

LaSalle Sation

2601 North 21st Road Marseilles, IL 61341

815 415 2000 www.exeloncorp.com

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U. S. Nuclear Regulatory Commission ATTN: Document Control Desk Washington, DC 20555-0001

> LaSalle County Station, Unit 2 Facility Operating License No. NPF-18 NRC Docket No. 50-374

Subject: Unit 2 Cycle 16 Core Operating Limits Report (COLR)

In accordance with LaSalle County Station Technical Specifications (TS) 5.6.5.d, "Core Operating Limits Report (COLR)," attached is a copy of the COLR for Unit 2. This report was revised for LSCS Unit 2, Cycle 16.

Exelon Generation Company, LLC makes no new or revised regulatory commitments in this letter.

Should you have any questions concerning this submittal, please contact Mr. Guy V. Ford, Jr., Regulatory Assurance Manager, at (815) 415-2800.

Respectfully,

Peter J. Karaba Site Vice President LaSalle County Station

Attachment: Core Operating Limits Report for LaSalle Unit 2 Cycle 16, Revision 0

cc: Regional Administrator - NRC Region III NRC Senior Resident Inspector - LaSalle County Station

# **Core Operating Limits Report for**

# LaSalle Unit 2

Cycle 16 Revision 0

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### 1. References

- 1. Exelon Generation Company, LLC Docket No. 50-374 LaSalle County Station, Unit 2, Facility Operating License No. NPF-18.
- 2. NRC Letter from D. M. Crutchfield to All Power Reactor Licensees and Applicants, Generic Letter 88-16; Concerning the Removal of Cycle-Specific Parameter Limits from Tech Specs, October 3, 1988.
- 3. Nuclear Fuels Letter NFM:MW:01-0106, from A. Giancatarino to J. Nugent, "LaSalle Unit 1 and Unit 2 Rod Block Monitor COLR Setpoint Change," April 3, 2001.
- 4. GE Nuclear Energy Report NEDC-32694P-A, Revision 0, "Power Distribution Uncertainties for Safety Limit MCPR Evaluations," August 1999.
- 5. GE Nuclear Energy Document GE-NE-A1300384-07-01, Revision 1, "LaSalle County Station Power Uprate Project Task 201: Reactor Power/Flow Map", September 1999.
- 6. GE Hitachi Nuclear Energy Report, GE-NE-0000-0099-8344-R1, Revision 1, "Exelon Nuclear LaSalle Units 1 and 2 Thermal Power Optimization Task T0201: Operating Power/Flow Map", November 2009.
- 7. GNF Report GNF-000N9256-SRLR-R0, Revision 0, "Supplemental Reload Licensing Report for LaSalle Unit 2 Reload 15 Cycle 16," January 2015.
- GNF Letter from B. R. Moore to Document Control Desk, Subject: "GNF2 Advantage Generic Compliance with NEDE-24011-P-A (GESTAR II), NEDC-33270P, Revision 5, May 2013," MFN 13-029, May 24, 2013 (ADAMS Accession No. ML13148A318)
- 9. AREVA Report ANP-2914(P), Revision 1, "Mechanical Design Report for LaSalle Units 1 and 2 MUR ATRIUM-10 Fuel Assemblies," AREVA NP Inc., June 2010.
- 10. Exelon Transmittal ES1400016, Revision 0, "LaSalle 2 Cycle 16 Final Resolved OPL-3 Parameters," September 11, 2014.
- 11. GNF DRF A12-00038-3, Vol. 4, "Scram Times Verses Notch Position," G. A. Watford, May 22, 1992.
- 12. GEH Nuclear Energy DRF Section 0000-0151-0765 Rev. 0, "Application of SLO MCPR", February 12, 2013.
- 13. GNF Report GNF-000N9257-FBIR-R0, Revision 0, "Fuel Bundle Information Report for LaSalle Unit 2 Reload 15 Cycle 16," January 2015.

# 2. Terms and Definitions

ARTS	Average Power Range Monitor, Rod Block Monitor and Technical Specification
AITO	Improvement Program
ATRM10	AREVA ATRIUM-10 fuel type
ATRM10XM	AREVA ATRIUM-10XM fuel type
BOC	Beginning of cycle
BWR	Boiling water reactor
COLR	Core operating limits report
CRD	Control rod drive mechanism
DLO	Dual loop operation
ELLLA	Extended load line limit analysis
EOC	End of cycle
EOOS	Equipment out of service
EOR16 FFWTR	End of rated operation for Cycle 16 Final feedwater temperature reduction
FWHOOS	Feedwater heater out of service
GNF	Global Nuclear Fuels - Americas
ICF	Increased core flow
К <sub>Р</sub>	Power-dependent MCPR Multiplier
L2C16	LaSalle Unit 2 Cycle 16
LHGR	Linear heat generation rate
LHGRFAC <sub>F</sub>	Flow-dependent LHGR multiplier
	Power-dependent LHGR multiplier
	Local power range monitor
MAPLHGR MCPR	Maximum average planar linear heat generation rate Minimum critical power ratio
	Flow-dependent MCPR
MELLLA	Maximum extended load line limit analysis
MOC	Middle of Cycle Point for Licensing Purposes
MSIVOOS	Main steam isolation valve out of service
OLMCPR	Operating limit minimum critical power ratio
OOS	Out of service
OPRM	Oscillation power range monitor
PBDA	Period based detection algorithm
PLUOOS PROOS	Power load unbalance out of service
RPTOOS	Pressure regulator out of service Recirculation pump trip out of service
RWE	Rod withdrawal error
SLO	Single loop operation
SRVOOS	Safety-relief valve out of service
TBV	Turbine bypass valve
TBVOOS	Turbine bypass valve out of service
TCV	Turbine control valve
TCVSC	Turbine control valve slow closure
TIP	Traversing in-core probe
TIPOOS	Traversing in-core probe out of service
TSV 3DM	Turbine stop valve 3D-MONICORE
SDIM	JU-WUNUUKE

# 3. General Information

Power and flow dependent limits are listed for various power and flow levels. Linear interpolation is to be used to find intermediate values.

Rated core flow is 108.5 Mlbm/hr. Operation up to 105% rated flow is licensed for this cycle. Licensed rated thermal power is 3546 MWth.

For thermal limit monitoring above 100% rated power or 100% rated core flow, the 100% rated power and the 100% core flow values, respectively, can be used unless otherwise indicated in the applicable table.

Table 3-1 defines the three exposure ranges used in the COLR. The end of rated (EOR) exposure is defined as the cycle exposure corresponding to all rods out, 100% power/100% flow, and normal feedwater temperature. The term (EOR – 2288 MWd/ST) means the EOR exposure minus 2288 MWd/ST of exposure. The value of the EOR exposure is based on actual plant operation and is thus determined from projections to this condition made near, but before, the time when the EOR16 – 2288 MWd/ST exposure will be reached. For cycle exposure dependent limits at the exact MOC exposure, the more limiting of the BOC to MOC and the MOC to EOC limits should be used. This can be achieved by applying the MOC to EOC limits to the MOC point as all cycle exposure dependent limits in the MOC to EOC limit sets are the same as, or more limiting than, those in the BOC to MOC limit sets.

Nomenclature	Cycle Exposure Range
BOC to MOC	BOC16 to (EOR16 – 2288 MWd/ST)
MOC to EOC	(EOR16 – 2288 MWd/ST) to EOC16
BOC to EOC	BOC16 to EOC16

#### Table 3-1 Cycle Exposure Range Definitions (Reference 7)

# 4. Average Planar Linear Heat Generation Rate

### Technical Specification Sections 3.2.1 and 3.4.1

The MAPLHGR values for the most limiting lattice of each fuel type as a function of average planar exposure are given in Tables 4-1 and 4-2. During single loop operation, these limits are multiplied by the fuel-dependent SLO multiplier listed in Table 4-3. The MAPLHGR values in Tables 4-1 and 4-2 along with the MAPLHGR SLO multipliers in Table 4-3 provide coverage for all modes of operation.

Avg. Planar Exposure (GWd/ST)	MAPLHGR (kW/FT)
0.00	13.78
17.15	13.78
60.78	6.87
63.50	5.50

### Table 4-1 MAPLHGR for GNF2 and GNF3 Fuel (Reference 7)

### Table 4-2 MAPLHGR for ATRIUM-10 and ATRIUM-10XM Fuel (Reference 7)

Avg. Planar Exposure (GWd/ST)	MAPLHGR (kW/FT)
0	12.81
21.41	12.81
55.42	9.10
63.86	7.30

### Table 4-3 MAPLHGR SLO Multiplier for GNF2, GNF3, ATRIUM-10, and ATRIUM-10XM Fuel, BOC to EOC

(Reference 7)

Fuel Type	SLO MAPLHGR Multiplier
GNF2	0.78
GNF3	0.78
ATRIUM-10	0.78
ATRIUM-10XM	0.78

# 5. Operating Limit Minimum Critical Power Ratio

Technical Specification Sections 3.2.2, 3.3.4.1, 3.4.1, and 3.7.7

# 5.1. Manual Flow Control MCPR Limits

The steady-state OLMCPRs given in Table 5-2 are the maximum values obtained from analysis of the pressurization events, non-pressurization events, and the Option III stability evaluation. MCPR values are determined by the cycle-specific fuel reload analyses in Reference 7. Table 5-2 is used in conjunction with the ARTS-based power (Kp) and flow (MCPR<sub>F</sub>) dependencies presented in Tables 5-3, 5-4, and 5-5 below. The OLMCPR is determined for a given power and flow condition by evaluating the power and flow dependent MCPR values and selecting the greater of the two.

### 5.1.1. Power-Dependent MCPR

The power-dependent MCPR multiplier,  $K_P$ , is determined from Table 5-3, and is dependent only on the power level and the Application Group (EOOS). The product of the steady state OLMCPR and the proper  $K_P$  provides the power-dependent OLMCPR.

### 5.1.2. Flow-Dependent MCPR

Tables 5-4 through 5-5 give the MCPR<sub>F</sub> limit as a function of the core flow, based on the applicable plant conditions. The MCPR<sub>F</sub> limit determined from these tables is the flow-dependent OLMCPR.

### 5.2. Scram Time

Option A and Option B MCPR analyses and results are dependent upon core average control rod blade scram speed insertion times.

The Option A scram time is the Improved Technical Specification scram speed based insertion time. The core average scram speed insertion time for 20% insertion must be less than or equal to the Technical Specification scram speed insertion time to utilize the Option A MCPR limits. Reload analyses performed by GNF for Cycle 16 Option A MCPR limits utilized a 20% core average insertion time of 0.900 seconds (Reference 10).

To utilize the MCPR limits for the Option B scram speed insertion times, the core average scram speed insertion time for 20% insertion must be less than or equal to 0.694 seconds (Reference 10) (0.672 seconds at notch position 39, Reference 11). See Table 5-1 for a summary of scram time requirements related to the use of Option A and Option B MCPR limits.

If the core average scram insertion time does not meet the Option B criteria, but is within the Option A criteria, the appropriate steady state MCPR value may be determined from a linear interpolation between the Option A and B limits with standard mathematical rounding to two decimal places. When performing the linear interpolation to determine MCPR limits, ensure that the time used for Option A is 0.900 seconds (0.875 seconds to notch position 39, Reference 11). Note that making interpolations using the Table 5-2 data is conservative because the stability based OLMCPR sets the limit in many conditions. The Option A to Option B linear interpolation need not include the stability OLMCPR penalty on the endpoints when the calculation is made. However, the result of the linear interpolation is required to be 1.51 or greater for the steady state OLMCPR due to the OPRM PBDA setpoint (see Section 9 of the COLR and Reference 7).

Notch	Scram Time Required for Option A	Scram Time Required for Option B
Position*	Application	Application
39	≤ 0.875 sec.	≤ 0.672 sec.

### Table 5-1 Scram Times Required for Option A and Option B Application at Notch Position 39 (References 10 and 11)

\* - The insertion time to a notch position is conservatively calculated using the CRD reed switch drop-out time per Reference 11.

# 5.3. Recirculation Flow Control Valve Settings

Cycle 16 was analyzed with a maximum core flow runout of 105%; therefore the recirculation pump flow control valves must be set to maintain core flow less than 105% (113.925 Mlbm/hr) for all runout events.

	DLO/	Exposure		Optior	ו A		Optior	ו B
Application Group	SLO	Range	GNF2	GNF3	ATRM10/ ATRM10XM	GNF2	GNF3	ATRM10/ ATRM10XM
Base Case	DLO	BOC-MOC	1.54	1.55	1.51	1.51	1.51	1.51
Dase Case	DLU	MOC-EOC	1.63	1.64	1.53	1.58	1.59	1.51
Base Case	SLO	BOC-MOC	1.59	1.59	1.51	1.59	1.59	1.51
	JEO	MOC-EOC	1.66	1.67	1.56	1.61	1.62	1.53
Base Case + TCVSC	DLO	BOC-MOC	1.61	1.62	1.59	1.51	1.52	1.51
+ RPTOOS + PROOS		MOC-EOC	1.70	1.72	1.72	1.60	1.62	1.55
Base Case + TCVSC	SLO	BOC-MOC	1.64	1.65	1.62	1.59	1.59	1.51
+ RPTOOS + PROOS	310	MOC-EOC	1.73	1.75	1.75	1.63	1.65	1.58
Base Case + TCVSC +	DLO	BOC-MOC	1.57	1.58	1.51	1.52	1.53	1.51
TBVOOS (all 5 valves)	DLO	MOC-EOC	1.66	1.68	1.57	1.61	1.63	1.54
Base Case + TCVSC +	SLO	BOC-MOC	1.60	1.61	1.54	1.59	1.59	1.51
TBVOOS (all 5 valves)	320	MOC-EOC	1.69	1.71	1.60	1.64	1.66	1.57
Base Case + TCVSC + TBVOOS (all 5 valves) +	DLO	BOC-MOC	1.65	1.66	1.62	1.55	1.56	1.51
RPTOOS + PROOS		MOC-EOC	1.75	1.76	1.75	1.65	1.66	1.58
Base Case + TCVSC + TBVOOS (all 5 valves) +	SLO	BOC-MOC	1.68	1.69	1.65	1.59	1.59	1.54
RPTOOS + PROOS		MOC-EOC	1.78	1.79	1.78	1.68	1.69	1.61

# Table 5-2 Operating Limit Minimum Critical Power Ratio (OLMCPR) for GNF2, GNF3, ATRIUM-10, and ATRIUM-10XM Fuel (Reference 7)

#### K<sub>P</sub>, MCPR Limit Multiplier (as a function of % rated power) **Application Group** 0% P 25% P 45% P 60% P 85% P 85.01%P 100% P **Base Case** 1.338 1.338 1.191 1.191 1.061 1.061 1.000 Base Case + TCVSC + RPTOOS + 1.488 1.488 1.378 1.296 1.174 1.097 1.000 PROOS Base Case + TCVSC + TBVOOS (all 5 1.379 1.379 1.228 1.207 1.097 1.097 1.000 valves) Base Case + TCVSC + TBVOOS (all 5 1.488 1.378 1.296 1.488 1.174 1.097 1.000 valves) + **RPTOOS + PROOS**

### Table 5-3 Power-Dependent MCPR Multipliers (K<sub>P</sub>) for GNF2, GNF3, ATRIUM-10, and ATRIUM-10XM Fuel, DLO and SLO, BOC to EOC, Option A and Option B (Reference 7)

# Table 5-4 DLO Flow-Dependent MCPR Limits (MCPR<sub>F</sub>) for GNF2, GNF3, ATRIUM-10, and ATRIUM-10XM Fuel, BOC to EOC, All Application Groups, Option A and Option B

(Reference 7)

Flow (% Rated)	MCPR <sub>F</sub>
0.0	1.91
30.0	1.72
105.0	1.25

# Table 5-5 SLO Flow-Dependent MCPR Limits (MCPR<sub>F</sub>) for GNF2, GNF3, ATRIUM-10, and ATRIUM-10XM Fuel, BOC to EOC, All Application Groups, Option A and Option B

(References 7 and 12)

Flow (% Rated)	MCPR <sub>F</sub>
0.0	1.94
30.0	1.75
105.0	1.28

# 6. Linear Heat Generation Rate

### Technical Specification Sections 3.2.3 and 3.4.1

The linear heat generation rate (LHGR) limit is the product of the exposure dependent LHGR limit from Table 6-1 or Table 6-2 and the minimum of: the power dependent LHGR Factor, LHGRFAC<sub>P</sub>, or the flow dependent LHGR Factor, LHGRFAC<sub>F</sub> as applicable. The LHGRFAC<sub>P</sub> multiplier is determined from Table 6-3. The LHGRFAC<sub>F</sub> multiplier is determined from either Table 6-4 or Table 6-5. The SLO multipliers in Tables 6-4 and 6-5 have been limited to a maximum value of 0.78, the SLO LHGR multiplier for GNF2, GNF3, ATRIUM-10, and ATRIUM-10XM fuel.

Peak Pellet Exposure	UO₂ LHGR Limit
See Table B-1 o	f Reference 8
Peak Pellet Exposure	Most Limiting
eak Pellet Exposure See Table B-2 of	Most Limiting Gadolinia LHGR Limit

#### Table 6-1 LHGR Limit for GNF2 and GNF3 Fuel (References 8 and 13)

### Table 6-2 LHGR Limit for ATRIUM-10 and ATRIUM-10XM Fuel (Reference 9)

Peak Pellet Exposure (GWd/ST)	LHGR Limit (kW/ft)
0.0	13.4
16.06	13.4
55.43	9.1
63.87	7.3

Application Group	LHGRFAC <sub>P</sub> (as a function of % rate			rated pow	ted power)	
	0% P	25% P	45% P	60% P	85% P	100% P
Base Case	0.608	0.608	0.713	0.791	0.922	1.000
Base Case + TCVSC + RPTOOS + PROOS	0.608	0.608	0.713	0.761	0.831	1.000
Base Case + TCVSC + TBVOOS (all 5 valves)	0.608	0.608	0.713	0.791	0.922	1.000
Base Case + TCVSC + TBVOOS (all 5 valves) + RPTOOS + PROOS	0.608	0.608	0.713	0.761	0.822	1.000

### Table 6-3 Power-Dependent LHGR Multipliers (LHGRFAC<sub>P</sub>) for GNF2, GNF3, ATRIUM-10, and ATRIUM-10XM Fuel, DLO and SLO, BOC to EOC (Reference 7)

### Table 6-4 Flow-Dependent LHGR Multipliers (LHGRFAC<sub>F</sub>) for GNF2, GNF3, ATRIUM-10, and ATRIUM-10XM Fuel, BOC to EOC, Pressurization (1 TCV/TSV Closed or OOS), All Application Groups (Reference 7)

Flow (% Rated)	DLO LHGRFAC <sub>F</sub>	SLO LHGRFAC <sub>F</sub>
0.0	0.110	0.110
30.0	0.410	0.410
67.0	0.780	0.780
89.0	1.000	0.780
105.0	1.000	0.780

### Table 6-5 Flow-Dependent LHGR Multipliers (LHGRFAC<sub>F</sub>) for GNF2, GNF3, ATRIUM-10, and ATRIUM-10XM Fuel, BOC to EOC, No Pressurization (All TCV/TSV In-Service), All Application Groups (Reference 7)

Flow (% Rated)	DLO LHGRFAC <sub>F</sub>	SLO LHGRFAC <sub>F</sub>
0.0	0.250	0.250
30.0	0.550	0.550
53.0	0.780	0.780
75.0	1.000	0.780
105.0	1.000	0.780

# 7. Rod Block Monitor

### Technical Specification Sections 3.3.2.1 and 3.4.1

The Rod Block Monitor Upscale Instrumentation Setpoints are determined from the relationships shown below (Reference 3):

Rod Block Monitor Upscale Trip Function	Allowable Value
Two Recirculation Loop Operation	0.66 W <sub>d</sub> + 54.0%
Single Recirculation Loop Operation	0.66 W <sub>d</sub> + 48.7%

### **Table 7-1 Rod Block Monitor Setpoints**

The setpoint may be lower/higher and will still comply with the rod withdrawal error (RWE) analysis because RWE is analyzed unblocked. The allowable value is clamped with a maximum value not to exceed the allowable value for a recirculation loop drive flow ( $W_d$ ) of 100%.

W<sub>d</sub> – percent of recirculation loop drive flow required to produce a rated core flow of 108.5 Mlbm/hr.

# 8. Traversing In-Core Probe System

## 8.1. Description

When the traversing in-core probe (TIP) system (for the required measurement locations) is used for recalibration of the LPRM detectors and monitoring thermal limits, the TIP system shall be operable with the following:

- 1. movable detectors, drives and readout equipment to map the core in the required measurement locations, and
- 2. indexing equipment to allow all required detectors to be calibrated in a common location.

The following applies for use with 3DM (Reference 4):

The total number of failed and/or bypassed LPRMs does not exceed 25%. In addition, no more than 14 TIP channels can be OOS (failed or rejected).

Otherwise, with the TIP system inoperable, suspend use of the system for the above applicable calibration functions.

### 8.2. Bases

The operability of the TIP system with the above specified minimum complement of equipment ensures that the measurements obtained from use of this equipment accurately represent the spatial neutron flux distribution of the reactor core. The normalization of the required detectors is performed internal to the core monitoring software system.

# 9. Stability Protection Setpoints

Technical Specification Section 3.3.1.3

### Table 9-1 OPRM PBDA Trip Setpoints

(Reference 7)

PBDA Trip Amplitude Setpoint (Sp)	Corresponding Maximum Confirmation Count Setpoint (Np)
1.11	14

The PBDA is the only OPRM setting credited in the safety analysis as documented in the licensing basis for the OPRM system.

The OPRM PBDA trip settings are based, in part, on the cycle specific OLMCPR and the power and flow dependent MCPR limits. Any change to the OLMCPR values and/or the power and flow dependent MCPR limits should be evaluated for potential impact on the OPRM PBDA trip settings.

The OPRM PBDA trip settings are applicable when the OPRM system is declared operable, and the associated Technical Specifications are implemented.

# 10. Modes of Operation

The allowed modes of operation with combinations of equipment out-of-service are as described below (Reference 7).

### Table 10-1 Allowed Modes of Operation and EOOS Combinations (References 4 and 7)

Equipment Out of Service Options <sup>(1) (2) (4) (5)</sup>	Short Name
Base Case (Option A or B) <sup>(3)</sup>	Base
Base Case + SLO (Option A or B)	Base SLO
Base Case + TCVSC + RPTOOS + PROOS (Option A or B)	Combined EOOS 1
Base Case + TCVSC + RPTOOS + PROOS + SLO (Option A or B)	Combined EOOS 1 SLO
Base Case + TCVSC + TBVOOS (all 5 valves) (Option A or B)	Combined EOOS 2
Base Case + TCVSC + TBVOOS (all 5 valves) + SLO (Option A or B)	Combined EOOS 2 SLO
Base Case + TCVSC + TBVOOS (all 5 valves) + RPTOOS + PROOS (Option A or B)	Combined EOOS 3
Base Case + TCVSC + TBVOOS (all 5 valves) + RPTOOS + PROOS + SLO (Option A or B)	Combined EOOS 3 SLO

(1) Base case includes 1 SRVOOS + 1 TCV/TSV OOS + FWHOOS/FFWTR + 1 MSIVOOS + 2 TBVOOS + PLUOOS (Reference 7). All Modes of Operation and EOOS Combinations allow 1 TIPOOS (up to 14 TIP channels not available) any time during the cycle, including BOC, and up to 25% of the LPRMs out-of-service (Reference 4). The FWHOOS/FFWTR analyses cover a maximum reduction of 100°F for the feedwater temperature. A nominal LPRM calibration interval of 2000 EFPH (2500 EFPH maximum) is supported for L2C16.

(2) TBVOOS (all 5 valves) is the turbine bypass system out of service which means that 5 TBVs are <u>not</u> credited for fast opening and 3 TBVs are <u>not</u> credited to open in pressure control. For the 2 TBVOOS condition that is a part of the base case, the assumption is that both of the TBVs do not open on any signal and thus remain shut for the transients analyzed (i.e. 3 TBVs are credited to open in pressure control) (Reference 10). The MCFL is currently set at 126.6 and will only allow opening of TBV's #1, #2, #3, and #4 during a slow pressurization event. The MCFL does not use the TBV position feedback signal to know how many TBVs have opened or how far each has opened. The #5 TBV is not available based on the current MCFL setpoint and thus cannot be used as one of the credited valves to open in pressure control.

(3) With all TCV/TSV In-Service, the Base Case should be used with the LHGRFAC<sub>F</sub> values from Table 6-5 (Reference 7). With 1 TCV/TSV OOS, the Base Case must be used with the LHGRFAC<sub>F</sub> values from Table 6-4. The one Stuck Closed TCV and/or TSV EOOS conditions require power level  $\leq$  85% of rated. The one MSIVOOS condition is also supported as long as thermal power is maintained  $\leq$  75% of the rated (Reference 7 Appendix D).

(4) The + sign that is used in the Equipment Out of Service Option / Application Group descriptions designates an "and/or".

(5) All EOOS Options (Reference 7 Application Groups) are applicable to ELLLA, MELLLA, ICF and Coastdown realms of operation with the exception that SLO is not applicable to MELLLA or ICF (References 5 and 6). The MOC to EOC exposure range limit sets are generated by GNF to include application to coastdown operation (Methodology Reference 5).

# 11. Methodology

The analytical methods used to determine the core operating limits shall be those previously reviewed and approved by the NRC, specifically those described in the following documents:

- 1. XN-NF-81-58 (P)(A), Revision 2 and Supplements 1 and 2, "RODEX2 Fuel Rod Thermal-Mechanical Response Evaluation Model," March 1984.
- 2. ANF-89-98 (P)(A), Revision 1 and Supplement 1, "Generic Mechanical Design Criteria for BWR Fuel Designs," May 1995.
- 3. EMF-85-74 (P) Revision 0 Supplement 1(P)(A) and Supplement 2(P)(A), "RODEX2A (BWR) Fuel Rod Thermal-Mechanical Evaluation Model," February 1998.
- XN-NF-85-67 (P)(A) Revision 1, "Generic Mechanical Design for Exxon Nuclear Jet Pump BWR Reload Fuel," September 1986.
- 5. NEDE-24011-P-A-20 (Revision 20), "General Electric Standard Application for Reactor Fuel," December 2013 and the U.S. Supplement NEDE-24011-P-A-20-US, of December 2013.
- 6. NEDC-33106P-A Revision 2, "GEXL97 Correlation for ATRIUM-10 Fuel," June 2004.
- 7. NEDO-32465-A, "BWR Owner's Group Reactor Stability Detect and Suppress Solutions Licensing Basis Methodology for Reload Applications," August 1996.