



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

February 19, 2015

Mr. Lawrence J. Weber
Senior Vice President and
Chief Nuclear Officer
Indiana Michigan Power Company
Nuclear Generation Group
One Cook Place
Bridgman, MI 49106

SUBJECT: DONALD C. COOK NUCLEAR PLANT, UNIT 2 - CORRECTION OF
TECHNICAL SPECIFICATION TYPOGRAPHICAL ERROR INCURRED
DURING LICENSE AMENDMENT NO. 304 (TAC NO. MF5673)

Dear Mr. Weber:

By letter dated March 22, 2013 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML13046A114), the U.S. Nuclear Regulatory Commission (NRC) issued Amendment Nos. 320 and 304 to Renewed Facility Operating License Nos. DPR-58 and DPR-74 for Donald C. Cook Nuclear Plant (CNP), Units 1 and 2. The amendments modified the technical specifications (TSs) to adopt Technical Specifications Task Force (TSTF) Change Traveler TSTF-510.

On February 2, 2015, the NRC was notified by letter from Michael Scarpello, Regulatory Affairs Manager (ADAMS Accession No. ML15035A065), that errors had been identified page 5.6-4 of the TSs for CNP Unit 2. Specifically, an extraneous reference was listed under section 5.6.5 CORE OPERATING LIMITS REPORT (COLR) (continued). Also, in Section 5.6.6 Post Accident Monitoring Report, "Condition B or G of LCO 3.3.3" was incorrectly written as "Condition B or H of LCO 3.3.3." Condition H of LCO 3.3.3 does not exist in the CNP Unit 2 TSs. Both errors were introduced during the issuance of License Amendment No. 304.

The NRC staff has determined that these were inadvertent typographical errors and are entirely editorial in nature. The proposed corrections do not change any of the conclusions in the safety evaluation associated with Amendment No. 304 for CNP Unit 2, and do not affect the associated notice to the public. Please find enclosed the replacement page.

L. Weber

- 2 -

If you have any questions regarding this matter, please contact me at (301) 415-8371.

Sincerely,

A handwritten signature in black ink, appearing to read "Chawla M".

Mahesh L. Chawla, Project Manager
Plant Licensing Branch III-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket No. 50-316

Enclosure:
Corrected Pages to License Amendment No. 304

cc w/encl: Distribution via ListServ

ENCLOSURE

DONALD C. COOK NUCLEAR PLANT, UNIT 2

DOCKET NO. 50-316

CORRECTED PAGES TO LICENSE AMENDMENT NO. 304

TECHNICAL SPECIFICATIONS
PAGE 5.6-4

5.6 Reporting Requirements

5.6.5 CORE OPERATING LIMITS REPORT (COLR) (continued)

7. WCAP-13749-P-A, "Safety Evaluation Supporting the Conditional Exemption of the Most Negative EOL Moderator Temperature Coefficient Measurement," (Westinghouse Proprietary); and
 8. WCAP-12610-P-A & CENPD-404-P-A, Addendum 1-A, "Optimized ZIRLO™," July 2006 (Westinghouse Proprietary)
- c. The core operating limits shall be determined such that all applicable limits (e.g., fuel thermal mechanical limits, core thermal hydraulic limits, Emergency Core Cooling Systems (ECCS) limits, nuclear limits such as SDM, transient analysis limits, and accident analysis limits) of the safety analysis are met.
 - d. The COLR, including any midcycle revisions or supplements, shall be provided upon issuance for each reload cycle to the NRC.

5.6.6 Post Accident Monitoring Report

When a report is required by Condition B or G of LCO 3.3.3, "Post Accident Monitoring (PAM) Instrumentation," a report shall be submitted within the following 14 days. The report shall outline the preplanned alternate method of monitoring, the cause of the inoperability, and the plans and schedule for restoring the instrumentation channels of the Function to OPERABLE status.

5.6.7 Steam Generator Tube Inspection Report

A report shall be submitted within 180 days after the initial entry into MODE 4 following completion of an inspection performed in accordance with the Specification 5.5.7, Steam Generator (SG) Program. The report shall include:

- a. The scope of inspections performed on each SG,
- b. Degradation mechanisms found,
- c. Nondestructive examination techniques utilized for each degradation mechanism,
- d. Location, orientation (if linear), and measured sizes (if available) of service induced indications,
- e. Number of tubes plugged during the inspection outage for each degradation,
- f. The number and percentages of tubes plugged to date, and the effective plugging percentage in each generator, and

L. Weber

- 2 -

If you have any questions regarding this matter, please contact me at (301) 415-8371.

Sincerely,

/RA/

Mahesh L. Chawla, Project Manager
Plant Licensing Branch III-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket No. 50-316

Enclosure:
Corrected Pages to License Amendment No. 304

cc w/encl: Distribution via ListServ

DISTRIBUTION:

PUBLIC
LPL3-1 r/f
RidsNrrDorlLpl3-1 Resource
RidsNrrPMDCCook Resource
RidsNrrLAMHendersonResource
RidsAcrsAcnw_MailCTRRResource
RidsRgn3MailCenterResource
RecordsAmend Resource

ADAMS Accession No.: ML15041A723

OFFICE	LPL3-1/PMiT	LPL3-1/PM	LPL3-1/LA	DSS/STSB/BC	LPL3-1/BC	LPL3-1/PM
NAME	ADietrich	MChawla	MHenderson	RElliott	DPelton	MChawla
DATE	2/10/2015	02/11/2015	02/12/2015	2/19/2015	02/19/2015	02/19/2015

OFFICIAL RECORD COPY