UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

April 3, 2015

Mr. Bryan C. Hanson
President and Chief Nuclear Officer
Exelon Nuclear
4300 Winfield Road
Warrenville, IL 60555

SUBJECT: OYSTER CREEK NUCLEAR GENERATING STATION - ISSUANCE OF AMENDMENT TO REVISE TECHNICAL SPECIFICATIONS FOR THE SNUBBER SURVEILLANCE REQUIREMENTS (TAC NO. MF4124)

Dear Mr. Hanson:

The U.S. Nuclear Regulatory Commission has issued the enclosed Amendment No. 286 to Renewed Facility Operating License No. DPR-16 and the technical specification (TS) for Oyster Creek Nuclear Generating Station (OCNGS), in response to your application dated April 30, 2014, as supplemented by letter dated October 16, 2014.

The amendment revises TS 4.5.M, "Shock Suppressors (Snubbers)," to delete detailed snubber inservice inspection (ISI) and testing requirements and to add Administrative Control, TS 6.25, "Snubber Inspection Program," which includes the planned revisions to the OCNGS snubber ISI, testing, and service life monitoring programs. The fifth 10-year ISI interval at OCNGS began on January 15, 2013, and is scheduled to end January 14, 2023. Currently, snubber examination and testing are performed in accordance with the specific requirements of TS 4.5.M and the OCNGS plant procedures. For the remainder of the fifth 10-year ISI interval at OCNGS, the snubber program will meet the requirements of the American Society of Mechanical Engineers Code for Operation and Maintenance of Nuclear Power Plants, Subsection ISTD, Edition 2004 with 2005 and 2006 Addenda per Title 10 of the *Code of Federal Regulations* (10 CFR), Section 50.55a(b)(3)(v), "OM condition: Snubbers ISTD."

A copy of the related Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly *Federal Register* notice.

Sincerely,

John G. Lamb, Senior Project Manager
Plant Licensing Branch I-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket No. 50-219

Enclosures:
1. Amendment No. 286 to DPR-16
2. Safety Evaluation

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**UNITED STATES**
**NUCLEAR REGULATORY COMMISSION**
**WASHINGTON, D.C. 20555-0001**

EXELON GENERATION COMPANY, LLC

DOCKET NO. 50-219

OYSTER CREEK NUCLEAR GENERATING STATION

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 286
Renewed License No. DPR-16

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Exelon Generation Company, LLC (the licensee), dated April 30, 2014, as supplemented by letter dated October 16, 2014, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C(2) of Renewed Facility Operating License No. DPR-16 is hereby amended to read as follows:

   (2) Technical Specifications

   The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 286, are hereby incorporated in the license. Exelon Generation Company shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of issuance, and shall be implemented within 60 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

for

Douglas A. Broaddus, Chief
Plant Licensing Branch I-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Renewed License and
Technical Specifications

Date of Issuance: April 3, 2015

ATTACHMENT TO LICENSE AMENDMENT NO. 286

RENEWED FACILITY OPERATING LICENSE NO. DPR-16

DOCKET NO. 50-219

Replace the following page of the Facility Operating License with the attached revised page. The revised page is identified by amendment number and contains marginal lines indicating the areas of change.

| Remove | Insert |
|---|---|
| Page 3 | Page 3 |

Replace the following pages of the Appendix A, Technical Specifications, with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

| Remove | Insert |
|---|---|
| iii | iii |
| 4.5-6 | 4.5-6 |
| 4.5-7 | 4.5-7 |
| 4.5-8 | 4.5-8 |
| 4.5-9 | 4.5-9 |
| 4.5-17 | 4.5-17 |
| 4.5-18 | 4.5-18 |
| 6-18 | 6-18 |
| 6-23 | 6-23 |

(3) Pursuant to the Act and 10 CFR Parts 30, 40, and 70, to receive, possess, and use at any time any byproduct, source, or special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;

(4) Pursuant to the Act and 10 CFR Parts 30, 40, and 70, to receive, possess and use in amounts as required any byproduct source, or special nuclear materials without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components; and

(5) Pursuant to the Act and 10 CFR Parts 30, 40, and 70, to possess, but not separate such byproduct, source, or special nuclear materials as may be produced by the operation of the facility.

C. This license shall be deemed to contain and is subject to the conditions specified in the Commission's regulations set forth in 10 CFR Chapter I and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect and is subject to the additional conditions specified or incorporated below:

(1) Maximum Power Level

Exelon Generation Company is authorized to operate the facility at steady-state power levels not in excess of 1930 megawatts (thermal) (100 percent rated power) in accordance with the conditions specified herein.

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 286, are hereby incorporated in the license. Exelon Generation Company shall operate the facility in accordance with the Technical Specifications.

(3) Fire Protection

Exelon Generation Company shall implement and maintain in effect all provisions of the approved fire protection program as described in the Updated Final Safety Analysis Report for the facility and as approved in the Safety Evaluation Report dated March 3, 1978, and supplements thereto, subject to the following provision:

The licensee may make changes to the approved fire protection program without prior approval of the Commission only if those changes would not adversely affect the ability to achieve and maintain safe shutdown in the event of a fire.

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*Issued by NRC Order dated 10-24-80

Leakage at instrument fittings and valves
Venting an unisolated instrument or instrument line
Flushing or draining an instrument
Installation of a new instrument or instrument line

L. Suppression Chamber Surveillance

1. At the frequency specified in the Surveillance Frequency Control Program, the suppression chamber water level and temperature and pressure suppression system pressure shall be checked.

2. A visual inspection of the suppression chamber interior, including water line regions, shall be made at the frequency specified in the Surveillance Frequency Control Program.

3. Whenever heat from relief valve operation is being added to the suppression pool, the pool temperature shall be continually monitored and also observed until the heat addition is terminated.

4. Whenever operation of a relief valve is indicated and the suppression pool temperature reaches 160°F or above while the reactor primary coolant system pressure is greater than 180 psig, an external visual examination of the suppression chamber shall be made before resuming normal power operation.

M. Shock Suppressors (Snubbers)

As used in this specification, "type of snubber" shall mean snubbers of the same design and manufacturer, irrespective of capacity.

1. Each snubber shall be demonstrated OPERABLE by performance of the Snubber Inspection Program.

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N. Secondary Containment Isolation Valves

1. Each secondary containment isolation valve shall be demonstrated operable prior to returning the valve to service after maintenance, repair or replacement work is performed on the valve or its associated actuator by cycling the valve through at least one complete cycle of full travel. Following maintenance, repair or replacement work on the control or power circuit for the valves, the affected component shall be tested to assure it will perform its intended function in the circuit.

2. At the frequency specified in the Surveillance Frequency Control Program, all valves shall be tested for automatic closure by an isolation signal.

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k. Records of Environmental Qualification which are covered under the provisions for paragraph 6.14.

l. Deleted.

m. Records of results of analyses required by the Radiological Environmental Monitoring Program.

n. Records of reviews performed for changes made to the OFFSITE DOSE CALCULATION MANUAL and the PROCESS CONTROL PLAN.

o. Records of radioactive shipments

6.10.3 Quality Assurance Records shall be retained as specified by the QATR.

6.11 RADIATION PROTECTION PROGRAM

Procedures for personnel radiation protection shall be prepared consistent with the requirements of 10 CFR 20 and shall be approved, maintained and adhered to for all operations involving personnel radiation exposure.

6.12 (Deleted)

6.13 HIGH RADIATION AREA

6.13.1 In lieu of the "control device" or "alarm signal" required by Section 20.1601 of 10 CFR 20, each high radiation area in which the intensity of radiation at 30 cm (11.8 in.) is greater than deep dose equivalent of 100 mRem/hr but less than 1,000 mRem/hr shall be barricaded and conspicuously posted as a high radiation area and entrance thereto shall be controlled by requiring issuance of a Radiation Work Permit (RWP).

NOTE: Health Physics personnel shall be exempt from the RWP issuance requirement during the performance of their assigned radiation protection duties, provided they are following plant radiation protection procedures for entry into high radiation areas.

An individual or group of individuals permitted to enter such areas shall be provided with one or more of the following:

a. A radiation monitoring device which continuously indicates the radiation dose rate in the area.

b. A radiation monitoring device which continuously integrates the radiation dose rate in the area and alarms when a pre-set integrated dose is received. Entry into such areas with this monitoring device may be made after the dose rate levels in the area have been established and personnel have been made knowledgeable of them.

c. A health physics qualified individual (i.e., qualified in radiation protection procedures) with a radiation dose rate monitoring device who is responsible for providing positive exposure control over the activities within the area and who will perform periodic radiation surveillance at the frequency in the RWP. The surveillance frequency will be established by the management position responsible for radiological controls.

6.24 SURVEILLANCE FREQUENCY CONTROL PROGRAM

This program provides controls for Surveillance Frequencies. The program shall ensure that Surveillance Requirements specified in the Technical Specifications are performed at intervals sufficient to assure the associated Limiting Conditions for Operation are met.

a. The Surveillance Frequency Control Program shall contain a list of Frequencies of those Surveillance Requirements for which the Frequency is controlled by the program.

b. Changes to the Frequencies listed in the Surveillance Frequency Control Program shall be made in accordance with NEI 04-10, "Risk-Informed Method for Control of Surveillance Frequencies," Revision 1.

c. The provisions of Definition 1.24 and Surveillance Requirement 4.0.2 are applicable to the Frequencies established in the Surveillance Frequency Control Program.

6.25 SNUBBER INSPECTION PROGRAM

This program conforms to the examination, testing, and service life monitoring for dynamic restraints (snubbers) in accordance with 10 CFR 50.55a inservice inspection (ISI) requirements for supports. The program shall be in accordance with the following:

a. This program shall meet 10 CFR 50.55a(g) ISI requirements for supports.

b. The program shall meet the requirements for ISI of supports set forth in subsequent editions of the Code of Record and addenda of the American Society of Mechanical Engineers (ASME) Boiler and Pressure Vessel (B&PV) Code and the ASME Code for Operation and Maintenance of Nuclear Power Plants (OM Code) that are incorporated by reference in 10 CFR 50.55a(a), 50.55a(a)(1), 50.55a(a)(1)(i), and 50.55a(a)(1)(iv), subject to its limitations and modifications, and subject to Commission approval.

c. The program shall, as allowed by 10 CFR 50.55a(b)(3)(v)(B), meet Subsection ISTA, "General Requirements," and Subsection ISTD, "Preservice and Inservice Examination and Testing of Dynamic Restraints (Snubbers) in Light-Water Reactor Nuclear Power Plants," in lieu of Section XI of the ASME B&PV Code ISI requirements for snubbers, or meet authorized alternatives pursuant to 10 CFR 50.55a(z).

d. The 120-month program updates shall be made in accordance with 10 CFR 50.55a (including 10 CFR 50.55a(g)(4)(ii)) subject to the conditions listed therein.

e. Records of the service life of all snubbers, including the date which the service life commences, and associated installation and maintenance records shall be maintained for the duration of the Facility Operating License.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

# SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

# RELATED TO AMENDMENT NO. 286

# TO RENEWED FACILITY OPERATING LICENSE NO. DPR-16

# EXELON GENERATION COMPANY, LLC

# OYSTER CREEK NUCLEAR GENERATING STATION

# DOCKET NO. 50-219

## 1.0 INTRODUCTION

By letter dated April 30, 2014 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML14127A435), as supplemented by letter dated October 16, 2014 (ADAMS Accession No. ML14294A056), Exelon Generation Company, LLC (Exelon or the licensee) requested changes to the technical specifications (TSs) for Oyster Creek Nuclear Generating Station (Oyster Creek or OCNGS). Specifically, the licensee requested to revise TS 4.5.M, "Shock Suppressors (Snubbers)," to delete detailed snubber inservice inspection (ISI) and testing requirements, and add Administrative Control TS 6.25, "Snubber Inspection Program," which includes the planned revisions to the OCNGS snubber ISI, testing, and service life monitoring programs.

The fifth 10-year ISI interval at OCNGS began on January 15, 2013, and is scheduled to end January 14, 2023. Currently, snubber examination and testing are performed in accordance with the specific requirements of TS 4.5.M and the OCNGS plant procedures. For the remainder of the fifth 10-year ISI interval at OCNGS, the snubber program will meet the requirements of the American Society of Mechanical Engineers Code for Operation and Maintenance of Nuclear Power Plants (ASME OM Code), Subsection ISTD, Edition 2004 with 2005 and 2006 Addenda per Title 10 of the *Code of Federal Regulations* (10 CFR), Section 50.55a(b)(3)(v).

The supplemental letter dated October 16, 2014, provided additional information that clarified the application, did not expand the scope of the application as originally noticed, and did not change the U.S. Nuclear Regulatory Commission (NRC) staff's original proposed no significant hazards consideration determination as published in the *Federal Register* on July 8, 2014 (79 FR 38590).

## 2.0 REGULATORY EVALUATION

A dynamic restraint (snubber) is a device designed to protect components from excess shock or sway caused by seismic disturbances or other transient forces. During normal operating conditions, the snubber allows for movement in tension and compression. When an impulse

event occurs, the snubber becomes activated and acts as a restraint device. The device becomes rigid, absorbs the dynamic energy, and transfers it to the supporting structure. The proposed change, in essence, relocates the specific snubber inservice examination and testing requirements located in TS 4.5.M, and replaces them with a new TS surveillance requirement 4.5.M.1 that requires "[e]ach snubber shall be demonstrated OPERABLE by performance of the Snubber Inspection Program." The Snubber Inspection Program is added in newly added Administrative Control TS Section 6.25, "Snubber Inspection Program."

The revised Snubber Inspection Program includes revisions to the OCNGS snubber ISI, testing, and service life monitoring programs. The purpose of the Snubber Inservice Program, as provided in the letter dated April 30, 2014, and as supplemented by letter dated October 16, 2014, is to provide requirements for the performance and administration of assessing the operational readiness of those dynamic restraints, whose specific functions are required to ensure the integrity of the reactor coolant pressure boundary.

The regulations in 10 CFR 50.36, "Technical Specifications," provide the regulatory requirements for the contents in a licensee's TSs. This regulation requires that the TSs include items in the following five specific categories related to station operation. These categories include: (1) safety limits, limiting safety system settings, and limiting control settings; (2) limiting conditions for operation (LCO); (3) surveillance requirements (SRs); (4) design features; and (5) administrative controls. The regulation does not specify the particular requirements to be included in a plant's TSs. In Item 3, 10 CFR 50.36(c)(3) notes that SRs are requirements relating to test, calibration, or inspection to assure that necessary quality of systems and components is maintained.

In general, there are two classes of changes to TS: (1) changes needed to reflect modifications to the design bases (TS are derived from the design basis); and (2) voluntary changes to take advantage of the evolution in policy and guidance as to the required contents and preferred format of TS over time. This amendment deals with the second class of changes.

Requirements for ISI of Class 1, Class 2, Class 3, Class MC, and Class CC components (including supports) are located in 10 CFR 50.55a(g). The regulations in 10 CFR 50.55a(g)(5)(ii) require, in part, that, if a revised ISI program for a facility conflicts with the TS for that facility, the licensee shall apply to the Commission for amendment of the TS to conform the TS to the revised program. The licensee shall submit this application at least 6 months before the start of the period during which the provisions become applicable.

Licensees are required to perform the ISI and testing of snubbers in accordance with Section XI of the ASME *Boiler and Pressure Vessel Code* (B&PV Code) or the ASME OM Code and the applicable addenda, as required by Section 50.55a(g) of 10 CFR or 10 CFR 50.55a(b)(3)(v), except where the NRC has granted specific relief, pursuant to 10 CFR 50.55a(g)(6)(i), or authorized alternatives pursuant to 10 CFR 50.55a(z). (Note: the paragraph headings in 10 CFR 50.55a were changed by *Federal Register* notice dated November 5, 2014 (79 FR 65776), which became effective December 5, 2014 (e.g., 10 CFR 50.55a(a)(3) is now 10 CFR 50.55a(z). See the cross-reference tables, which are cited in the notice, at ADAMS Accession No. ML14015A191 and ADAMS package Accession No. ML14211A050).

As discussed in NRC Regulatory Issue Summary 2010-06, "Inservice Inspection and Testing Requirements of Dynamic Restraints (Snubbers)," dated June 1, 2010 (ADAMS Accession No. ML101310338), licensees have the option to control the ASME Code-required ISI and testing of snubbers through its TSs or through licensee-controlled documents (e.g., the Technical Requirements Manual).

The regulations in 10 CFR 50.55a(b)(3)(v) allow licensees the option of using the inservice examination and testing provisions for snubbers in ASME B&PV Code Section XI or the ASME OM Code. However, 10 CFR 50.55a(b)(3)(v)(B) states, in part, that licensees shall comply with the provisions for examining and testing snubbers in Subsection ISTD of the ASME OM Code and make appropriate changes to its TS or licensee-controlled documents when using the 2006 Addenda or later editions and addenda of Section XI of the ASME B&PV Code. OCNGS is currently using the 2007 Edition with the 2008 Addenda of the Section XI ASME Code.

The fifth 10-year ISI interval at OCNGS began on January 15, 2013, and is scheduled to end January 14, 2023. Currently, snubber examination and testing are performed in accordance with the specific requirements of TS 4.5.M and the OCNGS plant procedures. For the rest of the fifth 10-year ISI interval at OCNGS, the snubber program will meet the requirements of the ASME OM Code, Subsection ISTD, Edition 2004 with 2005 and 2006 Addenda per 10 CFR 50.55a(b)(3)(v).

## 3.0 TECHNICAL EVALUATION

### 3.1 Licensee's Proposed TS Changes and Basis Information

The proposed changes to the OCNGS TSs are summarized below:

- TS Specification 4.5.M.1 of TS 4.5.M, "Shock Suppressors (Snubbers)," will be revised to replace specific SRs for demonstrating snubber operability with the newly added Administrative Control Section 6.25, "Snubber Inspection Program," as follows:

  TS SR 4.5.M.1 Each snubber shall be demonstrated OPERABLE by performance of the Snubber Inspection Program.

  (Note: All other details about snubber visual inspection, functional testing and service life program requirements as described in TS SR 4.5.M a, b, c, d, e, and f, which are proposed to be deleted, are not being repeated here).

- Section 6.25, "Snubber Inspection Program," will be added to the TS "Administrative Controls," Section 6 to provide a description of the snubber testing program as follows:

  6.25 Snubber Inspection Program

  This program conforms to the examination, testing, and service life monitoring for dynamic restraints (snubbers) in accordance with 10 CFR 50.55a inservice inspection (ISI) requirements for supports. The program shall be in accordance with the following:

  a. This program shall meet 10 CFR 50.55a(g) ISI requirements for supports.

b. The program shall meet the requirements for ISI of supports set forth in subsequent editions of the Code of Record and addenda of the American Society of Mechanical Engineers (ASME) Boiler and Pressure Vessel (B&PV) Code and the ASME Code for Operation and Maintenance of Nuclear Power Plants (OM Code) that are incorporated by reference in 10 CFR 50.55a(a), 50.55a(a)(1), 50.55a(a)(1)(i), and 50.55a(a)(1)(iv), subject to its limitations and modifications, and subject to Commission approval.

c. The program shall, as allowed by 10 CFR 50.55a(b)(3)(v)(B), meet Subsection ISTA, "General Requirements," and Subsection ISTD, "Preservice and Inservice Examination and Testing of Dynamic Restraints (Snubbers) in Light-Water Reactor Nuclear Power Plants," in lieu of Section XI of the ASME B&PV Code ISI requirements for snubbers, or meet authorized alternatives pursuant to 10 CFR 50.55a(z).

d. The 120-month program updates shall be made in accordance with 10 CFR 50.55a (including 10 CFR 50.55a(g)(4)(ii)) subject to the conditions listed therein.

e. Records of the service life of all snubbers, including the date which the service life commences, and associated installation and maintenance records shall be maintained for the duration of the Facility Operating License.

For the fifth 10-year ISI interval, as permitted by 10 CFR 50.55a(b)(3)(v), OCNGS intends to adopt Subsection ISTD, of the ASME OM Code, 2004 Edition with 2005 and 2006 addenda. As such, the proposed changes to TS 4.5.M are required to conform the TSs to this revised snubber program.

3.2 NRC Staff Evaluation of Proposed Changes

The licensee has developed a detailed "Snubber Inservice Program," as a licensee-controlled document at OCNGS, which will be implemented in lieu of the deleted TS snubber examination, testing and service life program requirements. This "Snubber Inspection Program," will contain snubber examination, testing and service life monitoring program and will be based on the requirements referenced in the TS Administrative Control Section 6.25.

The NRC staff has reviewed of the submitted LAR related to the snubber examination and testing program described in TS 4.5.M, "Shock Suppressors (Snubbers)," and concludes that the proposed amendment to revise the Snubbers TS 4.5.M and to add new "Snubber Inspection Program," in the TS Administrative Control Section 6.25 are acceptable. The NRC staff finds that the program requirements are consistent with the ISI and testing requirements for snubbers as required by 10 CFR 50.55a. The NRC staff also notes that the snubber ISI and testing program at OCNGS will be performed per the requirements of the ASME OM Code 2004 Edition with 2005 and 2006 Addenda), Subsection ISTD, as allowed by 10 CFR 50.55a(b)(3)(v)(B). Therefore, the NRC staff concludes that the proposed amendment to revise TS 4.5.M and to add new TS 6.25 is acceptable.

Based on the above finding that the "Snubber Inspection Program" is consistent with 10 CFR 50.55a, the NRC staff further finds that the examination, testing and service life monitoring for snubbers in accordance with the Snubber Inspection Program, as required by TS SR 4.5.M, is sufficient to demonstrate that snubbers are OPERABLE in accordance with TS 4.5.M.1. Therefore, the NRC staff concludes that the proposed changes to TS SR 4.5.M are consistent with the requirements in 10 CFR 50.36(c)(3) and, therefore, are acceptable.

Based on the above considerations, the NRC staff concludes that the proposed amendment is acceptable.

### 3.3 NRC Staff Technical Conclusion

As set forth above, the NRC staff concludes that the proposed amendment to revise the Snubber TS 4.5.M is acceptable. Specifically, the change replaces the TS SR 4.5.M.1 requirements for snubber examination, testing and service life monitoring with newly added ISI program requirements for snubbers contained in TS Administrative Control Section 6.25, "Snubber Inservice Program."

## 4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the New Jersey State official was notified of the proposed issuance of the amendment. The State official had no comments.

## 5.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes SRs. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding on July 8, 2014 (79 FR 38590). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

## 6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) there is reasonable assurance that such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: G. Bedi

Date of Issuance: April 3, 2015

A copy of the related Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly *Federal Register* notice.

Sincerely,

***/RA/***

John G. Lamb, Senior Project Manager
Plant Licensing Branch I-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket No. 50-219

Enclosures:
1. Amendment No. 286 to DPR-16
2. Safety Evaluation

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