

February 19, 2015

MEMORANDUM TO: Edwin M. Hackett, Executive Director
Advisory Committee on Reactor Safeguards

FROM: Richard P. Correia, Director */RA/*
Division of Risk Analysis
Office of Nuclear Regulatory Research

SUBJECT: TRANSMITTAL OF NUREG-YYYY, "CONSEQUENTIAL SGTR
ANALYSIS FOR WESTINGHOUSE AND COMBUSTION
ENGINEERING PLANTS WITH THERMALLY-TREATED ALLOY
600 AND 690 STEAM GENERATOR TUBES"

I am forwarding the enclosed draft NUREG-YYYY "Consequential SGTR Analysis for Westinghouse and Combustion Engineering Plants with Thermally-Treated Alloy 600 and 690 Steam Generator Tubes". This report is publicly available.

This draft report is intended to support the April 7, 2015, Advisory Committee on Reactor Safeguards, joint meeting with the Reliability and Probabilistic Risk Assessment (PRA) and Metallurgy and Reactor Fuels Subcommittees. The report summarizes the multi-disciplinary work performed in the last few years in the Office of Nuclear Regulatory Research on severe accident-induced consequential steam generator tube rupture (C-SGTR). New conclusions and insights based on the steam generator geometry, and material properties commonly associated with two different PWR designs are provided. Assessment of potential fission product release potential due to C-SGTR following a severe accident sequence based on a Large Early Release Frequency PRA Model is also presented.

Enclosure:
As stated

Cc: A. Zoulis, NRR
K. Karwoski, NRR
E. Murphy, NRR

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301-251-7572

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