TRANSMISSION VERIFICATION REPORT

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2443 WARRENVILLE ROAD, SUITE 210 LISLE, ILLINOIS 60532-4352

TELEFAX TRANSMITTAL

DATE February 19, 2015

NUMBER OF PAGES 11

SEND TO Lisa Maddox, Radiology Director

LOCATION St. Catherine Regional Hospital, NRC License 13-23665-01

FAX NUMBER (812) 256-7496

VERIFY BY CALLING CK. Received Lisa 2/19/15

FROM: Bill Reichhold (Sender)

TELEPHONE NUMBER (630) 829-9839

FAX NUMBER (630) 515-1078

If you do not receive the complete fax transmittal, please contact the sender as soon as possible at the telephone number provided above.

MESSAGE See accompanying documents.



UNITED STATES NUCLEAR REGULATORY COMMISSION REGION III

2443 WARRENVILLE ROAD, SUITE 210 LISLE, ILLINOIS 60532-4352

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NOTICE

This message is intended only for the use of the individual or entity to which it is addressed and may contain information that is privileged, confidential, or exempt from disclosure under applicable law. If the reader of this message is not the intended recipient or the employee responsible for delivering the message to the intended recipient, you are hereby notified that any dissemination, distribution or copying of this communication is strictly prohibited. If you received this communication in error, please notify the sender immediately by telephone and return the original to the above address, by U.S. Mail. Thank You.

MUSOSOA624

Thank you for notifying us that St. Catherine Regional Hospital, NRC License 13-23665-01 has "closed". We need to ensure that public health and safety is adequately protected and that radioactive material is properly controlled.

Please provide the following additional information:

- 1. If you need to close a building, you must first properly secure, dispose of, or transfer all radioactive (licensed) material that was in the building.
- 2. If you need to lay-off the RSO, you must first properly secure, dispose of or transfer all radioactive (licensed) material, and you must inform NRC of this situation. We need an individual identified on the license that has the training and experience in radiation safety and will be responsible for the security, transfer and disposal of any radionuclides you possess.
- 3. Please identify what radionuclides are possessed by your hospital at this time. From our previous correspondence dated November 19, 2012, it appears that you had the following sealed sources:

E-Vial sealed source – cobalt-57, Eckert & Ziegler, Source 1266-58-14 (located in hot lab behind lead shielding).

E-Vial sealed source – cesium-137, Source 235

E-Vial sealed source – cobalt-57, Eckert & Ziegler, Source 1000419.

Flood Source – cobalt-57, SRV-057-5M 1510-100-10

and

Well counter calibration source cesium -137, Model CAL 2602, Lot 52629-52678 (located in hot lab behind lead shielding).

- 4. Please indicate how you plan to dispose of the radionuclides, radioactive waste, etc. For example, radioactive active waste held for decay and disposed of in accordance with 10 CFR 35.92, "Decay in Storage". OR All radioactive waste and sealed sources were transferred to an authorized radioactive waste broker for disposal.
- 5. For each of the radionuclides identified above, please specify that the radionuclide(s) are in secured storage at this time and specify the storage location. For example all radioactive sealed sources and radioactive waste are locked in the "hot" lab. The keys to the lock are controlled by the Radiation Safety Officer.
- 6. Please indicate if there have been any changes in the Radiation Safety Officer or authorized user(s) for the radionuclides.
- 7. Please specify who will be the contact person and at what frequency will they update the NRC concerning any changes in the status of the radionuclides and or the hospital "closure" proceedings. Please make sure you immediately contact the NRC of any change in status of use, security and/or storage of the radionuclides and any change in the individual(s) responsible for the radionuclides.
- 8. If you are planning to terminate your NRC license, please submit the following information:
 - A. Please submit a completed NRC Form 314, "Certificate of Disposition of Materials". See accompanying copy of NRC Form 314.
 - B. Please be advised that we cannot authorize you to release your nuclear medicine department located at 2200 Market Street, Charlestown, Indiana, for unrestricted use (even by other members of your staff) until we have received and reviewed a copy of the results of your close-out survey. The survey should consist of exposure rate measurements to show that all sources of radioactive material have been removed, and contamination checks of areas where radioactive materials were used or

stored. Average radiation levels associated with surface contamination and removable contaminations should not exceed those specified in the enclosed decontamination guide. Please submit the following information with your close-out survey:

- a. A history of all radionuclides used at your old facility.
- b. A current copy of the leak test results for the sealed sources used at your old facility. Also a history of leaking sealed sources (if any).
- c. A diagram of your old facility with survey and wipe test results keyed to specific locations. Please record your survey results using the appropriate units as described in 10 CFR 30.36 (j) (2) (I) (copy enclosed).
- d. The name of the person performing the survey.
- e. The date the survey was performed.
- f. The instrument(s) used for exposure rate measurements and for analysis of the wipes.
- g. Background readings.
- h. The date that the survey instrument was last calibrated.
- i. Confirm that all radioactive waste has been decayed to background radiation or has been transferred to a radioactive waste broker for disposal. Also, please specify the final disposition of the sealed sources. Please submit a copy of the acknowledgement that the radioactive materials were received by the company where the radioactive materials were shipped for transfer/disposal.

Please send a facsimile of your response to the above to William Reichhold (630) 515-1078 as soon as possible. Please call me at (630) 829-9839 if you have any questions.

From the desk of: But feithold **Bill Reichhold**

In accordance with 10 CFR 2.390 of the NRC's "Rules of Practice," a copy of this facsimile and the attached documents will be available electronically for public inspection in the NRC Public Document Room or from the Publicly Available Records (PARS) component of NRC's document system (ADAMS). The NRC's document system is accessible from the NRC Web site at http://www.nrc.gov/reading-rm/adams.html (the Public Electronic Reading Room).

CERTIFICATE OF DISPOSITION OF MATERIALS

PLEASE READ THESE INSTRUCTIONS BEFORE COMPLETING NRC FORM 314.

Subpart E of 10 CFR Part 20 establishes the radiological criteria for license terminations/decommissioning of facilities licensed under 10 CFR Parts 30, 40, 50, 60, 61, 70, and 72, as well as other facilities subject to the Commission's jurisdiction under the Atomic Energy Act of 1954, as amended, and the Energy Reorganization Act of 1974, as amended.

INSTRUCTIONS

Section B, Item 2.

Licensees should describe the specific radioactive material transfer actions. If radioactive wastes were generated in terminating this license, the licensee should describe the disposal actions taken, including the disposition of low-level radioactive waste, mixed waste, greater-than-Class-C waste, and sealed sources.

Section B, Item 2.a.

The information provided concerning the transfer of radioactive material to another licensee should specify the date of the transfer, the name of the licensee recipient, an individual contact name and telephone number for the licensee recipient, and the recipient's NRC or Agreement State license number.

Section B, Item 2.b.

For disposal of radioactive materials, licensees should describe the specific disposal method or procedure (e.g., decay-in-storage). For those cases when radioactive materials are disposed of by a licensed disposal site or by a waste contractor, the licensee should specify the name, address, and telephone number of the licensee disposal site operator or waste contractor.

Section B, Item 2.c.

"Residual radioactivity," as defined in 10 CFR 20.1003, means radioactivity in 'areas' (structures, materials, soils, etc.) remaining as a result of activities (licensed and unlicensed) under the licensee's control from sources used by the licensee, excluding background radiation. ALARA is defined in 10 CFR 20.1003.

FILE CERTIFICATES AS FOLLOWS:

IF YOU ARE LOCATED IN:

ALABAMA, CONNECTICUT, DELAWARE, DISTRICT OF COLUMBIA, FLORIDA, GEORGIA, KENTUCKY, MAINE, MARYLAND, MASSACHUSETTS, NEW HAMPSHIRE, NEW JERSEY, NEW YORK, NORTH CAROLINA, PENNSYLVANIA, PUERTO RICO, RHODE ISLAND, SOUTH CAROLINA, TENNESSEE, VERMONT, VIRGINIA, VIRGIN ISLANDS, OR WEST VIRGINIA, SEND CERTIFICATES TO:

LICENSING ASSISTANT SECTION NUCLEAR MATERIALS SAFETY BRANCH U.S. NUCLEAR REGULATORY COMMISSION, REGION I 2100 RENAISSANCE BOULEVARD, SUITE 100 KING OF PRUSSIA, PA 19406-2713

ILLINOIS, INDIANA, IOWA, MICHIGAN, MINNESOTA, MISSOURI, OHIO, OR WISCONSIN, SEND CERTIFICATES TO:

MATERIALS LICENSING SECTION U.S. NUCLEAR REGULATORY COMMISSION, REGION III 2443 WARRENVILLE ROAD, SUITE 210 LISLE, IL 60532-4352

IF YOU ARE LOCATED IN:

ALASKA, ARIZONA, ARKANSAS, CALIFORNIA, COLORADO, HAWAII, IDAHO, KANSAS, LOUISIANA, MISSISSIPPI, MONTANA, NEBRASKA, NEVADA, NEW MEXICO, NORTH DAKOTA, OKLAHOMA, OREGON, PACIFIC TRUST TERRITORIES, SOUTH DAKOTA, TEXAS, UTAH, WASHINGTON, OR WYOMING, SEND CERTIFICATES TO:

MATERIAL RADIATION PROTECTION SECTION U. S. NUCLEAR REGULATORY COMMISSION, REGION IV 1600 E. LAMAR BOULEVARD ARLINGTON, TX 76011-4511

NRC FORM 314 U.S. NUCLEAR REGULATORY COMMISSIO	N APPROVED BY OMB: NO. 3150-0028	EXPIRES: 02/28/2017		
(02-2014) 10 CFR 30.36(j)(1);	Estimated burden per response to comply with this mand used by NRC as part of the basis for its determination			
	comments regarding burden estimate to the FOIA, Privac	y, and Information Collections Branch (T-5 F53), U.S.		
CERTIFICATE OF DISPOSITION	Nuclear Regulalory Commission, Washington, DC 2 Resource@nrc.gov, and to the Desk Officer, Office of			
OF MATERIALS	(3150-0028), Office of Management and Budgel, Wash Information collection does not display a currently valid	ington, DC 20503. If a means used to impose an		
	sponsor, and a person is not required to respond to, the in			
LICENSEE NAME AND ADDRESS	LICENSE NUMBER	DOCKET NUMBER		
:				
	LICENSE EXPIRATION DATE			
	LIGENSE EXFIRATION DATE			
A. LICENSE STATUS (Check th	e appropriate box)			
This license has expired. This license has not yet expired; pleas	se terminate it.			
B. DISPOSAL OF RADIOAC				
(Check the appropriate boxes and complete as necessary. If a		attachments)		
The licensee, or any individual executing this certificate on behalf of the licen				
1. No radioactive materials have ever been procured or possessed b				
 All activities authorized by this license have ceased, and all radioa under this license number sited above have been dispersed of in the 	ctive materials procured and/or po	ssessed by the licensee		
under this license number cited above have been disposed of in th	le following manner.			
a. Transfer of radioactive materials to the licensee listed below:				
		- 1		
b. Disposal of radioactive materials:				
1. Directly by the licensee:				
2. By licensed disposal site:				
		5		
3. By waste contractor:				
	ining seciel under activity in within	the limits of 10 CED		
c. All radioactive materials have been removed such that any remaining residual radioactivity is within the limits of 10 CFR Part 20, Subpart E, and is ALARA.				
C. SURVEYS PERFORMED				
1. A radiation survey was conducted by the licensee. The survey confirms:				
a. the absence of licensed radioactive materials				
b. that any remaining residual radioactivity is within the limits of 10 CFR 20, Subpart E, and is ALARA.				
2. A copy of the radiation survey results:				
a. is attached; or b. is not attached (Provide explanation); or c. was forwarded to NRC on:				
Date Date				
a. The results of the latest leak test are attached; and/or	b. No leaking sources have eve	r boon identified		
	_ b. No leaking sources have eve			
The person to be contacted regarding the information provided on this form:				
NAME	TELEPHONE (Include Area Code) E-MAIL	ADDRESS		
Mail all future correspondence recording this license to:		1		
Mail all future correspondence regarding this license to:				
C. CERTIFYING OF	FICIAL			
I CERTIFY UNDER PENALTY OF PERJURY THAT THE FOREGOING IS TRUE AND CORRECT				
PRINTED NAME AND TITLE SIGNATURE		DATE		
WARNING: FALSE STATEMENTS IN THIS CERTIFICATE MAY BE SUBJECT TO CIVIL AND/O	R CRIMINAL PENALTIES. NRC REGULA	TIONS REQUIRE THAT		
SUBMISSIONS TO THE NRC BE COMPLETE AND ACCURATE IN ALL MATERIAL RESPECT.	18 U.S.C. SECTION 1001 MAKES IT A CRI	MINAL OFFENSE TO MAKE A		
WILLFULLY FALSE STATEMENT OR REPRESENTATION TO ANY DEPARTMENT OR AGENC	T OF THE UNITED STATES AS TO ANY M	ATTER WITHIN ITS JURISDICTION.		
NRC FORM 314 (02-2014)				

analysis disposal capacity is available to allow completion of decommissioning within the allotted 24month period;

(\Im) Whether a significant volume reduction in wastes requiring disposal will be achieved by allowing short-lived

(4) Whether a significant reduction in radiation exposure to workers can be achieved by allowing short-lived radionuclides

(5) Other site-specific factors which the Commission may consider appropriate on a case-by-case basis, such as the regulatory requirements of other government agencies, lawsuits, ground-water treatment activities, monitored natural ground-water restoration, actions that could result in more environmental harm than deferred cleanup, and other factors

(j) As the final step in decommissioning, the licensee shall--

(1) Certify the disposition of all licensed material, including accumulated wastes, by submitting a completed NRC Form 314

(2) Conduct a radiation survey of the premises where the licensed activities were carried out and submit a report of the results of this survey, unless the licensee demonstrates in some other manner that the premises are suitable for release in accordance with the criteria for decommissioning in 10 CFR part 20, subpart E. The licensee shall, as appropriate---

(i) Report levels of gamma radiation in units of millisieverts (microroentgen) per hour at one meter from surfaces, and report levels of radioactivity, including alpha and beta, in units of megabecquerels (disintegrations per minute or microcuries) per 100 square centimeters--removable and fixed--for surfaces, megabecquerels (microcuries) per milliliter for water, and becquerels (picocuries) per gram for solids such as soils or concrete; and

(ii) Specify the survey instrument(s) used and certify that each instrument is properly calibrated and tested.

(k) Specific licenses, including expired licenses, will be terminated by written notice to the licensee when the Commission

http://www.prc.gov/reading-rm/doc-collections/cfr/part030/full-text.html

GUIDELINES FOR DECONTAMINATION OF FACILITIES AND EQUIPMENT PRIOR TO RELEASE FOR UNRESTRICTED USE

OR TERMINATION OF LICENSES FOR BYPRODUCT, SOURCE,

OR SPECIAL, NUCLEAR MATERIAL

U.S. Nuclear Regulatory Commission Division of Fuel Cycle, Medical, Academic, and Commercial Use Safety Washington, DC 20555

April 1993

2 Juit ar stalling

The instructions in this guide, in conjunction with Table 1, specify the radionuclides and radiation exposure rate limits which should be used in decontamination and survey of surfaces or premises and equipment prior to abandonment or release for unrestricted use. The limits in Table 1 do not apply to premises, equipment, or scrap containing induced radioactivity for which the radiological considerations pertinent to their use may be different. The release of such facilities or items from regulatory control is considered on a case-by-case basis.

- 1. The licensee shall make a reasonable effort to eliminate residual contamination.
- 2. Radioactivity on equipment or surfaces shall not be covered by paint, plating, or other covering material unless contamination levels, as determined by a survey and documented, are below the limits specified in Table 1 prior to the application of the covering. A reasonable effort must be made to minimize the contamination prior to use of any covering.
- 3. The radioactivity on the interior surfaces of pipes, drain lines, or ductwork shall be determined by making measurements at all traps, and other appropriate access points, provided that contamination at these locations is likely to be representative of contamination on the interior of the pipes, drain lines, or ductwork. Surfaces of premises, equipment, or scrap which are likely to be contaminated but are of such size, construction, or location as to make the surface inaccessible for purposes of measurement shall be presumed to be contaminated in excess of the limits.
- 4. Upon request, the Commission may authorize a licensee to relinquish possession or control of premises, equipment, or scrap having surfaces contaminated with materials in excess of the limits specified. This may include, but would not be limited to, special circumstances such as razing of buildings, transfer of premises to another organization continuing work with radioactive materials, or conversion of facilities to a long-term storage or standby status. Such requests must:
 - a. Provide detailed, specific information describing the premises, equipment or scrap, radioactive contaminants, and the nature, extent and degree of residual surface contamination.
 - b. Provide a detailed health and safety analysis which reflects that the residual amounts of materials on surface areas, together with other considerations such as prospective use of the premises, equipment, or scrap, are unlikely to result in an unreasonable risk to the health and safety of the public.

1.4 milidite

- 5. Prior to release of premises for unrestricted use, the licensee shall make a comprehensive radiation survey which establishes that contamination is within the limits specified in Table 1. A copy of the survey report shall be filed with the Division of Fuel Cycle Safety and Safeguards, U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, and also the Administrator of the NRC Regional Office having jurisdiction. The reports should be filed at least 30 days prior to the planned date of abandonment. The survey report shall:
 - a. Identify the premises.
 - b. Show that reasonable effort has been made to eliminate residual contamination.
 - c. Describe the scope of the survey and general procedures followed.
 - d. State the findings of the survey in units specified in the instructions.

Following review of the report, the NRC will consider visiting the facilities to confirm the survey.

TABLE 1

NUCLES SOLUTION LEVELS			
NUCLIDES ^a	AVERAGE ^{b o f}	MAXIMUM ^{bd f}	REMOVABLE
U-nat, U-235, U-238, and associated decay products	5,000 dpm $\alpha/100 \text{ cm}^2$	15,000 dpm α/100 cm ²	$1,000 \text{ dpm } \alpha/100 \text{ cm}^2$
Transuranics, Ra-226, Ra-228, Th-230, Th-228, Pa-231, Ac-227, I-125, I-129	100 dpm/100 cm ²	300 dpm/100 cm ²	20 dpm/100 cm ²
Th-nat, Th-232, Sr-90, Ra-223, Ra-224, U-232, I-126, I-131, I-133	1,000 dpm/100 cm ²	3,000 dpm/100 cm ²	200 dpm/100 cm ²
Beta-gamma emitters (nuclides with decay modes other than alpha emission or spontaneous fission) except Sr-90 and others noted above.	5,000 dpm $\beta\gamma/100 \text{ cm}^2$	15,000 dpm βγ/100 cm²	1,000 dpm $\beta\gamma/100 \text{ cm}^2$

ACCEPTABLE SURFACE CONTAMINATION LEVELS

Where surface contamination by both alpha- and beta-gamma-emitting nuclides exists, the limits established for alpha- and beta-gamma-emitting 2

As used in this table, dpm (disintegrations per minute) means the rate of emission by radioactive material as determined by correcting the counts per Ъ minute observed by an appropriate detector for background, efficiency, and geometric factors associated with the instrumentation.

Measurements of average contaminant should not be averaged over more than 1 square meter. For objects of less surface area, the average should be

The maximum contamination level applies to an area of not more than 100 cm².

The amount of removable radioactive material per 100 cm² of surface area should be determined by wiping that area with dry filter or soft absorbent paper, applying moderate pressure, and assessing the amount of radioactive material on the wipe with an appropriate instrument of known efficiency. When removable contamination on objects of less surface area is determined, the pertinent levels should be reduced proportionally and the entire

The average and maximum radiation levels associated with surface contamination resulting from beta-gamma emitters should not exceed 0.2 mrad/hr at 1 cm and 1.0 mrad/hr at 1 cm, respectively, measured through not more than 7 milligrams per square centimeter of total absorber.

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