



Entergy Operations, Inc.  
1448 S.R. 333  
Russellville, AR 72802  
Tel 479-858-4704

**Stephenie L. Pyle**  
Manager, Regulatory Assurance  
Arkansas Nuclear One

1CAN011503

January 15, 2015

U.S. Nuclear Regulatory Commission  
Attn: Document Control Desk  
Washington, DC 20555

SUBJECT: License Amendment Request Supplemental  
Technical Specification Change to Extend the  
Type A Test Frequency to 15 Years  
Arkansas Nuclear One, Unit 1  
Docket No. 50-313  
License No. DPR-51

Dear Sir or Madam:

Pursuant to 10 CFR 50.90, Entergy Operations, Inc. (Entergy) requested an amendment to Arkansas Nuclear One, Unit 1 (ANO-1) Technical Specifications (TS). Specifically the proposed change would allow for the extension to the ten-year frequency of the ANO-1 Type A or Integrated Leak Rate Test (ILRT) that is required by TS 5.5.16 to be extended to 15 years on a permanent basis (Reference 1).

Due to issuance of ANO-1 TS Amendment 250 on November 24, 2014 (ML14254A133) associated with adoption of TSTF-500, "DC Electrical Rewrite – Update to TSTF-360," the footer of the TS page associated with the ILRT extension request has changed, including the page number. A revised (clean) TS page for the ILRT amendment request is included in the attachment to this letter. No technical information is change on the affected TS page.

In accordance with 10 CFR 50.91(b)(1), a copy of this application is being provided to the designated Arkansas state official.

No new regulatory commitments are included in this letter. With respect to the original Entergy request (Reference 1), changes included in this letter do not invalidate the assessment of the no significant hazards consideration included in the reference letter.

If you have any questions or require additional information, please contact me.

Sincerely,

**ORIGINAL SIGNED BY STEPHENIE L. PYLE**

SLP/dbb

REFERENCE: 1. Entergy letter to NRC, "License Amendment Request Technical Specification Change to Extend the Type A Frequency to 15 Years," dated December 20, 2013 (1CAN121302) (ML13358A195)

Attachment: Replacement Revised (Clean) Technical Specification Page

cc: Mr. Marc L. Dapas  
Regional Administrator  
U. S. Nuclear Regulatory Commission  
Region IV  
1600 East Lamar Boulevard  
Arlington, TX 76011-4511

NRC Senior Resident Inspector  
Arkansas Nuclear One  
P. O. Box 310  
London, AR 72847

U. S. Nuclear Regulatory Commission  
Attn: Ms. Andrea E. George  
MS O-8B1  
One White Flint North  
11555 Rockville Pike  
Rockville, MD 20852

Mr. Bernard R. Bevill  
Arkansas Department of Health  
Radiation Control Section  
4815 West Markham Street  
Slot #30  
Little Rock, AR 72205

**Attachment to**

**1CAN011503**

**Replacement Revised (Clean) Technical Specification Page**

5.0 ADMINISTRATIVE CONTROLS

5.5 Programs and Manuals

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5.5.16 Reactor Building Leakage Rate Testing Program

A program shall be established to implement the leakage rate testing of the reactor building as required by 10 CFR 50.54(o) and 10 CFR 50, Appendix J, Option B, as modified by approved exemptions. This program shall be in accordance with the guidelines contained in NEI 94-01, Revision 2-A, "Industry Guideline for Implementing Performance-Based Option of 10 CFR 50, Appendix J," dated October 2008. The next Type A test performed after the December 16, 2005 Type A test shall be performed no later than December 16, 2020.

In addition, the reactor building purge supply and exhaust isolation valves shall be leakage rate tested once prior to entering MODE 4 from MODE 5 if not performed within the previous 92 days.

The peak calculated reactor building internal pressure for the design basis loss of coolant accident,  $P_a$ , is 54 psig.

The maximum allowable reactor building leakage rate,  $L_a$ , shall be 0.20% of containment air weight per day at  $P_a$ .

Leakage rate acceptance criteria are:

- a. Reactor Building leakage rate acceptance criteria is  $\leq 1.0 L_a$ . During the first unit startup following each test performed in accordance with this program, the leakage rate acceptance criteria are  $< 0.60 L_a$  for the Type B and Type C tests and  $< 0.75 L_a$  for Type A tests.
- b. Air lock testing acceptance criteria are:
  1. Overall air lock leakage rate is  $\leq 0.05 L_a$  when tested at  $\geq P_a$ ;
  2. For each door, leakage rate is  $\leq 0.01 L_a$  when tested at  $\geq 10$  psig.

The provisions of SR 3.0.2 do not apply to the test frequencies specified in the Reactor Building Leakage Rate Testing Program.

The provisions of SR 3.0.3 are applicable to the Reactor Building Leakage Rate Testing Program.