN L-15-1 NRC EXAM

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QUESTION 55

Given the following conditions:

- Unit 3 trips from 100% power due to a 3A 4KV Bus lockout.
- Post trip, a large break LOCA develops.
- While performing 3-EOP-ES-1.3, Transfer To Cold Leg Recirculation, 3B RHR Pump trips and cannot be restarted.
- Containment pressure peaked at 22 psig and is now 18 psig.
- 2 Emergency Containment Coolers are running.
- RWST level is at 55,000 gallons.

Which ONE of the following completes the statements below?

The Containment Spray Pumps must be operated in accordance with (1).

(2) Containment Spray Pump(s) is/are required to be operating.

NOTE

3-EOP-FR-Z.1, Response to High Containment Pressure

3-EOP-ECA-1.1, Loss of Emergency Coolant Recirculation

- A. (1) 3-EOP-FR-Z.1
(2) One
- B. (1) 3-EOP- FR-Z.1
(2) Zero
- C. (1) 3-EOP-ECA-1.1
(2) One
- D. (1) 3-EOP-ECA-1.1
(2) Zero

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PTN L-15-1 NRC EXAM

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QUESTION 56

Given the following conditions:

- 3-EOP-ECA-2.1, Uncontrolled Depressurization of All Steam Generators, is in progress.
- RCS temperature decreases from 547°F to 422°F in the last hour.
- The crew adjusts AFW flow.
- SG NR levels are all off-scale low.
- 3A SG Safety Valve reseats.

Which ONE of the following describes the AFW flow requirement and the action for the next procedure transition?

- A.
 - AFW flow is at a minimum of 50 gpm per SG.
 - Immediately transition to 3-EOP-E-2, Faulted SG Isolation.
- B.
 - AFW flow is at a minimum of 400 gpm per SG.
 - Immediately transition to 3-EOP-E-2, Faulted SG Isolation.
- C.
 - AFW flow is at a minimum of 50 gpm per SG.
 - When 3A SG pressure rises, transition to 3-EOP-E-2, Faulted SG Isolation.
- D.
 - AFW flow is at a minimum of 400 gpm per SG.
 - When 3A SG pressure rises, transition to 3-EOP-E-2, Faulted SG Isolation.

PTN L-15-1 NRC EXAM

This information is controlled by PTN's 2015 LOIT (L-15-1) NRC EXAMINATION SECURITY AGREEMENT.

QUESTION 57

Given the following conditions:

- Unit 4 is at 50% steady state power.
- T_{avg} is matched with T_{ref} at 562°F.
- Automatic VCT makeup is in progress.
- TCS is in MW control at 410 MW.
- The Rod Motion Control Selector Switch is placed in AUTO after moving Control Bank D to 161 steps.
- The Axial Flux Difference is -3 when the Rod Motion Control Selector Switch is placed in AUTO.

Subsequently, a few minutes later:

- Unit 4 is at 51% and increasing.
- T_{avg} is 3°F higher than T_{ref} .
- TCS is in MW control at 412 MW with Turbine Control Valves closing.
- The Axial Flux Difference is -0.5 and trending more positive.

Which ONE of the following completes the following statement?

The conditions identified above are due to a(n) (1) and the applicable ONOP will subsequently (2) to restore T_{avg} .

- A. (1) inadvertent dilution
(2) adjust turbine load
- B. (1) continuous rod withdrawal
(2) adjust turbine load
- C. (1) inadvertent dilution
(2) insert control rods
- D. (1) continuous rod withdrawal
(2) insert control rods

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PTN L-15-1 NRC EXAM

This information is controlled by PTN's 2015 LOIT (L-15-1) NRC EXAMINATION SECURITY AGREEMENT.

QUESTION 58

Given the following conditions:

- Loss of Offsite Power occurs on Unit 3.
- 3A and 3B EDGs are powering the 3A and 3B 4KV Buses.
- 3-EOP-ES-0.4, Natural Circulation Cooldown with Steam Void in Vessel (Without RVLMS) is in progress.
- RCS pressure is 1635 psig.
- Pressurizer level is 30%.
- Prior to depressurizing the RCS, Pressurizer PORV PCV-3-456 fails open.

Which ONE of the following identifies the reason for the initial rapidly increasing Pressurizer level during this event?

- A. The steam space in the Pressurizer collapses allowing more makeup to be injected immediately into the RCS by the HHSI Pumps.
- B. Pressurizer level reference legs flash which results in an increase in indicated level.
- C. Safety Injection Accumulators inject into the RCS which increases Pressurizer level.
- D. Reactor upper head region voiding occurs which results in mass transfer from the Reactor Head to the Pressurizer.

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PTN L-15-1 NRC EXAM

This information is controlled by PTN's 2015 LOIT (L-15-1) NRC EXAMINATION SECURITY AGREEMENT.

QUESTION 59

Given the following conditions:

- Unit 3 is at 100% power.

Subsequently:

- E5/3, CONDENSER LO VACUUM, alarms.
- Crew enters 3-ONOP-014, Main Condenser Loss of Vacuum.
- The Crew commences a Fast Load Reduction IAW 3-GOP-100, Fast Load Reduction.
- Operators are evaluating Annunciator B8/1, ROD BANK LO LIMIT which has alarmed.

Which ONE of the following describes NEXT action required by 3-GOP-100?

- A. Place Control Rods in Manual.
- B. Borate the RCS at least 16 gpm.
- C. Slow or stop the load reduction.
- D. Trip the Reactor and enter 3-EOP-E-0.

This information is controlled by PTN's 2015 LOIT (L-15-1) NRC EXAMINATION SECURITY AGREEMENT.

PTN L-15-1 NRC EXAM

This information is controlled by PTN's 2015 LOIT (L-15-1) NRC EXAMINATION SECURITY AGREEMENT.

QUESTION 60

Given the following conditions:

- Waste Gas Decay Tank D contains high-activity gas.
- Waste Gas Decay Tank D Relief Valve develops a flange leak that is slowly dispersing into the Aux Bldg.
- R-14, Plant Vent Gas Monitor, alarms.
- Crew enters 3-ONOP-067, Radioactive Effluent Release.

Which ONE of the following identifies the plant response to an R-14 alarm, if any occurs?

- A. Aux Bldg Exhaust Fans trip.
- B. Aux Bldg Supply Fans trip.
- C. No effect on Aux Bldg or Control Room Ventilation.
- D. Control Room Ventilation shifts to recirculation mode.

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PTN L-15-1 NRC EXAM

This information is controlled by PTN's 2015 LOIT (L-15-1) NRC EXAMINATION SECURITY AGREEMENT.

QUESTION 61

Given the following conditions:

- A fire was confirmed in the Cable Spreading Room that was affecting plant equipment.
- The crew is implementing 0-ONOP-105, Control Room Evacuation.
- The site has a loss of offsite power.
- All emergency safeguards equipment operates as required.

Which ONE of the following identifies the EDG operation in accordance with 0-ONOP-105?

At the point when Control of Shutdown Systems is established, the _____.

- A. 3A EDG will remain loaded
- B. 4B EDG will remain loaded
- C. 3A EDG will need to be shutdown
- D. 4B EDG will need to be shutdown

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PTN L-15-1 NRC EXAM

This information is controlled by PTN's 2015 LOIT (L-15-1) NRC EXAMINATION SECURITY AGREEMENT.

QUESTION 62

Given the following conditions:

- Unit 3 is currently at 100% preparing for a shutdown due to high RCS Activity levels.

Subsequently:

- A RCS leak of 50 gpm continues to increase inside containment.

Which ONE of the following identifies which process radiation monitors will show a continued elevated trend after an automatic safety injection actuation?

NOTE

- R-3-11 Containment Air Particulate
- R-3-12 Containment Air Gas
- R-14 Plant Stack Gas

- A. R-3-11 Yes, R-3-12 Yes, R-14 Yes
- B. R-3-11 Yes, R-3-12 Yes, R-14 No
- C. R-3-11 No, R-3-12 No, R-14 Yes
- D. R-3-11 No, R-3-12 No, R-14 No

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PTN L-15-1 NRC EXAM

This information is controlled by PTN's 2015 LOIT (L-15-1) NRC EXAMINATION SECURITY AGREEMENT.

QUESTION 63

Given the following conditions:

- Unit 3 experiences a Safety Injection.
- Total AFW flow is throttled to 450 gpm.
- The crew transitions from 3-EOP-E-0, Reactor Trip or Safety Injection, to 3-EOP-E-1, Loss of Reactor or Secondary Coolant.
- The crew is determining SI Termination criteria with the following:
 - Containment temperature is 165°F and slowly decreasing.
 - Pressurizer level is 17% and rising.
 - RCS subcooling is 58°F and stable.
 - RCS pressure is 1550 psig and stable.
 - SG Levels are 5% and rising.

Which ONE of the following identifies the correct operator response and the reason?

- A. Terminate SI since all criteria is satisfied.
- B. Do NOT terminate SI since RCS pressure is too low.
- C. Do NOT terminate SI since SG levels are too low.
- D. Do NOT terminate SI since PZR level is too low.

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PTN L-15-1 NRC EXAM

This information is controlled by PTN's 2015 LOIT (L-15-1) NRC EXAMINATION SECURITY AGREEMENT.

QUESTION 64

Given the following conditions:

- Unit 3 experiences a major Steam Line Break inside containment concurrent with a Loss of Off-Site Power.
- 3A and 3B 4KV Buses are powered from the Emergency Diesel Generators.
- Containment Pressure Hi signal is actuated.
- Containment Temperature is 193°F.
- The crew is performing 3-EOP-FR-P.1, Response to Imminent Pressurized Thermal Shock.
- Both HHSI and RHR Pumps have been stopped and placed in AUTO.
- 3A Charging Pump is running with 40 gpm Charging flow.
- Letdown is unavailable.

Which ONE of the following is (1) the preferred method to depressurize the plant and (2) the earliest allowable CET subcooling temperature to terminate the depressurization?

- A. (1) Use Auxiliary Spray
(2) 28°F
- B. (1) Use Auxiliary Spray
(2) 82°F
- C. (1) Open one Pressurizer PORV
(2) 28°F
- D. (1) Open one Pressurizer PORV
(2) 82°F

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PTN L-15-1 NRC EXAM

This information is controlled by PTN's 2015 LOIT (L-15-1) NRC EXAMINATION SECURITY AGREEMENT.

QUESTION 65

Given the following conditions:

- Unit 4 Reactor trips due to a loss of offsite power.
- The crew performs 4-EOP-ES-0.3, Natural Circulation Cooldown with Steam Void in Vessel (With RVLMS).

Subsequently:

- Train 4A RVLMS fails.
- Pressurizer level is 6% and lowering rapidly.

Which ONE of the following identifies the required procedural response?

- A. Remain in 4-EOP-ES-0.3, Natural Circulation Cooldown with Steam Void in Vessel (With RVLMS).
- B. Transition to 4-EOP-E-1, Loss of Reactor or Secondary Coolant.
- C. Transition to 4-EOP-ES-0.0, Rediagnosis.
- D. Transition to 4-EOP-E-0, Reactor Trip or Safety Injection.

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PTN L-15-1 NRC EXAM

This information is controlled by PTN's 2015 LOIT (L-15-1) NRC EXAMINATION SECURITY AGREEMENT.

QUESTION 66

Which ONE of the following identifies the required reviews prior to assuming the Unit 3 RCO responsibility in accordance with 0-ADM-202, Shift Relief and Turnover?

- A. Check LMS for quals, review eSOMs for clearances, and review Schedule of Plant Checks and Surveillances (Red Book).
- B. Review TSA log book (TCC Index), review Schedule of Plant Checks and Surveillances (Red Book), and perform a Minimum Equipment List check.
- C. Check Watchstander Out of Service Book, review eSOMs for clearances, and perform a Minimum Equipment List check.
- D. Check Watchstander Out of Service Book, review TSA log book (TCC Index), and review Annunciator Status Log.

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PTN L-15-1 NRC EXAM

This information is controlled by PTN's 2015 LOIT (L-15-1) NRC EXAMINATION SECURITY AGREEMENT.

QUESTION 67

Which ONE of the following completes the following statements?

The Tech Spec minimum required Spent Fuel Pool water level is (1) .

While raising the fuel assembly in the Spent Fuel Pool, the Bridge Crane hoist stops when the load limit of (2) lbs. has been reached.

- A. (1) 57'
(2) 2075
- B. (1) 56'10"
(2) 2075
- C. (1) 57'
(2) 4000
- D. (1) 56'10"
(2) 4000

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PTN L-15-1 NRC EXAM

This information is controlled by PTN's 2015 LOIT (L-15-1) NRC EXAMINATION SECURITY AGREEMENT.

QUESTION 68

Which ONE of the following identifies the individual(s) who may enter a Guarded Area per OP-AA-102-1003, Guarded Equipment, without requesting permission from the Control Room and first completing OP-AA-102-1003-F01, Protected/Guarded Equipment Work Approval Form?

A Security Officer performing official rounds (1) required to obtain permission.

A Maintenance Supervisor performing a walkdown of a jobsite for an upcoming high risk activity (2) required to obtain permission.

- A. (1) is not
(2) is not
- B. (1) is not
(2) is
- C. (1) is
(2) is not
- D. (1) is
(2) is

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PTN L-15-1 NRC EXAM

This information is controlled by PTN's 2015 LOIT (L-15-1) NRC EXAMINATION SECURITY AGREEMENT.

QUESTION 69

Given the following plant conditions:

- Unit 4 is in MODE 5.
- 4A RHR loop is in operation.

Which ONE of the following is a criterion for the RCS Loop Filled requirement of 0-ADM-051, Outage Risk Assessment and Control?

- A. 4B RCP is operating.
- B. Reactor Coolant System pressure is 50 psig.
- C. Unit 4 Steam Generator Wide Range levels are greater than 10%.
- D. Unit 4 Steam Generator Narrow Range levels are greater than 10%.

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PTN L-15-1 NRC EXAM

This information is controlled by PTN's 2015 LOIT (L-15-1) NRC EXAMINATION SECURITY AGREEMENT.

QUESTION 70

Given the following conditions:

- Unit 3 is in MODE 6 with a core onload and containment purge in progress.
- The refueling crew is lowering an irradiated fuel assembly into the core, when the assembly is inadvertently dropped.
- Annunciators X4/1, ARMS HI RADIATION, and H1/4, PRMS HI RADIATION, are lit.
- The crew verifies that R-3-12, Containment Gas Monitor, and RI-3-1402B, Unit 3 Containment Operating Floor, are in alarm.
- No other annunciators have been received.

As a result of this event:

- (1) The Containment Evacuation Alarm will be (1) initiated.
- (2) A Control Ventilation Isolation will be (2) initiated.

- A. (1) automatically
(2) automatically
- B. (1) manually
(2) automatically
- C. (1) automatically
(2) manually
- D. (1) manually
(2) manually

This information is controlled by PTN's 2015 LOIT (L-15-1) NRC EXAMINATION SECURITY AGREEMENT.

PTN L-15-1 NRC EXAM

This information is controlled by PTN's 2015 LOIT (L-15-1) NRC EXAMINATION SECURITY AGREEMENT.

QUESTION 71

Given the following conditions:

- Unit 3 has just entered MODE 5 for a refueling outage.
- All equipment is in a normal alignment for plant conditions.
- A containment purge has been initiated using the Unit 4 Purge Supply and Exhaust Fans.

Subsequently:

- Containment Radiation Monitor R-3-11, Containment Air Particulate Monitor, alarms.
- The crew enters 3-ONOP-067, Radioactive Effluent Release

Which ONE of the following identifies the required actions in accordance with 3-ONOP-067?

- A. (1) Check that Unit 4 Purge Supply and Exhaust Fans automatically trip
(2) Verify that TS-002, TSC EMER VENT AUTO INITIATE Switch is in INHIBIT
- B. (1) Manually stop the U4 Purge Supply and Exhaust Fans
(2) Verify that TS-002, TSC EMER VENT AUTO INITIATE Switch is in ENABLE
- C. (1) Check that Unit 4 Purge Supply and Exhaust Fans automatically trip
(2) Verify that TS-002, TSC EMER VENT AUTO INITIATE Switch is in ENABLE
- D. (1) Manually stop the U4 Purge Supply and Exhaust Fans
(2) Verify that TS-002, TSC EMER VENT AUTO INITIATE Switch is in INHIBIT

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PTN L-15-1 NRC EXAM

This information is controlled by PTN's 2015 LOIT (L-15-1) NRC EXAMINATION SECURITY AGREEMENT.

QUESTION 72

Given the following conditions:

- Unit 3 RCS temperature is 150°F.
- Time to boil in the RCS of 35 minutes.
- 3B RHR pump is Inoperable.

Subsequently,

- The 3A RHR pump trips and cannot be restarted.
- The crew enters 3-ONOP-050, Loss of RHR.

Which ONE of the following completes the statements below?

Containment Closure is required to be completed in (1) minutes, in accordance with 3-ONOP-050.

The equipment hatch is required to be closed with (2) bolts, in accordance with 0-ADM-051, Outage Risk Assessment and Control.

- A. (1) 30
(2) four
- B. (1) 30
(2) eight
- C. (1) 35
(2) four
- D. (1) 35
(2) eight

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PTN L-15-1 NRC EXAM

This information is controlled by PTN's 2015 LOIT (L-15-1) NRC EXAMINATION SECURITY AGREEMENT.

QUESTION 73

Given the following conditions:

- Alignment is complete for Unit 3 hot leg recirculation in accordance with 3-EOP-ES-1.4, Transfer to Hot Leg Recirculation.
- Plant conditions are stabilized for the accident in progress.
- The highest Critical Safety Function Status Tree (CSFST) is a yellow path on Inventory.

Which ONE of the following identifies the CSFST monitoring requirement in accordance with 3-EOP-F-0, Critical Safety Function Status Trees?

- A. Monitor continuously.
- B. Monitored every 10 to 20 minutes.
- C. Suspend monitoring since conditions are stable.
- D. Suspend monitoring immediately after completing 3-EOP-ES-1.4.

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PTN L-15-1 NRC EXAM

This information is controlled by PTN's 2015 LOIT (L-15-1) NRC EXAMINATION SECURITY AGREEMENT.

QUESTION 74

Given the following conditions:

- Unit 3 is at 100% power.
- Work is ongoing in the Unit 3 480V Load Center Rooms.
- The foreman requests the fire door between A & B and C & D Load Centers be opened to allow better air circulation for the worker's comfort.
- A Fire Protection Impairment has NOT been issued.

In response to the foreman's request, which ONE of the following identifies the policy concerning the propping open of fire doors in accordance with 0-ADM-016, Fire Protection Program?

- A. May be open without compensatory actions as long as work is ongoing.
- B. May be open if an hourly roving fire watch is provided.
- C. Cannot be opened solely for comfort of personnel.
- D. Cannot be opened for more than 30 minutes.

This information is controlled by PTN's 2015 LOIT (L-15-1) NRC EXAMINATION SECURITY AGREEMENT.

PTN L-15-1 NRC EXAM

This information is controlled by PTN's 2015 LOIT (L-15-1) NRC EXAMINATION SECURITY AGREEMENT.

QUESTION 75

Given the following conditions:

- Unit 3 is at 100% power.

Subsequently:

- Unit 3 experiences an accident with the following conditions:

	<u>Pressure</u>	<u>AFW Flow</u>
S/G A	950 psig and lowering	260 gpm
S/G B	290 psig and lowering	340 gpm
S/G C	940 psig and lowering	230 gpm

	<u>NR Level</u>	<u>WR Level</u>
S/G A	21.5 % and lowering	55.8% and lowering
S/G B	80.3 % and rising	55.9% and rising
S/G C	18.6 % and lowering	54.5% and lowering

Containment pressure: 21.5 psig and rising
Containment Sump level: 36.2 inches and rising
Pressurizer pressure: 1274 psig and lowering
Pressurizer level: 1 % and lowering
All safeguards equipment is operating as expected.

Based on the indicated parameters, which ONE of the following identifies the initial transition from 3-EOP-E-0, Reactor Trip or Safety Injection?

- A. 3-EOP-FR-Z.1, Response To High Containment Pressure
- B. 3-EOP-E-1, Loss of Reactor or Secondary Coolant
- C. 3-EOP-E-2, Faulted Steam Generator
- D. 3-EOP-E-3, Steam Generator Tube Rupture

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PTN L-15-1 NRC EXAM

This information is controlled by PTN's 2015 LOIT (L-15-1) NRC EXAMINATION SECURITY AGREEMENT.

QUESTION 76

Maintaining containment pressure at greater than or equal to the LCO lower pressure limits ensures that (1).

The Tech Spec 336 hour per calendar year time limit for containment average air temperature between 120 °F and 125 °F (2) allowed to be extended to credit times when containment temperatures are lower.

- A. (1) ECCS systems will have sufficient net positive suction head during the recirculation phase of a design basis accident
(2) is NOT
- B. (1) ECCS systems will have sufficient net positive suction head during the recirculation phase of a design basis accident.
(2) is
- C. (1) the containment structure is prevented from exceeding its design negative differential pressure limit
(2) is NOT
- D. (1) the containment structure is prevented from exceeding its design negative differential pressure limit
(2) is

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PTN L-15-1 NRC EXAM

This information is controlled by PTN's 2015 LOIT (L-15-1) NRC EXAMINATION SECURITY AGREEMENT.

QUESTION 77

Given the following conditions:

- Unit 3 is at 100% power.
- The following annunciators are lit:
X4/1 ARMS HI Radiation
H1/4 PRMS HI Radiation

STA reports that for the last 20 minutes the radiation monitors have been reading as follows:

- Plant Vent SPING RAD-6304 is in alarm at $2.9E-1$ $\mu\text{Ci/cc}$.
- Containment Mezzanine Area Radiation Monitor RI-3-1401B is in alarm at 15 mr/hr.
- Letdown Radiation Monitor R-3-20 is in alarm at 255 mr/hr.
- U3 Main Steam Line Radiation Monitor MSL-3-6426A is in alarm at $2.6E+2$ $\mu\text{Ci/cc}$.

Which ONE of the following is the event classification in accordance with 0-EPIP-20101, Duties of the Emergency Coordinator?

REFERENCE PROVIDED

- A. Unusual Event
- B. Alert
- C. Site Area Emergency
- D. General Emergency

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PTN L-15-1 NRC EXAM

This information is controlled by PTN's 2015 LOIT (L-15-1) NRC EXAMINATION SECURITY AGREEMENT.

QUESTION 78

Given the following conditions:

- Unit 3 is at 100% power.
- An equalizing charge is in progress on the 3B battery in accordance with 0-PME-003.16.
- The Common HVAC Unit (E16D) and the North DC Equipment/Inverter Room HVAC Unit (E16E) in the DC Equipment and Inverter Rooms have failed.
- DC Equipment and Inverter Rooms and electrolyte temperatures are at 100°F.
- 0-ONOP-025.3 is in progress.

Which ONE of the following completes the statements below?

The Vital Batteries 3A, 3B, 4A, and 4B and Spare Battery D52 are (1). The Shift Manager (2) required to invoke 10CFR50.54(x) and (y).

NOTE

- 0-PME-003.16, Individual Cell Equalizing Charge for Vital Batteries 3A, 3B, 4A, and 4B and Spare Battery D52
- 0-ONOP-025.3, DC Equipment and Inverter Rooms Supplemental Cooling

- A. (1) OPERABLE
(2) is NOT
- B. (1) OPERABLE
(2) is
- C. (1) NOT OPERABLE
(2) is NOT
- D. (1) NOT OPERABLE
(2) is

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PTN L-15-1 NRC EXAM

This information is controlled by PTN's 2015 LOIT (L-15-1) NRC EXAMINATION SECURITY AGREEMENT.

QUESTION 79

Given the following conditions:

- Unit 3 is at 100% power.
- 3A and 3C ICW Pumps are operating.

Subsequently:

- Annunciator I-4/4, ICW HEADER A/B LO PRESS, alarms.
- ICW Header Pressure Indicators, PI-3-1619 and PI-3-1620, indicate 0 psig.
- The crew enters 3-ONOP-019, Intake Cooling Water Malfunction.
- A pipe break on the 3A ICW/CCW Basket Strainer is isolated.

Which ONE of the following identifies if TS LCO 3.0.3 is met and if a plant shutdown is required to comply with TS after the 3A ICW/CCW Basket Strainer is isolated?

Prior to leak isolation, TS LCO 3.0.3 action(s) _____ (1) _____ required.

After the 3A ICW/CCW Basket Strainer leak is isolated and repairs are estimated to be completed in 5 days, the Unit _____ (2) _____ in a TS shutdown action statement.

- A. (1) are
(2) is
- B. (1) are
(2) is NOT
- C. (1) are NOT
(2) is
- D. (1) are NOT
(2) is NOT

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PTN L-15-1 NRC EXAM

This information is controlled by PTN's 2015 LOIT (L-15-1) NRC EXAMINATION SECURITY AGREEMENT.

QUESTION 80

Given the following conditions:

- Unit 3 was at 100% power with high activity in the RCS when a loss of coolant accident occurs inside Containment.
- MOV-3-381, RCP Seal Water Return and Excess Letdown Isolation fails to isolate.
- MOV-3-6386, RCP Seal Return Isolation fails to isolate.
- MOV-3-626, RCP Thermal Barrier CCW Outlet fails to isolate.
- RP reports elevated dose rates in the Auxiliary Building.
- The Plant Vent Stack radiation Monitor R-14 has risen from pre-event value of 300 cpm to current value of 6000 cpm.

Which ONE of the following choices identifies the minimum action required to isolate the Containment penetrations and if a release is in progress in accordance with 0-EPIP-20134, Offsite Notifications and Protective Action Recommendation?

- A.
 - close MOV-3-381 and MOV-3-626.
 - No Release
- B.
 - close MOV-3-381 and MOV-3-626.
 - Release
- C.
 - close MOV-3-6386 and MOV-3-381.
 - No Release
- D.
 - close MOV-3-6386 and MOV-3-381.
 - Release

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PTN L-15-1 NRC EXAM

This information is controlled by PTN's 2015 LOIT (L-15-1) NRC EXAMINATION SECURITY AGREEMENT.

QUESTION 81

Given the following conditions:

- Unit 3 is in MODE 2.
- A 2 gpm pinhole leak on the piping is discovered upstream of the first off RCS valve, 3-954B, RCS Hot Leg Sample Valve.
- 3-954B is closed.

Which ONE of the following states the required Tech Spec actions in accordance TS LCO 3.4.6.2, Reactor Coolant System Operational Leakage?

REFERENCE PROVIDED

- A. No Technical Specification actions are required.
- B. Isolate the sample line with 1 valve within 4 hours AND no further Technical Specification actions are required.
- C. Place Unit 3 in at least MODE 3 within 6 hours AND be in MODE 5 within the following 30 hours.
- D. Isolate the sample line with 2 valves within 4 hours AND no further Technical Specification actions are required.

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PTN L-15-1 NRC EXAM

This information is controlled by PTN's 2015 LOIT (L-15-1) NRC EXAMINATION SECURITY AGREEMENT.

QUESTION 82

Given the following conditions:

- Unit 3 is operating at 100% power.
- 3-OSP-059.10, Determination of Quadrant Power Tilt Ratio is complete.
- The Quadrant Power Tilt Ratio (QPTR) is 1.08.

Which ONE of the following completes the statements below?

Power is required to be lowered to (1) within two hours.

The Technical Specification action requires that the QUADRANT POWER TILT RATIO be calculated at least once (2) until either the QUADRANT POWER TILT RATIO is reduced to within its limit, or THERMAL POWER is reduced to less than 50% of RATED THERMAL POWER.

- A. (1) 76 %
(2) per hour
- B. (1) 76 %
(2) every two hours
- C. (1) 82 %
(2) per hour
- D. (1) 82 %
(2) every two hours

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PTN L-15-1 NRC EXAM

This information is controlled by PTN's 2015 LOIT (L-15-1) NRC EXAMINATION SECURITY AGREEMENT.

QUESTION 83

Given the following conditions:

- Unit 4 is at 100% power.
- 4C Steam Generator tube leakage is 20 gpd.
- Blowdown radiation monitor R-4-19 is Inoperable.

Which ONE of the following completes the statements below?

In accordance with the Offsite Dose Calculation Manual (ODCM), blowdown operations
___(1)___ with R-4-19 Inoperable.

R-4-19 inoperability affects the automatic isolation of ___(2)___.

- A. (1) are not permitted
(2) CV-4-6275A/B/C, Blowdown Isolation Valves
- B. (1) may continue with periodic grab samples
(2) CV-4-6275A/B/C, Blowdown Isolation Valves
- C. (1) are not permitted
(2) FCV-4-6278A/B/C, Blowdown Flow Control Valves
- D. (1) may continue with periodic grab samples
(2) FCV-4-6278A/B/C, Blowdown Flow Control Valves

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PTN L-15-1 NRC EXAM

This information is controlled by PTN's 2015 LOIT (L-15-1) NRC EXAMINATION SECURITY AGREEMENT.

QUESTION 84

Given the following conditions:

- Unit 3 is at 100% power.
- Unit 4 is at 10% power.
- B AFW Pump is OOS for repairs.

Subsequently:

- The switchyard is de-energized during severe weather.
- All emergency buses are powered from their respective EDGs.
- C AFW Pump trips on start.
- MOV-3-1405 fails to automatically open.
- The crew completed the reading of 3-EOP-E-0, Rx Trip or Safety Injection, Foldout Page.

Which ONE of the following completes the information below regarding the action to perform and why?

The Unit Supervisor _____ (1) _____ to start A AFW Pump and establish 340 gpm AFW flow to Unit 3 to minimize _____ (2) _____.

NOTE

MOV-3-1405, 3C Stm Supply To Aux Feedwater Pumps
AFSS-3-006/007, AFW Steam Supply Cross-connect valves

- A. (1) directs manually opening MOV-3-1405
(2) thermal stress due to excessive cooldown
- B. (1) must direct realigning AFSS-3-006 and 007
(2) the effects of an RCS heatup to prevent opening PZR PORVs
- C. (1) directs manually opening MOV-3-1405
(2) the effects of an RCS heatup to prevent opening PZR PORVs
- D. (1) must direct realigning AFSS-3-006 and 007
(2) thermal stress due to excessive cooldown

This information is controlled by PTN's 2015 LOIT (L-15-1) NRC EXAMINATION SECURITY AGREEMENT.

PTN L-15-1 NRC EXAM

This information is controlled by PTN's 2015 LOIT (L-15-1) NRC EXAMINATION SECURITY AGREEMENT.

QUESTION 85

Given the following conditions:

- Unit 3 is MODE 4, on RHR.

Subsequently:

- Pressurizer level is off-scale low.
- Annunciator H6/2, RHR HX HI/LO FLOW, alarms.
- R-3-11, Containment Air Particulate Monitor, is in alarm.
- The crew enters the appropriate off normal procedure.
- A leak is discovered downstream of RHR Discharge To Cold Leg Isolation Valves, MOV-4-744A and MOV-4-744B.

Which ONE of the following completes the statements below?

The steps containing the operator actions to isolate the leak are in (1).

The ONOP will (2) during leak isolation.

- A. (1) 3-ONOP-041.3, Excessive Reactor Coolant System Leakage
(2) stop the RHR pumps
- B. (1) 3-ONOP-041.3, Excessive Reactor Coolant System Leakage
(2) NOT stop the RHR pumps
- C. (1) 3-ONOP-041.7, Shutdown LOCA [Mode 3(Less than 1000 psig) or Mode 4]
(2) stop the RHR pumps
- D. (1) 3-ONOP-041.7, Shutdown LOCA [Mode 3(Less than 1000 psig) or Mode 4]
(2) NOT stop the RHR pumps

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PTN L-15-1 NRC EXAM

This information is controlled by PTN's 2015 LOIT (L-15-1) NRC EXAMINATION SECURITY AGREEMENT.

QUESTION 86

Given the following conditions:

- Unit 3 is at 95% power.
- 3A CCW Pump is OOS for maintenance.
- 3B CCW Pump tripped.
- 3C CCW Pump is running.
- The Unit Supervisor reviews DCS - RCP Detailed Data Summary (see attached).

Which ONE of the following identifies the earliest notification requirement, if any, for the given plant conditions after the crew has completed initial actions?

REFERENCE PROVIDED

- A. No notifications are required.
- B. 15 minute to the State and Counties.
- C. 4 hour to the Nuclear Regulatory Commission Operations Center (NRCOC).
- D. 8 hour to the Nuclear Regulatory Commission Operations Center (NRCOC).

This information is controlled by PTN's 2015 LOIT (L-15-1) NRC EXAMINATION SECURITY AGREEMENT.

PTN L-15-1 NRC EXAM

This information is controlled by PTN's 2015 LOIT (L-15-1) NRC EXAMINATION SECURITY AGREEMENT.

QUESTION 87

Given the following conditions:

- Unit 3 is at 100% power.

Subsequently:

- The Reactor fails to automatically trip on low SG levels following the loss of 3A SGFP.
- 3-EOP-FR-S.1, Response to Nuclear Power Generation/ATWS, is in progress.
- Annunciator B9/4, ROD CONTROL URGENT FAILURE, alarms.
- Local actions to trip the reactor are unsuccessful.
- 3B Boric Acid Transfer pump is running.
- Prior to manipulation, MOV-3-350, Emergency Boration Valve, position red and green lights are found NOT lit.
- CET temperature is 1250°F and rising.

Which ONE of the following identifies (1) the required action and (2) whether PARS are required for this event?

REFERENCE PROVIDED

- A. (1) Closing LCV-3-114B, Blender Flow to VCT.
(2) PARS required.
- B. (1) Opening 3-356, Manual Emergency Boration valve
(2) PARS NOT required.
- C. (1) Closing LCV-3-114B, Blender Flow to VCT.
(2) PARS NOT required.
- D. (1) Opening 3-356, Manual Emergency Boration valve.
(2) PARS required.

This information is controlled by PTN's 2015 LOIT (L-15-1) NRC EXAMINATION SECURITY AGREEMENT.

PTN L-15-1 NRC EXAM

This information is controlled by PTN's 2015 LOIT (L-15-1) NRC EXAMINATION SECURITY AGREEMENT.

QUESTION 88

Given the following conditions:

- Unit 4 is in MODE 3
- The switchyard lost power 18 minutes earlier and remains de-energized.
- 4A EDG is running and its output breaker red light is lit.
- 4B EDG is running and its output breaker green light is lit.
- 4B RCP has no light indication.
- The station blackout tie permissive blue light is off.

The Unit Supervisor enters 4-ONOP-004.3, Loss of 4B 4KV Bus, and has verified that control room evacuation is not required.

Which ONE of the following identifies the whether Attachment 1, 4B 4KV Bus Stripping, is required in accordance with 4-ONOP-004.3 and the correct Emergency Classification?

REFERENCE PROVIDED

- A.
 - Is NOT
 - Alert
- B.
 - Is
 - Alert
- C.
 - Is
 - Unusual Event
- D.
 - Is NOT
 - Unusual Event

This information is controlled by PTN's 2015 LOIT (L-15-1) NRC EXAMINATION SECURITY AGREEMENT.

PTN L-15-1 NRC EXAM

This information is controlled by PTN's 2015 LOIT (L-15-1) NRC EXAMINATION SECURITY AGREEMENT.

QUESTION 89

Given the following conditions:

- Unit 3 is at 100% power.
- I 5/4, TPCW HI TEMP/LO PRESS, alarms.
- POV-3-4882, ICW to TPCW Heat Exchanger, is found closed.
- The crew enters 3-ONOP-019, Intake Cooling Water Malfunction.

Subsequently:

- Action is taken to locally open POV-3-4882 with the handwheel.

Which ONE of the following identifies the following Technical Specification requirements?

When POV-3-4882 is locally opened, Unit 3 (1) a 72 hour action statement.

POV-3-4882 has an instrument air accumulator designed to maintain the valve position for (2) hours.

- A. (1) enters
(2) two
- B. (1) enters
(2) four
- C. (1) does NOT enter
(2) two
- D. (1) does NOT enter
(2) four

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PTN L-15-1 NRC EXAM

This information is controlled by PTN's 2015 LOIT (L-15-1) NRC EXAMINATION SECURITY AGREEMENT.

QUESTION 90

Given the following conditions:

- A safety injection has occurred due to lowering pressurizer level and pressure.
- 3-EOP-E-0, Reactor Trip or Safety Injection, is in progress
- Containment radiation parameters are normal.
- R-14, Plant Vent, and RI-3-1415B, Aux Bldg North N/S Corridor, are in alarm.

Which ONE of the following identifies the first transition out of 3-EOP-E-0 based on the given conditions and the action to be taken?

___(1)___ is entered and isolating ___(2)___ is the first action taken.

- A. (1) 3-EOP-ECA-1.1, Loss of Emergency Coolant Recirculation
(2) MOV-3-869, Safety Injection Hot Leg Isolation
- B. (1) 3-EOP-ECA-1.2, LOCA Outside Containment
(2) MOV-3-869, Safety Injection Hot Leg Isolation
- C. (1) 3-EOP-ECA-1.1, Loss of Emergency Coolant Recirculation
(2) MOV-3-750/751, RHR Suction Stop valves
- D. (1) 3-EOP-ECA-1.2, LOCA Outside Containment
(2) MOV-3-750/751, RHR Suction Stop valves

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PTN L-15-1 NRC EXAM

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QUESTION 91

Given the following conditions:

- The Control Room is evacuated due to smoke and a fire in a cable tray.
- Unit Supervisor is performing Attachment 13, Unit Supervisor from 0-ONOP-105.
- Unit 3 RO initiates an Emergency Boration.
- All CRDM Fans are running.
- The Unit 3 RCO starts Attachment 22, Unit 3 Cool Down from ASP.
- Containment temperature is 185°F.
- Core Exit Thermocouple (CET) temps are at 700°F.
- Reactor Coolant System (RCS) pressure is 2220 psig.

Which ONE of the following strategies does the Unit Supervisor next direct to mitigate the given conditions?

NOTE

- 0-ONOP-105, Control Room Evacuation
 - 3-EOP-FR-C.2, Response to Degraded Core Cooling
-
- A. Continue in 0-ONOP-105, and then direct dumping steam while maintaining 20% to 50% corrected S/G levels.
 - B. Continue in 0-ONOP-105, and then direct a RCS depressurization to allow ECCS Accumulator injection.
 - C. Immediately transition to 3-EOP-FR-C.2, and then direct dumping steam while maintaining 20% to 50% corrected S/G levels.
 - D. Immediately transition to 3-EOP-FR-C.2, and then direct a RCS depressurization to allow ECCS Accumulator injection.

This information is controlled by PTN's 2015 LOIT (L-15-1) NRC EXAMINATION SECURITY AGREEMENT.

PTN L-15-1 NRC EXAM

This information is controlled by PTN's 2015 LOIT (L-15-1) NRC EXAMINATION SECURITY AGREEMENT.

QUESTION 92

Given the following conditions:

- A Unit 4 Steam Generator Tube Rupture occurs.
- Due to equipment failures, the crew is performing actions contained in 4-EOP-ECA-3.2, SGTR With Loss Of Reactor Coolant - Saturated Recovery Desired.
- All CSF Status Trees are GREEN with the exception of the following:
 - Core Cooling - YELLOW path for 4-EOP-FR-C.3, Response To Saturated Core Cooling
 - Inventory - YELLOW path for 4-EOP-FR-I.2, Response To Low Pressurizer Level

Which ONE of the following describes the required implementation of procedures for this event, and the reason?

- A. 4-EOP-ECA-3.2 directs steps to be performed in accordance with 4-EOP-FR-I.2 to restore the Inventory.
- B. 4-EOP-ECA-3.2 directs steps to be performed in accordance with 4-EOP-FR-C.3 to restore the Core Cooling.
- C. Remain in 4-EOP-ECA-3.2. The actions contained in 4-EOP-FR-C.3 and 4-EOP-FR-I.2 conflict with 4-EOP-ECA-3.2 actions.
- D. Remain in 4-EOP-ECA-3.2. Implementation of YELLOW Path procedures is not allowed when using Emergency Contingency Action procedures.

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PTN L-15-1 NRC EXAM

This information is controlled by PTN's 2015 LOIT (L-15-1) NRC EXAMINATION SECURITY AGREEMENT.

QUESTION 93

Given the following conditions:

- During a 3-EOP-ES-0.3 cooldown, RVLMS indicates the Plenum is less than 40%.

NOTE

- 3-EOP-ES-0.3, Natural Circulation Cooldown With Steam Void In Vessel (With RVLMS)
- 3-EOP-FR-I.3, Response To Voids In The Reactor Vessel

Which ONE of the following is the required strategy and the basis?

_____ (1) _____ is used to re-pressurize the RCS. The basis for this action is _____ (2) _____.

- A. (1) 3-EOP-FR-I.3
(2) to prevent the void from expanding into the S/G U-tubes
- B. (1) 3-EOP-FR-I.3
(2) to raise CET subcooling to greater than 19°F to collapse the void
- C. (1) 3-EOP-ES-0.3
(2) to prevent the void from expanding into the S/G U-tubes
- D. (1) 3-EOP-ES-0.3
(2) to raise CET subcooling to greater than 19°F to collapse the void

This information is controlled by PTN's 2015 LOIT (L-15-1) NRC EXAMINATION SECURITY AGREEMENT.

PTN L-15-1 NRC EXAM

This information is controlled by PTN's 2015 LOIT (L-15-1) NRC EXAMINATION SECURITY AGREEMENT.

QUESTION 94

Given the following conditions:

- Unit 3 is in MODE 3 following a refueling outage.
- 3-OSP-202.1, Safety Injection/Residual Heat Removal Flowpath Verification, Attachment 2 is in progress.
- Prior to exceeding 380°F, the Shift Manager is verifying that the preceding steps of 3-GOP-503, Cold Shutdown to Hot Standby, are complete prior to signing the Shift Manager Verification Point.

Which ONE of the following identifies whether subsequent 3-GOP-503 procedure steps, past the Shift Manager Verification Point, may be performed?

- A. Steps may NOT be performed until 3-OSP-202.1, Attachment 2 is complete.
- B. Steps may NOT be performed without the Plant General Manager concurrence.
- C. Steps may be performed if they will not heat the RCS temperature above 380°F.
- D. Steps may be performed if they does not affect the SI/RHR Flowpath verification.

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PTN L-15-1 NRC EXAM

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QUESTION 95

Which ONE of the following completes the statements below?

The Emergency Coordinator (EC) is responsible to ensure the NRC is notified within
___(1)___ minutes of declaration of an emergency.

The Emergency Coordinator (EC) task of making Offsite Protection Action Recommendations
(PARS) ___(2)___ be delegated to the TSC Operations Manager.

- A. (1) 15
(2) can NOT
- B. (1) 15
(2) can
- C. (1) 60
(2) can NOT
- D. (1) 60
(2) can

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PTN L-15-1 NRC EXAM

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QUESTION 96

Given the following conditions:

- Unit 4 is in MODE 3, following a spurious trip.
- The A AFW Pump fails during the trip.
- The AFW system has NOT been re-aligned.

Which ONE of the following identifies if a MODE change is allowed or required, and why?

REFERENCE PROVIDED

- A. Entry into MODE 1 and 2 is allowed, with no restrictions.
- B. Entry into MODE 1 and 2 is allowed. Restore the A AFW pump to Operable within 30 days, or place the unit in MODE 3 within 6 hours and in MODE 4 within the following 6 hours.
- C. Entry into MODE 2 is not allowed. Restore the A AFW pump to Operable within 30 days, or place the unit in MODE 3 within 6 hours and in MODE 4 within the following 6 hours.
- D. Entry into MODE 2 is not allowed. Restore the A AFW pump to Operable within 72 hours, or place the unit in MODE 3 within the next 6 hours and in MODE 4 within the following 6 hours.

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PTN L-15-1 NRC EXAM

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QUESTION 97

Given the following conditions:

- Unit 4 is in a refueling outage.
- A Control Room annunciator is to be defeated for a clearance order on equipment required to be operable for the current plant conditions.

In accordance with 0-ADM-219, Annunciator Response Procedure Usage, which ONE of the choices below completes the following:

0-OSP-200.5, Attachment 1, Defeated/Out-Of-Service Annunciator Checklist, _____ retained in the Equipment Out of Service Index.

The applicable portions of 0-OSP-200.5, Miscellaneous Tests, and Operating Evolutions, for Defeated/Out-Of-Service Annunciators must be completed _____.

- A. is NOT; within ONE hour
- B. is NOT; prior to defeating the annunciator
- C. is; within ONE hour
- D. is; prior to defeating the annunciator

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PTN L-15-1 NRC EXAM

This information is controlled by PTN's 2015 LOIT (L-15-1) NRC EXAMINATION SECURITY AGREEMENT.

QUESTION 98

Given the following conditions:

- An Emergency Response Team is being briefed on a mission to rescue a worker who has suffered a sprained ankle.
- Radiation Protection estimates each team member will receive a total dose (TEDE) of 11 rem and a thyroid dose of 6 rem while performing this rescue.

In accordance with 0-EPIP-20111, Re-entry, which ONE of the following correctly describes (1) the estimated dose as compared to the Emergency Worker Exposure TEDE limit and (2) the guidance for issuing Potassium Iodide (KI) to the team members?

- A. (1) is within the Emergency Worker Exposure TEDE limit;
(2) KI should be issued to prevent iodine absorption
- B. (1) is within the Emergency Worker Exposure TEDE limit;
(2) KI should NOT be issued
- C. (1) exceeds the Emergency Worker Exposure TEDE limit;
(2) KI should NOT be issued
- D. (1) exceeds the Emergency Worker Exposure TEDE limit;
(2) KI should be issued to prevent iodine absorption

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PTN L-15-1 NRC EXAM

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QUESTION 99

Given the following plant conditions:

- Annunciator C4/4 PZR HI PRESS TRIP alarms and is confirmed valid
- Unit 3 reactor is at 100%
- CETs are 1225 °F and rising.

Which ONE of the following completes the statements below?

In accordance with 3-FRP-S.1, Response to Nuclear Power Generations/ATWS, the crew unsuccessfully tried tripping the reactor by opening the (1).

The Unit Supervisor is required to transition to (2).

- A. (1) 3A and 3B motor generator input and output breakers
(2) SACRG-1, Severe Accident Control Room Guideline Initial Response
- B. (1) 3A and 3B motor generator input and output breakers
(2) 3-EOP-FR-C.1, Response to Inadequate Core Cooling
- C. (1) feeder breakers to 3A and 3D 480V Load Centers
(2) SACRG-1, Severe Accident Control Room Guideline Initial Response
- D. (1) feeder breakers to 3A and 3D 480V Load Centers
(2) 3-EOP-FR-C.1, Response to Inadequate Core Cooling

This information is controlled by PTN's 2015 LOIT (L-15-1) NRC EXAMINATION SECURITY AGREEMENT.

PTN L-15-1 NRC EXAM

This information is controlled by PTN's 2015 LOIT (L-15-1) NRC EXAMINATION SECURITY AGREEMENT.

QUESTION 100

The following plant conditions exist:

- A SITE AREA EMERGENCY was declared 37 minutes ago.
- The emergency response facilities are NOT activated.
- The communicator has made the notification to the State and NRC.
- Conditions are stabilized and the event no longer meets the emergency action level criteria.

Who is responsible for de-escalation of the classification?

- A. Recovery Manager
- B. Emergency Coordinator
- C. Nuclear Division Duty Officer
- D. Emergency Control Officer

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SUBJECTIVE SCORE INSTRUCTOR USE ONLY



FPL

**TURKEY POINT PLANT
NUCLEAR TRAINING
DEPARTMENT**

DIRECTIONS

Darken Blocks Completely
Make No Stray Marks

USE HB # PENCIL ONLY

	010	020	030	040	050
01	A	B	C	D	E
02	A	B	C	D	E
03	A	B	C	D	E
04	A	B	C	D	E

KEY

NAME: PTN L-15-1 NRC Exam - SRO

SOC. SEC. #: N/A

DEPT: Training DEPT. SUPV. Cashwell

DATE: 02/18/15

EXAM TITLE: PTN L-15-1 NRC Exam - SRO

EXAM NO.: Exam Key

TOTAL POINTS: 100

I hereby certify that I will take the exam and abide by the rules of conduct provided by the proctor.

N/A
Student's Signature

N/A
Date

Student signature below certifies that the graded exam has been reviewed:

N/A
Student's Signature

N/A
Date

Grader's Name:

Score: N/A

N/A
Print

Signature

Date

QC Grader's Name:

Score: N/A

N/A
Print

Signature

Date

RAR IE#: N/A

*Q42: Both 'A' and 'C' accepted as correct
iaw post-exam comment.

KEY