



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

February 25, 2015

Mr. Louis P. Cortopassi
Site Vice President and Chief Nuclear Officer
Omaha Public Power District
Fort Calhoun Station
9610 Power Lane, Mail Stop FC-2-4
Omaha, NE 68008

SUBJECT: FORT CALHOUN STATION, UNIT NO. 1 – REQUEST FOR ADDITIONAL INFORMATION ROUND 2 RE: LICENSE AMENDMENT REQUEST TO ADOPT ASME CODE, SECTION III, 1980 EDITION (NO ADDENDA) AS AN ALTERNATIVE TO CURRENT CODE OF RECORD (TAC NO. MF4160)

Dear Mr. Cortopassi:

By letter dated May 16, 2014, as supplemented by letter dated January 9, 2015 (Agencywide Documents Access and Management System (ADAMS) Accession Nos. ML14143A370 and ML15009A319, respectively), Omaha Public Power District submitted a license amendment request to revise the current licensing basis to adopt the American Society of Mechanical Engineers *Boiler and Pressure Vessel Code*, Section III, 1980 Edition (no Addenda), as an alternative to the current Code of record.

The U.S. Nuclear Regulatory Commission staff has reviewed the information provided in your application and determined that additional information is required in order to complete its formal review. The enclosed questions were provided to B. Hansher of your staff on February 12, 2015. Please provide a response to the enclosed questions within 30 days of the date of this letter. If you have any questions, please contact me at 301-415-2296 or via e-mail at Fred.Lyon@nrc.gov.

Sincerely,

A handwritten signature in black ink that reads "CF Lyon".

Carl F. Lyon, Project Manager
Plant Licensing Branch IV-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket No. 50-285

Enclosure:
Request for Additional Information

cc w/encl: Distribution via Listserv



UNITED STATES
NUCLEAR REGULATORY COMMISSION
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REQUEST FOR ADDITIONAL INFORMATION

OMAHA PUBLIC POWER DISTRICT

FORT CALHOUN STATION, UNIT NO. 1

DOCKET NO. 50-285

By letter dated May 16, 2014, as supplemented by letter dated January 9, 2015 (Agencywide Documents Access and Management System (ADAMS) Accession Nos. ML14143A370 and ML15009A319, respectively), Omaha Public Power District (the licensee) submitted a license amendment request to revise the current licensing basis to adopt the American Society of Mechanical Engineers *Boiler and Pressure Vessel Code* (ASME Code), Section III, 1980 Edition (no Addenda), as an alternative to the current Code of record.

The U.S. Nuclear Regulatory Commission staff has reviewed the information provided in the licensee's application and determined that the additional information below is required in order to complete its formal review.

1. Please verify that the equation numbers listed in the Load Combination Table associated with Note (e) are from the ASME Code, Section III, 1980 Edition (no Addenda), and indicate the proposed changes as necessary.
2. Table F-1 provides a list of Load Combinations; however, only deadweight loads are considered for Service Level A (equation 8 of ASME Code, Section III, 1980 Edition). According to the ASME Code, all sustained loads (including, but not limited to, deadweight loads) must be considered for steam pipe lines or air service piping lines which do not contain water. Please explain the basis for limiting the load combinations to only deadweight loads.
3. Service Level C for Note (e) refers to Note (d), which states that the listed load cases and limits apply only to the pressurizer relief valve piping and supports (reactor coolant system (RCS) as defined in Updated Safety Analysis Report (USAR) Table 4.3-6). However, Note (e) of the proposed USAR change, applies only to non-RCS safety-related piping analysis. Therefore, load combinations for non-RCS piping system should not be limited to RCS fluid transient loading as indicated in Service Level C. Please clarify this apparent inconsistency.
4. The licensee indicated that dynamic effects, as applicable to specific operating and accident conditions, were considered in the piping and component design. However, Table F-1 in the proposed USAR change does not address occasional dynamic loadings. Please explain this discrepancy and address where loading from pipe breaks is included.

Enclosure

February 25, 2015

Mr. Louis P. Cortopassi
Site Vice President and Chief Nuclear Officer
Omaha Public Power District
Fort Calhoun Station
9610 Power Lane, Mail Stop FC-2-4
Omaha, NE 68008

SUBJECT: FORT CALHOUN STATION, UNIT NO. 1 – REQUEST FOR ADDITIONAL INFORMATION ROUND 2 RE: LICENSE AMENDMENT REQUEST TO ADOPT ASME CODE, SECTION III, 1980 EDITION (NO ADDENDA) AS AN ALTERNATIVE TO CURRENT CODE OF RECORD (TAC NO. MF4160)

Dear Mr. Cortopassi:

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Sincerely,
/RA/

Carl F. Lyon, Project Manager
Plant Licensing Branch IV-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket No. 50-285

Enclosure:
Request for Additional Information

cc w/encl: Distribution via Listserv

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