

January 15, 2015

Mr. Brian H. Whitley, Director  
Regulatory Affairs  
Southern Nuclear Operating Company, Inc.  
42 Inverness Center Parkway  
BIN B237  
Birmingham, AL 35242

SUBJECT: ACCEPTANCE REVIEW OF SOUTHERN NUCLEAR OPERATING COMPANY'S  
REQUEST FOR LICENSE AMENDMENT AND EXEMPTION (LAR 15-002) FOR  
THE VOGTLE ELECTRIC GENERATING PLANT UNITS 3 AND 4: CONTROL  
ROD DRIVE MECHANISM MOTOR GENERATOR SET FIELD RELAY CHANGE  
(TAC NO. RP9504)

Dear Mr. Whitley:

By letter dated January 8, 2015 (Agencywide Documents Access and Management System (ADAMS) under Accession No. ML15008A466), Southern Nuclear Operating Company (SNC/licensee) submitted a license amendment request (LAR) to Combined License (COL) Numbers NPF-91 and NPF-92, for the Vogtle Electric Generating Plant (VEGP) Units 3 and 4, respectively. The proposed license amendment specifies the use of Control Rod Drive Mechanism (CRDM) latching control relays in lieu of field breakers to de-energize the CRDM motor generator set generator field on a diverse actuation signal.

SNC has also requested an exemption from the provisions of Title 10 of the *Code of Federal Regulations* (10 CFR) Part 52, Appendix D, Section III.B, "Design Certification Rule for the AP1000 Design, Scope and Contents," to allow a departure from the elements of the certification information in Tier 1 of the generic design control document.

The purpose of this letter is to provide the results of the U.S. Nuclear Regulatory Commission (NRC) staff's acceptance review of the LAR. The acceptance review was performed to determine if there is sufficient technical information in scope and depth to allow the NRC staff to complete its detailed technical reviews. The acceptance review is also intended to identify whether the application has any readily apparent information insufficiencies in its characterization of the regulatory requirements or the licensing basis of the plant.

B. H. Whitley

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Consistent with Section 50.90 of the 10 CFR, an amendment to the license must fully describe the changes requested, and follow as far as applicable, the form prescribed for original applications. Section 52.79 of the 10 CFR addresses the content of technical information required. This section stipulates that the submittal address the design and operating characteristics, unusual or novel design features, and principal safety considerations.

The NRC staff has reviewed your application and concluded that it does provide technical information in sufficient detail to enable the NRC staff to complete its detailed technical review and make an independent assessment regarding the acceptability of the proposed amendment in terms of regulatory requirements and the protection of public health and safety and the environment.

Given the lesser scope and depth of the acceptance review as compared to the detailed technical review, there may be instances in which issues that impact the NRC staff's ability to complete the detailed technical review are identified despite completion of an adequate acceptance review. You will be advised of any further information needed to support the NRC staff's detailed technical review by separate correspondence.

If you have any questions, please contact me at (301) 415-7871 or [Michael.Takacs@nrc.gov](mailto:Michael.Takacs@nrc.gov).

Sincerely,

*/RA/*

Michael Takacs, Project Manager  
Licensing Branch 1  
Division of New Reactor Licensing  
Office of New Reactors

Docket Nos.: 52-025  
52-026

cc: See next page

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Sincerely,

*/RA/*

Michael Takacs, Project Manager  
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 Office of New Reactors

Docket Nos.: 52-025  
 52-026

cc: See next page

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(Revised 12/02/2014)

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